Virginia Electric and Power Company North Anna Power Station 1022 Haley Drive Mineral, Virginia 23117

November 21, 2013

Attention: Document Control Desk U. S. Nuclear Regulatory Commission Washington, DC 20555-0001 Serial No.:13-574NAPS:MPWDocket No.:50-338License No.:NPF-4

Dear Sirs:

Pursuant to 10CFR50.73, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Power Station Unit 1.

Report No. 50-338/2013-001-00

This report has been reviewed by the Facility Safety Review Committee and will be forwarded to the Management Safety Review Committee for its review.

Sincerely,

Gerald T. Bischof Site Vice President North Anna Power Station

Enclosure

Commitments contained in this letter: None

cc: United States Nuclear Regulatory Commission Region II

Marquis One Tower 245 Peachtree Center Ave., NE, Suite 1200 Atlanta, Georgia 30303-1257

NRC Senior Resident Inspector North Anna Power Station



NRC FOR	M 366	. :	U.S. I	NUCLEAR REG	ULATO	ORY COMM	ISSION	APPR		NO 3150-0104		EXPIRES	10/31/2013
U.S. NUCLEAR REGULATORY COMMISSION (10-2010) LICENSEE EVENT REPORT (LER) (See reverse for required number of digits/characters for each block)						Estima reques licensi estima Comm infoco and R Budge collect not co	APPROVED BY OMB NO. 3150-0104 EXPIRES: 10/31/2013 Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202 (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.						
1. FACILIT				·					CKET NUMB	ER		3. PAG	_
North A	Anna I	Power	Statior	n, Unit 1				050	05000 338 1 OF 4				
	4. TITLE Unit 1 Emergency Diesel Generators Inoperable During Core Alterations												
	ENT DA			LER NUMBER				<u> </u>		8. OTHER FAC			
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NA				T NUMBER
09	26	2013	2013	001	00	11	21	2013	FACILITY NA	ME		DOCUMEN 05000	TNUMBER
10. POW	9. OPERATING MODE 11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check all that apply) 20.2201(b) 20.2203(a)(3)(i) 50.73(a)(2)(i)(C) 50.73(a)(2)(vii) 20.2201(d) 20.2203(a)(3)(i) 50.73(a)(2)(i)(A) 50.73(a)(2)(vii)(A) 6 20.2203(a)(1) 20.2203(a)(3)(i) 50.73(a)(2)(ii)(B) 50.73(a)(2)(vii)(A) 6 20.2203(a)(2)(i) 50.36(c)(1)(i)(A) 50.73(a)(2)(ii)(B) 50.73(a)(2)(vii)(B) 70.2203(a)(2)(ii) 50.36(c)(1)(ii)(A) 50.73(a)(2)(iv)(A) 50.73(a)(2)(ix)(A) 10. POWER LEVEL 20.2203(a)(2)(ii) 50.36(c)(1)(ii)(A) 50.73(a)(2)(iv)(A) 50.73(a)(2)(x)(A) 10. POWER LEVEL 20.2203(a)(2)(iii) 50.36(c)(2) 50.73(a)(2)(vi)(A) 50.73(a)(2)(x)(A) 20.2203(a)(2)(iii) 50.36(c)(2) 50.73(a)(2)(v)(A) 73.71(a)(4) 20.2203(a)(2)(iv) 50.73(a)(2)(i)(A) 50.73(a)(2)(v)(C) OTHER 0 20.2203(a)(2)(vi) 50.73(a)(2)(i)(B) 50.73(a)(2)(v)(D) Specify in Abstract below												
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CAUSE	SYSTE	м сом	PONENT	MANU- FACTURER	REPOR TO E		CAUS	Е 	SYSTEM	COMPONENT	MANU- FACTUREI		ORTABLE DEPIX
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ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) On September 26, 2013, at 1717 hours with Unit 1 in Mode 6, zero percent power, core alterations began with two inoperable emergency diesel generators. One emergency diesel generator (1J EDG), in a two train system, was out of service for maintenance with the second emergency diesel generator (1H EDG) inoperable, but unknown at the time. A subsequent failure of the 1H EDG during a 24 hour run, parallel to the grid, determined prior inoperability. Technical Specifications 3.8.2 requires one operable qualified circuit between the offsite transmission network and the onsite Class 1E AC electrical power distribution subsystem(s) required by TS 3.8.10 and one OPERABLE EDG. A cause evaluation is currently being finalized to determine the apparent cause of the 1H EDG inoperability. This event is reportable per 10 CFR 50.73(a)(2)(i)(B) for a condition prohibited by Technical Specifications. One qualified circuit between the offsite transmission network and the onsite													
Specifications. One qualified circuit between the offsite transmission network and the onsite Class 1E AC electrical power distribution subsystem(s), required by TS 3.8.10, was OPERABLE and the station blackout EDG was also available. Additionally, had a loss of offsite power occurred, the 1H EDG would have been able to supply power in the isochronous mode to the emergency bus. Therefore, the health and safety of the public were not affected by the event.													

