

Southern California Edison Company

SAN ONOFRE NUCLEAR GENERATING STATION

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April 1, 1988

U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region V
1450 Maria Lane, Suite 210
Walnut Creek, California 94596-5368

Attention: Mr. J. B. Martin, Regional Administrator

Dear Sir:

Subject: Docket No. 50-206
Special Report
San Onofre Nuclear Generating Station Unit 1

Pursuant to Section 4.16.E.4, of the Appendix A Technical Specifications to Provisional Operating License No. DPR-13 for San Onofre Unit 1, this submittal provides a report on conditions identified during Inservice Inspection of Steam Generator Tubing. It is noted that Specification 4.16.E.4 requires providing a report to the Commission in accordance with Section 6.6, Reportable Event Action. Section 6.6 requires reporting in accordance with 10 CFR 50.73, however, the rule does not require reporting the level of degradation or plugging of steam generator tubes experienced at San Onofre, Unit 1. We therefore are submitting this report in accordance with Section 6.9.2, Special Reports. Technical Specification proposed change No. 176 submitted to the NRC on August 27, 1987, will correct this administrative discrepancy. The remedial action taken is the subject of separate correspondence submitted to the NRC PWR Project Directorate on March 25, 1988.

The inservice inspection of steam generator tubing required by Technical Specification 4.16 was conducted between February 22, 1988 and March 7, 1988. A total of 7,425 tubes (72 percent of the tubes in service) were inspected and 169 tubes were removed from service by mechanical plugging. This inspection demonstrated that there has been no detectable progression of intergranular attack (IGA), denting, antivibration bar (AVB) wear or sleeving degradation.

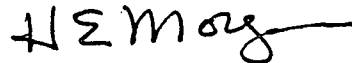
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The 169 tubes which required plugging included 147 defects in the roll transition zone on the hot leg side, 3 defects at or above the top of the hot leg tubesheet, and 19 leaking sleeves. The 147 tubes with indications in the roll transition zone were required to be plugged only due to the improved inspection and analysis techniques used. All tubes having imperfections above the plugging limit have been plugged and therefore, the remedial action taken to resolve the steam generator tube degradation identified during this inspection is appropriate.

There was no impact on plant operation or the health and safety of plant personnel or the public as a result of the steam generator tube degradation and plugging. The remedial actions taken are considered to be appropriate to resolve steam generator tube degradation identified during this inspection. As previously stated, further details regarding the above described inspection and repair have been provided to the NRC/NRR. If there are any questions regarding the above or attached, please contact me.

Sincerely,



cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)

U.S. Nuclear Regulatory Commission
Document Control Desk

C. Trammell (USNRC Unit 1 Project Manager)

Institute of Nuclear Power Operations (INPO)