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April 12, 1990

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-206
Monthly Operating Report for March 1990
San Onofre Nuclear Generating Station, Unit 1

This letter provides the Monthly Operating Report required by Section 6.9.1.10 of Appendix A, Technical Specifications (TS) to Provisional Operating License DPR-13 for San Onofre Nuclear Generating Station, Unit 1. The report covers routine operating statistics and shutdown experience for the reporting period.

TS Section 6.9.1.10 also requires reporting all challenges to the pressurizer safety and relief valves. Unit 1 did not experience a challenge to any of these valves during the reporting period.

If you require additional information, please let me know.

Very truly yours,

Enclosures

cc: J. B. Martin (Regional Administrator, USNRC Region V)
C. W. Caldwell (USNRC Senior Resident Inspector, Units 1, 2 and 3)
Institute of Nuclear Power Operations (INPO)

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NRC MONTHLY OPERATING REPORT

DOCKET NO: 50-206
 UNIT NAME: SONGS - 1
 DATE: 4/12/90
 COMPLETED BY: E. R. Siacor
 TELEPHONE: (714) 368-6223

OPERATING STATUS

1. Unit Name: San Onofre Nuclear Generating Station, Unit 1
 2. Reporting Period: March 1990
 3. Licensed Thermal Power (MWt): 1347
 4. Nameplate Rating (Gross MWe): 456
 5. Design Electrical Rating (Net MWe): 436
 6. Maximum Dependable Capacity (Gross MWe): 456
 7. Maximum Dependable Capacity (Net MWe): 436
 8. If Changes Occur In Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: NA
-
9. Power Level To Which Restricted, If Any (Net MWe): 390
 10. Reasons For Restrictions, If Any: Self-imposed power level limit to control Steam Generator tube corrosion

| | This Month | Yr.-to-Date | Cumulative |
|--|--|--------------|----------------|
| 11. Hours In Reporting Period | 744.00 | 2,160.00 | 199,808.00 |
| 12. Number Of Hours Reactor Was Critical | 744.00 | 2,160.00 | 116,021.84 |
| 13. Reactor Reserve Shutdown Hours | 0.00 | 0.00 | 0.00 |
| 14. Hours Generator On-Line | 744.00 | 2,160.00 | 111,839.48 |
| 15. Unit Reserve Shutdown Hours | 0.00 | 0.00 | 0.00 |
| 16. Gross Thermal Energy Generated (MWH) | 910,014.05 | 2,623,357.63 | 140,193,204.75 |
| 17. Gross Electrical Energy Generated (MWH) | 300,000.00 | 863,400.00 | 47,283,728.42 |
| 18. Net Electrical Energy Generated (MWH) | 283,415.00 | 815,618.00 | 44,578,038.00 |
| 19. Unit Service Factor | 100.00% | 100.00% | 55.97% |
| 20. Unit Availability Factor | 100.00% | 100.00% | 55.97% |
| 21. Unit Capacity Factor (Using MDC Net) | 87.37% | 86.61% | 51.17% |
| 22. Unit Capacity Factor (Using DER Net) | 87.37% | 86.61% | 51.17% |
| 23. Unit Forced Outage Rate | 0.00% | 0.00% | 19.51% |
| 24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): | <u>Cycle 11 refueling outage scheduled to commence on June 30, 1990.</u> | | |

25. If Shutdown At End Of Report Period, Estimated Date of Startup: NA
 26. Units In Test Status (Prior To Commercial Operation):
- | | Forecast | Achieved |
|----------------------|----------|----------|
| INITIAL CRITICALITY | NA | NA |
| INITIAL ELECTRICITY | NA | NA |
| COMMERCIAL OPERATION | NA | NA |

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-206
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MONTH: March 1990

| DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) | DAY | AVERAGE DAILY POWER LEVEL (MWe-Net) |
|-----|--|-----|--|
| 1 | <u>383.00</u> | 17 | <u>383.58</u> |
| 2 | <u>383.29</u> | 18 | <u>384.04</u> |
| 3 | <u>384.08</u> | 19 | <u>384.04</u> |
| 4 | <u>383.25</u> | 20 | <u>385.33</u> |
| 5 | <u>383.67</u> | 21 | <u>385.17</u> |
| 6 | <u>384.71</u> | 22 | <u>384.17</u> |
| 7 | <u>384.54</u> | 23 | <u>383.13</u> |
| 8 | <u>382.71</u> | 24 | <u>382.50</u> |
| 9 | <u>383.50</u> | 25 | <u>382.58</u> |
| 10 | <u>383.79</u> | 26 | <u>382.08</u> |
| 11 | <u>383.75</u> | 27 | <u>381.71</u> |
| 12 | <u>384.33</u> | 28 | <u>381.75</u> |
| 13 | <u>384.88</u> | 29 | <u>381.96</u> |
| 14 | <u>384.33</u> | 30 | <u>381.83</u> |
| 15 | <u>384.58</u> | 31 | <u>302.92</u> |
| 16 | <u>383.75</u> | | |

