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July 14, 1989

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-206
Monthly Operating Report for June 1989
San Onofre Nuclear Generating Station, Unit 1

Enclosed is the Monthly Operating Report as required by Section 6.9.1.10 of Appendix A, Technical Specifications to Provisional Operating License DPR-13 for San Onofre Nuclear Generating Station, Unit 1.

Please contact us if we can be of further assistance.

Sincerely,

Enclosures

cc: J. B. Martin (Regional Administrator, USNRC Region V)
F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
Institute of Nuclear Power Operations (INPO)

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NRC MONTHLY OPERATING REPORT

DOCKET NO: 50-206
 UNIT NAME: SONGS - 1
 DATE: July 14, 1989
 COMPLETED BY: E. R. Siacor
 TELEPHONE: (714) 368-6223

OPERATING STATUS

1. Unit Name: San Onofre Nuclear Generating Station, Unit 1
2. Reporting Period: June 1989
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 456
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436
8. If Changes Occur In Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: NA

9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons For Restrictions, If Any: Self-imposed power level limit to control Steam Generator tube corrosion

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>720.00</u>	<u>4,343.00</u>	<u>193,231.00</u>
12. Number Of Hours Reactor Was Critical	<u>77.67</u>	<u>207.70</u>	<u>110,486.89</u>
13. Reactor Reserve Shutdown Hours	<u>0.00</u>	<u>0.00</u>	<u>0.00</u>
14. Hours Generator On-Line	<u>67.48</u>	<u>113.60</u>	<u>106,344.09</u>
15. Unit Reserve Shutdown Hours	<u>0.00</u>	<u>0.00</u>	<u>0.00</u>
16. Gross Thermal Energy Generated (MWH)	<u>36,800.21</u>	<u>45,122.00</u>	<u>133,698,859.75</u>
17. Gross Electrical Energy Generated (MWH)	<u>11,400.00</u>	<u>13,200.00</u>	<u>45,165,128.42</u>
18. Net Electrical Energy Generated (MWH)	<u>6,868.00</u>	<u>(6,434.00)</u>	<u>42,589,584.00</u>
19. Unit Service Factor	<u>9.37%</u>	<u>2.62%</u>	<u>55.03%</u>
20. Unit Availability Factor	<u>9.37%</u>	<u>2.62%</u>	<u>55.03%</u>
21. Unit Capacity Factor (Using MDC Net)	<u>2.19%</u>	<u>0.00%</u>	<u>50.55%</u>
22. Unit Capacity Factor (Using DER Net)	<u>2.19%</u>	<u>0.00%</u>	<u>50.55%</u>
23. Unit Forced Outage Rate	<u>90.63%</u>	<u>87.19%</u>	<u>19.96%</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):	<u>NA</u>		

25. If Shutdown At End Of Report Period, Estimated Date of Startup: NA
26. Units In Test Status (Prior To Commercial Operation): Forecast Achieved

INITIAL CRITICALITY	<u>NA</u>	<u>NA</u>
INITIAL ELECTRICITY	<u>NA</u>	<u>NA</u>
COMMERCIAL OPERATION	<u>NA</u>	<u>NA</u>

AVERAGE DAILY UNIT POWER LEVEL

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MONTH: June 1989

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0.00	17	0.00
2	0.00	18	0.00
3	0.00	19	0.00
4	0.00	20	0.00
5	0.00	21	0.00
6	0.00	22	0.00
7	0.00	23	0.00
8	0.00	24	0.00
9	0.00	25	0.00
10	0.00	26	0.00
11	0.00	27	0.00
12	0.00	28	46.96
13	0.00	29	155.88
14	0.00	30	198.25
15	0.00	31	NA
16	0.00		

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: JUNE 1989

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No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	LER No.	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
125	890526	F	652.52	A	4	89-012	BA	LI	Reactor was manually shutdown from approximately 12-15% power to investigate and repair erroneous Steam Generator level indications due to decalibration of Auxiliary Feedwater wide range (WR) level instrumentation. A design change was implemented to repair the erroneous level indications.

¹F-Forced
S-Scheduled

²Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation from Previous Month
 5-Reduction of 20% or greater in the past 24 hours
 6-Other (Explain)

⁴IEEE Std 805-1984

⁵IEEE Std 803A-1983

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SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

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<u>Date</u>	<u>Time</u>	<u>Event</u>
June 1	0001	Unit is in Mode 5 for the Auxiliary Feedwater (AFW) level indication, Reactor Coolant Pump "B" seal, and miscellaneous maintenance repair outage.
June 24	0952	Commenced fill and pressurization of the Reactor Coolant System (RCS) following completion of maintenance repairs in preparation for Mode 4 entry.
	1801	Commenced RCS heatup.
	2359	Entered Mode 4.
June 25	1924	Entered Mode 3.
June 27	1759	Entered Mode 2.
	1820	Reactor made critical.
June 28	0413	Entered Mode 1.
	0431	Unit synchronized to the grid. Increasing reactor power to 20%.
	1526	Reactor at 20% power. AFW flow data gathering completed satisfactorily.
	1810	Reactor at 30% power for physics testing and additional AFW flow data gathering.

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<u>Date</u>	<u>Time</u>	<u>Event</u>
June 29	0230	Commenced reactor power increase for additional AFW flow data gathering following completion of satisfactory physics testing.
	0700	Reactor at 40% power for AFW data gathering and condenser water box cleaning.
June 30	0540	Reactor power increased to 70% following completion of AFW data gathering and condenser water box cleaning.
	0854	Commenced reversing circulating water tunnel flow for heat treating operations.
	0908	Returned circulating flow to normal. Heat treating operations suspended due to high influx of sea shells and sea weeds.
	2400	Unit is Mode 1 at 58% reactor power. Turbine load at 225 MWe gross.

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REFUELING INFORMATION

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MONTH: June 1989

1. Scheduled date for next refueling shutdown.
Not yet determined. Under evaluation.
2. Scheduled date for restart following refueling.
Not yet determined. Under Evaluation.
3. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?
Not yet determined. Under evaluation.
What will these be?
Not yet determined.
4. Scheduled date for submitting proposed licensing action and supporting information.
Not yet determined.
5. Important Licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.
Not yet determined. Under evaluation.
6. The number of fuel assemblies.
 - a) In the core. 157
 - b) In the spent fuel storage pool. 59

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7. Licensed spent fuel storage capacity. 216
Intended change in spent fuel storage capacity. None
8. Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.
Approximately 1995 (refueling only)
Approximately 1991 (full off load capability)

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