

**NRC MONTHLY OPERATING REPORT**

DOCKET NO. 50-206  
 UNIT SONGS - 1  
 DATE November 15, 1988  
 COMPLETED BY E. R. Siacor

TELEPHONE (714) 368-6223

OPERATING STATUS

1. Unit Name: San Onofre Nuclear Generating Station, Unit 1
2. Reporting Period: October 1988
3. Licensed Thermal Power (Mwt): 1347
4. Nameplate Rating (Gross MWe): 456
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436
8. If Changes Occur In Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: NA

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9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons For Restrictions, If Any: Self-imposed power level limit to control steam generator tube corrosion.

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>745.00</u>	<u>7,320.00</u>	<u>187,424.00</u>
12. Number Of Hours Reactor Was Critical	<u>745.00</u>	<u>3,167.84</u>	<u>109,629.36</u>
13. Reactor Reserve Shutdown Hours	<u>0.00</u>	<u>0.00</u>	<u>0.00</u>
14. Hours Generator On-Line	<u>745.00</u>	<u>3,154.08</u>	<u>105,580.77</u>
15. Unit Reserve Shutdown Hours	<u>0.00</u>	<u>0.00</u>	<u>0.00</u>
16. Gross Thermal Energy Generated (MWH)	<u>918,555.00</u>	<u>3,757,695.70</u>	<u>132,886,663.75</u>
17. Gross Electrical Energy Generated (MWH)	<u>293,400.00</u>	<u>1,219,200.00</u>	<u>44,896,928.42</u>
18. Net Electrical Energy Generated (MWH)	<u>275,734.00</u>	<u>1,134,711.00</u>	<u>42,357,374.00</u>
19. Unit Service Factor	<u>100.00%</u>	<u>43.09%</u>	<u>56.33%</u>
20. Unit Availability Factor	<u>100.00%</u>	<u>43.09%</u>	<u>56.33%</u>
21. Unit Capacity Factor (Using MDC Net)	<u>84.89%</u>	<u>35.55%</u>	<u>51.83%</u>
22. Unit Capacity Factor (Using DER Net)	<u>84.89%</u>	<u>35.55%</u>	<u>51.83%</u>
23. Unit Forced Outage Rate	<u>0.00%</u>	<u>0.00%</u>	<u>19.60%</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):	<u>Cycle 10 refueling outage scheduled to start November 27, 1988 - and scheduled for a duration of 89 days.</u>		

25. If Shutdown At End Of Report Period, Estimated Date of Startup: NA
26. Units In Test Status (Prior To Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	<u>NA</u>	<u>NA</u>
INITIAL ELECTRICITY	<u>NA</u>	<u>NA</u>
COMMERCIAL OPERATION	<u>NA</u>	<u>NA</u>

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AVERAGE DAILY UNIT POWER LEVEL

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DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>372.50</u>	17	<u>374.46</u>
2	<u>373.79</u>	18	<u>372.88</u>
3	<u>374.75</u>	19	<u>373.04</u>
4	<u>374.79</u>	20	<u>374.13</u>
5	<u>374.67</u>	21	<u>374.13</u>
6	<u>374.38</u>	22	<u>224.75</u>
7	<u>374.79</u>	23	<u>356.71</u>
8	<u>375.42</u>	24	<u>377.21</u>
9	<u>376.13</u>	25	<u>376.29</u>
10	<u>374.96</u>	26	<u>377.08</u>
11	<u>373.08</u>	27	<u>376.58</u>
12	<u>373.58</u>	28	<u>379.67</u>
13	<u>372.79</u>	29	<u>398.96</u>
14	<u>372.92</u>	30	<u>382.63</u>
15	<u>374.92</u>	31	<u>382.21</u>
16	<u>374.75</u>		

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH OCTOBER 1988

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No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	LER No.	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
123	881021	S	0.00	B	5	NA	KE SB	SHV	COND Power reduction of 20% or more to perform condensate water box cleaning and turbine stop valve testing.