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NRC FORM 366A (10-2010)	U.S. NUCLEAR REGULATORY COMMISSION					
LICENSEE EVENT REPORT (LER) CONTINUATION SHEET						
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		YEAR SEQUENTIAL REV NUMBER NO.				
NORTH ANNA POWER STATION UNIT 1	05000 - 338	2013001 00	2 OF 4			

NARRATIVE

1.0 DESCRIPTION OF THE EVENT

On September 26, 2013, at 1717 hours with Unit 1 in Mode 6, zero percent power, a condition prohibited by Technical Specifications (TS) occurred when reactor core alterations began with one emergency diesel generator (1J EDG), in a two train system, out of service for maintenance with the second emergency diesel generator (1H EDG) (EIIS System EK, Component DG) inoperable, but unknown at the time. A subsequent failure of the 1H EDG determined prior inoperability. Technical Specifications 3.8.2 requires one qualified circuit between the offsite transmission network and the onsite Class 1E AC electrical power distribution subsystem(s) required by TS 3.8.10 and one OPERABLE EDG.

On September 24, 2013 the 1H EDG was returned to service and declared operable following post maintenance testing (PMT) for a design change modification of the governor control system. Parts installed by the Design Change included a 2301A Load Sharing and Speed Control, a Digital Reference Unit (DRU) and an EGB-13P Governor/Actuator.

On September 24, 2013 the 1J EDG was removed from service for maintenance. Reactor core reload commenced on September 26, 2013 and completed on September 28, 2013. During this time the 1H EDG was relied upon as the OPERABLE EDG to meet TS. Subsequently on October 3, 2013 at 1115 hours, the 1J EDG was declared operable following repairs and PMT.

On October 5, 2015 at 0828 hours, the 1H EDG was started for a twenty four hour periodic test run. On October 6, 2013 at 0006 hours, 1H EDG experienced a loss of load with local indication showing the fuel racks moved to a lower fuel output. The 1H EDG was secured and declared inoperable.

The failure of the governor control system occurred with the 1H EDG parallel to the grid under 24-hour load testing. Assuming a complete and permanent loss of control power to the 2301A Electronic Governor and DRU, the mechanical governor would have operated at the high speed stop at 63 Hz per design.

However, a momentary loss of power to the Electronic Governor and DRU, due an internal issue in the DRU electronics, a loose wiring connection, or a loose fuse holder would have resulted in the DRU resetting to 900 RPM / 60 Hz set point. The 900 RPM / 60 Hz set point is the reference value used in isochronous mode of operation (design basis function for the EDG).

A complete loss of power or momentary power interruption to the 2301A and DRU would not have prevented the 1H EDG from supplying emergency electrical power during a loss of offsite power to the emergency bus in the isochronous mode. However, the 1H EDG was unable to fulfill its design function since the 1H EDG could not meet TS Surveillance Requirement 3.8.1.15. Synchronization with the offsite power source while loaded with emergency loads upon a simulated restoration of offsite power and the ability to transfer

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loads to offsite power could not be performed to satisfy the TS surveillance requirement.

2.0 SIGNIFICANT SAFETY CONSEQUENCES AND IMPLICATIONS

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No significant safety consequences resulted from this event since one OPERABLE qualified circuit between the offsite transmission network and the onsite Class 1E AC electrical power distribution subsystem(s), required by TS 3.8.10, was available and the station blackout EDG was also available to supply power if required. Additionally, had a loss of offsite power occurred, the 1H EDG would have been able to supply power to the emergency bus. As such, the health and safety of the public were not affected by the event.

This event is reportable per 10 CFR 50.73(a)(2)(i)(B) for a condition prohibited by Technical Specifications.

3.0 <u>CAUSE</u>

A cause evaluation is currently being finalized to determine the apparent cause of the 1H EDG inoperability.

4.0 IMMEDIATE CORRECTIVE ACTION(S)

Following the loss of load the 1H EDG was tripped from the Control Room.

5.0 ADDITIONAL CORRECTIVE ACTIONS

The 1H EDG mechanical actuator, DRU, and power supply fuse holder for fuse FU1NA were subsequently replaced. All wiring connections were verified tight. Post maintenance testing was performed under 1-PT-82.12H, 1H Diesel Generator Isochronous Mode (Start by ESF Actuation) and 1-PT-83.12H, 1H Diesel Generator Test (Start by ESF Actuation) followed by 24-hour run and hot restart test. Post maintenance testing was completed satisfactorily.

The mechanical governor and DRU removed from the 1H EDG were sent to a vendor for testing. The mechanical governor and DRU were found to operate properly following 24-hour bench testing. A loose power supply fuse holder (fuse holder for FU1NA) to the 2301A Load Sharing and Speed Control and DRU was also identified. Additional testing is ongoing at the vendor to validate a momentary loss of power scenario.

6.0 ACTIONS TO PREVENT RECURRENCE

Actions to prevent recurrence will be developed upon completion of the apparent cause evaluation.

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LICENSEE EVENT REPORT (LER)

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7.0 SIMILAR EVENTS

None.

8.0 ADDITIONAL INFORMATION

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Unit 2 was operating in Mode 1, 100 percent power, and was not affected by this event.

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