

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 11, 2013

Mr. Thomas D. Gatlin Vice President, Nuclear Operations South Carolina Electric & Gas Company Virgil C. Summer Nuclear Station Post Office Box 88, Mail Code 800 Jenkinsville, SC 29065

SUBJECT: VIRGIL C. SUMMER NUCLEAR STATION, UNIT NO. 1 - REQUEST FOR

ADDITIONAL INFORMATION CONCERNING RELIEF REQUEST NO. RR-III-10

(TAC NO. MF1848)

Dear Mr. Gatlin:

By letter dated May 6, 2013, South Carolina Electric & Gas Company, (SCE&G, the licensee) for Virgil C. Summer Nuclear Station, Unit 1 (VCSNS) submitted Relief Request No. RR-III-10. In response to a request for additional information, the licensee submitted a supplement dated October 11, 2013.

The staff has determined that additional information is necessary to complete the review of Relief Request No. RR-III-10, as stated in the Enclosure. Please respond by January 24, 2014.

Sincerely

Shawn Williams, Senior Project Manager

Shaw Williams

Plant Licensing Branch II-1

Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket No. 50-395

Enclosure:

Request for Additional Information

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION REGARDING

VIRGIL C. SUMMER NUCLEAR STATION UNIT 1,

SOUTH CAROLINA ELECTRIC & GAS CO.

RELIEF REQUEST NO. RR-III-10

DOCKET NO. 50-395 (TAC NO. MF1848)

The U.S. Nuclear Regulatory Commission (NRC) staff has reviewed the information provided by South Carolina Electric & Gas Co. (SCE&G, the licensee) for Virgil C. Summer Nuclear Station, Unit 1 (VCSNS) Relief Request No. RR-III-10 in its letter dated May 6, 2013 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML13129A178), as supplement dated October 11, 2013 (ADAMS Accession No. ML13289A183) and has determined that additional information is necessary to complete the review of Relief Request No. RR-III-10.

Regulatory Basis:

10 CFR 50.55a(g) requires the use of ASME Code, Section XI for in-service inspection (ISI) examinations. The relevant requirements of the ASME Code, Section XI for the pressurizer surge nozzle examination are that essentially 100% of the nozzle-to-vessel shell weld be examined. Code Case N-460, which is approved for use by the NRC in Regulatory Guide 1.147, Revision 16, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1," clarifies "essentially 100 percent" to be greater than 90 percent coverage of the examination volume, or surface area, as applicable. Since the licensee's examination of the pressurizer surge nozzle-to-vessel shell weld only obtained 51% coverage, the NRC needs to understand clearly how that coverage was achieved in order to satisfy the requirements of 10 CFR 50.55a(a)(3), which states, in part, that alternatives to the requirements of paragraph (g) may be used, when authorized by the NRC, if the applicant demonstrates that: (i) the proposed alternatives would provide an acceptable level of quality and safety, or (ii) compliance with the specified requirements would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety.

Follow-up RAI 6.c response on October 11, 2013

The licensee's response to RAI6(c) did not provide the specifics necessary for the NRC to fully understand how the value of 51% coverage was determined. Describe how the value of 51% was determined. Provide the calculation and an illustration as necessary for clarity. From the information previously provided, it appears that the coverage from 45° and 60° probes should be different, and coverage may also be different from each scan direction, especially in light of the fact that the vendor appears to be taking credit for capturing additional coverage from performing the examination in the gaps between the heater penetrations on the head side. Specifically, describe how coverage from each of the transducers and scan directions were credited in determining the 51% coverage value. Address whether credit may be given for one angle only, or if coverage with both angles is required for credit.

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Shawn Williams, Senior Project Manager Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

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