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: MARCH 1990

DOCKET NO: 50-206
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| No. | Date | Type ¹ | Duration (Hours) | Reason ² | Method of Shutting Down Reactor ³ | LER No. | System Code ⁴ | Component Code ⁵ | Cause & Corrective Action to Prevent Recurrence |
|-----|------|-------------------|------------------|---------------------|--|---------|--------------------------|-----------------------------|---|
| NA | NA | NA | NA | NA | NA | NA | NA | NA | NA |

¹F-Forced
S-Scheduled

²Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation from Previous Month
 5-Reduction of 20% or greater in the past 24 hours
 6-Other (Explain)

⁴IEEE Std 805-1984

⁵IEEE Std 803A-1983

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SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

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 DATE: 4-12-90
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| <u>Date</u> | <u>Time</u> | <u>Event</u> |
|-------------|-------------|---|
| March 1 | 0001 | Unit is in Mode 1 at 92% reactor power. Turbine load at 404 MWe gross. |
| March 31 | 0515 | Commenced reactor power decrease to approximately 50% to perform quarterly reactor coolant flow instrumentation channel test required by Technical Specification 4.1.1. In addition, circulating water heat treating, turbine stop valve testing and South condenser water box cleaning were scheduled to be performed. |
| | 0615 | Reactor at 47% power. Scheduled evolutions were initiated. |
| | 0655 | Completed turbine stop valve testing satisfactorily. |
| | 0900 | Completed quarterly reactor coolant flow instrumentation channel test satisfactorily. |
| | 1202 | Completed South condenser water box cleaning. |
| | 1226 | Reactor power increased to 68% to perform circulating water heat treat. |
| | 1352 | Commenced heat treat operations. |
| | 1756 | Commenced reactor power increase following completion of heat treat operations. |
| | 1938 | Reactor at 91% power. |
| | 2400 | Unit is in Mode 1 at 91% power. Turbine load at 402 MWe gross. |

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REFUELING INFORMATION

DOCKET NO: 50-206
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MONTH: March 1990

1. Scheduled date for next refueling shutdown.
June 30, 1990
2. Scheduled date for restart following refueling.
November 17, 1990
3. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Yes.

What will these be?

- a) License Amendment associated with the resolution of the 480V breaker overload issue.
 - b) License Amendment associated with removal of the license condition related to the TDI diesel generators.
 - * c) License Amendment associated with revision of the basis to Technical Specification 3.3.1, "Safety Injection and Containment Spray", and resolution of other issues related to TS 3.3.1 which were identified during Cycle 10 refueling.
 - ** d) License Amendment associated with the provision of an alternate intermediate spent fuel cooling capability during the upcoming Cycle 11 refueling outage and concurrent CCW outage.
- * Will not be included in next month's report - Although this license amendment will be submitted by April 30, 1990, as indicated in last month's report, re-evaluation concluded that NRC approval of the change is not required prior to refueling or resumption of operation.
- ** Will not be included in next month's report - CCW outage will not be conducted during Cycle 11 refueling.

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MONTH: March 1990

4. Scheduled date for submitting proposed licensing action and supporting information.
- a) SCE expects to submit the license amendment associated with the 480V breaker overload issue by April 30, 1990.
 - b) SCE expects to submit the request to remove the TDI diesel generator license condition by April 30, 1990, in accordance with the letter from SCE to the NRC, dated February 21, 1990.
5. Important Licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
- None have been identified at this time.
6. The number of fuel assemblies.
- a) In the core. 157
 - b) In the spent fuel storage pool. 59
7. Licensed spent fuel storage capacity. 216
- Intended change in spent fuel storage capacity. None
8. Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.
- Approximately 1995 (refueling only)
- Approximately 1991 (full off load capability)

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