<sup>1</sup>F-Forced  
S-Scheduled

<sup>2</sup>Reason:  
 A-Equipment Failure (Explain)  
 B-Maintenance or Test  
 C-Refueling  
 D-Regulatory Restriction  
 E-Operator Training & License Examination  
 F-Administrative  
 G-Operational Error (Explain)  
 H-Other (Explain)

<sup>3</sup>Method:  
 1-Manual  
 2-Manual Scram.  
 3-Automatic Scram.  
 4-Continuation from  
 Previous Month  
 5-Reduction of 20%  
 or greater in the  
 past 24 hours  
 6-Other (Explain)

<sup>4</sup>IEEE Std 805-1984

<sup>5</sup>IEEE Std 803A-1983

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

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<u>Date</u>	<u>Time</u>	<u>Event</u>
October 1	0001	Unit is in Mode 1 at 92% reactor power. Turbine load at 397 MWe.
October 21	2230	Commence power reduction to 160 MWe for circulating water system heat treat, turbine stop valve testing, water box cleaning, and reactor low flow quarterly surveillance.
October 22	1530	Unit load is at 160 MWe.
	1800	Water box cleaning, turbine stop valve testing, circulating system heat treat and reactor low flow testing completed. Commence power increase to 92% reactor power.
October 23	1221	Unit is at 92% reactor power, turbine load is at 402 MWe.
October 31	2400	Unit is in Mode 1 at 92% reactor power. Turbine load is at 406 MWe.

## REFUELING INFORMATION

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1. Scheduled date for next refueling shutdown.

November 27, 1988

2. Scheduled date for restart following refueling.

February 24, 1989

3. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Yes.

What will these be?

Proposed Technical Specification Changes are as follows:

- PCN 180 - Revises the technical specifications to be consistent with the planned replacement of the Nuclear Instrumentation System.
- PCN 181 - Revises the license to allow the shipment of spent fuel with the turbine gantry crane and deletes the requirement to perform a turbine deck load test.
- PCN 182 - Revises TS 4.16, "Inservice Inspection of Steam Generator Tubing" to include an alternate tube plugging criteria.
- PCN 184 - Revises the demonstration of the CEA trip ability to 7 days prior to reducing shutdown margin below limits.
- PCN 185 - Revises Appendix A TS 2.1 & 3.5.1 to include the revised steam/feedwater flow mismatch trip setpoints and associated Limiting Conditions for Operations.
- PCN 186 - Revises TS 4.2, "Safety Injection and Containment Spray System" periodic testing to include surveillance of a planned modification to add Safety Injection/Feedwater Pump trip on low-low Refueling Water Storage Tank (RWST) level.
- PCN 194 - Modifies the positive isolation barrier requirements in TS 3.3.2 due to modifications to include MOV-358 onto the Uninterruptible Power Supply (UPS) for MOV-850C.
- PCN 197 - Increases load limit to 6,000 KW for each of the two Emergency Diesel Generators.

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4. Scheduled date for submitting proposed licensing action and supporting information.  
  
PCN 180 - submitted to the NRC on April 15, 1988  
PCN 181 - submitted to the NRC on April 28, 1988  
PCN 182 - submitted to the NRC on 03/10/88  
PCN 184 - scheduled for submittal to the NRC by mid-November 1988  
PCN 185 - scheduled for submittal to the NRC by mid-November 1988  
PCN 186 - scheduled for submittal to the NRC by mid-November 1988  
PCN 194 - submitted to the NRC on November 7, 1988  
PCN 197 - scheduled for submittal to the NRC by mid-November 1988
5. Important Licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.  
  
None
6. The number of fuel assemblies.
  - a) In the core. 157
  - b) In the spent fuel storage pool. 146
7. Licensed spent fuel storage capacity. 216  
  
Intended change in spent fuel storage capacity. Under review
8. Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.  
  
12/88, Cycle 10 refueling outage. Transshipment program will enable continued operation of Unit 1 beyond 12/88. A minimum of 139 fuel assemblies will be transferred during the cycle 10 refueling outage which will support the planned cycle 11 reactor vessel ISI program.



**Southern California Edison Company**

SAN ONOFRE NUCLEAR GENERATING STATION

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H. E. MORGAN  
STATION MANAGER

November 15, 1988

TELEPHONE  
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U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: Docket No. 50-206  
Monthly Operating Report for October 1988  
San Onofre Nuclear Generating Station, Unit 1

Enclosed is the Monthly Operating Report as required by Section 6.9.1.10 of Appendix A, Technical Specifications to Provisional Operating License DPR-13 for San Onofre Nuclear Generating Station, Unit 1.

Please contact us if we can be of further assistance.

Sincerely,

Enclosures

cc: J. B. Martin (Regional Administrator, USNRC Region V)  
F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)  
Institute of Nuclear Power Operations (INPO)

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