

ENCLOSURE 1

SONGS 1 RCS Inventory Analysis for Station Blackout

Calculation # DC-3601

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CALCULATION TITLE PAGE

Sheet 1

SONGS Unit 1 DCP/MMP No. & Rev. N/A Calc. No. DC-3601
 Subject SONGS 1 RCS Inventory Analysis for Station Blackout
 Engineering System Number N/A Primary Station System Designator RCS Q-Class SR
 Tech Spec Effecting ☐ YES ☒ NO Section No. N/A Equipment Tag No. N/A

Computer Program	STANDARD COMPUTER PROGRAM ☒ YES ☐ NO	PROGRAM NO.(S) <u>RETRAN-02</u>	VERSION/RELEASE NO. <u>MOD 004</u>
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RECORD OF ISSUES

SCE DISC. or ESC	REV.	DESCRIPTION	TOTAL NO. of SHEETS	LAST SHEET NO.	ORIG.	IRE	GS	DM	DATE
NEDO	O-A	Issued for Comments	59	59	VFN	WMA			5/17/91
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Space for RPE Stamp, reference alternate calc., and notes as applicable.

This calc. was prepared for the identified DCP/MMP. DCP completion and turnover acceptance to be verified by receipt of a memorandum directing DCN Conversion. Upon receipt, this calc. represents the as-built condition. Memo date _____ by _____.

CALCULATION CROSS-INDEX

Subject Calculation No. DC-3601

Subject Calculation Revision No.	Superseded By Calc. No.	INPUTS These interfacing calculations and /or documents provide input to the subject calculation, and if revised may require revision of the subject calculation.		OUTPUTS Results and conclusions of the subject calculation are used in these interfacing calculations and/or documents.		Does the output interface calc/document require revision?	Identify output interface calc/document CCN or DCN TCN/Rev.	Group Supervisor or Station Technical Group Supervising Engineer Signature/Date
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0	N/A	DC-3533	0	N/A		NO	N/A	
0	N/A	EOI-501-1.0-60	9	N/A		NO	N/A	
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Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT**

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△	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	△					
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Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT**

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I. PURPOSE

10 CFR 50.63¹ requires that SCE demonstrate the ability of SONGS Unit 1 to cope with a Loss of all AC power (Station Blackout). Regulatory Guide 1.155 provides NRC guidance for addressing this situation². This guide states that the NUMARC (Nuclear Management And Resource Council) published guidelines³ to demonstrate ability to cope with a station blackout (NUMARC 87-00) is compatible with the Reg. Guide. The basic NUMARC/REG. GUIDE criteria are as follows:

- (i) procedures and equipment relied upon in station blackout should ensure that satisfactory performance of necessary decay heat removal systems is maintained for the required station blackout coping duration,
- (ii) the core must be kept covered for the coping duration, and
- (iii) appropriate containment integrity should be provided in a blackout to the extent that isolation valves perform their intended function without AC power.

The purpose of this calculation is to determine whether adequate RCS water inventory exists to perform the decay heat removal function and keep the core covered during a station blackout for the NUMARC defined coping duration.

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Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT**

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II. RESULTS/CONCLUSIONS AND RECOMMENDATIONS

A. RESULTS/CONCLUSIONS

The significant events occurring during a station blackout including operator actions are shown on Table II-1. A trend of some of the important parameters during the critical times of the coping duration are shown in Table II-2.

On initiation of the blackout, the Pressurizer begins to empty rapidly due to the RCS leakage (171 gpm (cold) for the first 2 minutes and 101 gpm (cold) thereafter) and coolant shrinkage. In 21 minutes the pressurizer has emptied. Thereafter, the Reactor vessel upper head begins to drain resulting in a steam bubble in the upper head of the vessel. In 45 minutes the vessel upper head is completely voided. The outlet plenum then begins to drain causing the steam bubble to expand into the outlet plenum. The mixture level in the outlet plenum drops to just above the top of the hot leg at the end of one hour. During this one hour period natural circulation removes the core decay heat and keeps the core covered.

At one hour after the initiation of the blackout the charging pumps when powered by the DSD will provide makeup flow to the RCS. Thereafter the RCS inventory will be maintained (or will increase) for the remaining three hours of the coping duration. Again the core decay heat continues to be removed and the core remains covered for this period.

B. RECOMMENDATIONS

1. The Technical Specification leakage allowance of 6 gpm (total identified & unidentified) remains valid for Station Blackout. No changes to the Technical Specifications are thus required.
2. The allowable letdown isolation valve leakage of 20 gpm remains acceptable.

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TABLE II-1
SEQUENCE OF MAJOR EVENTS FOR STATION BLACKOUT

<u>TIME (MINUTES)</u>	<u>EVENT/ACTION</u>
0	Loss of all on and offsite AC power accompanied by reactor and turbine trip
3	Operator takes manual control of ADV's
4.7	Auxiliary Feedwater Flow injected into the S.G.'s through turbine driven pump (G-10) to achieve and maintain S.G. level between 50% and 70% NR.
20.8	Pressurizer empty; Voiding begins in the upper Reactor head
45	Upper Reactor head completely voided; Voiding begins in the outlet plenum
60	Outlet plenum* approximately 40% voided; No voiding in the hot legs
60.1	RCS charging restored through powering north charging pump by the DSD diesel generator; RCS inventory reduction stops
240	End of coping duration

* Outlet plenum extends from the reactor head flange to the top of the core

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TABLE II-2
SIGNIFICANT PARAMETER TRENDS DURING A SBO*

TIME min	ACTIVITY/EVENT	RCS LEAKAGE lbm/sec	RCS CHARGING lbm/sec	RCS PRESSURE psia	RCS T _{cold} F	RCS T _{hot} F	RCS FLOW* % flow	RCS SUBCOOL -ING(F)
0	Steady State full power	0	0	2100	527.0	576.0	100	66.9
0.1	Reactor at 10% power	24.2	0	2062	526.5	572	68	68.2
2.1	letdown leakage reduced	14.6	0	1885	515.1	557.3	10	70.3
3.1	manual ADV control**	14.3	0	1838	515.8	539.7	7	84.3
4.7	AFW flow starts	14.2	0	1870	528.3	539.6	4.9	86.8
20.8	Pressurizer empty	14.0	0	1587	535.4	558.1	4.4	45.8
27	RCS at Saturation	13.9	0	1133	534.7	559.6	4.7	0
45	RV upper head voided	13.9	0	1073	535.3	553.1	4.4	0
55	Intermediate condition	13.9	0	1061	535.5	551.6	4.0	0
60	Outlet plenum 40% voided	13.9	0	1067	535.4	551.0	4.1	0
60.3	Charging starts	13.9	26.9***	1065	520.2	552.0	4.8	0

* from Fiche ECFUELA J1262

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CCN NO. CCN -Subject SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUTSheet No. 8TABLE II-2 (continued)
SIGNIFICANT PARAMETER TRENDS DURING A SBO*

TIME min	ACTIVITY/EVENT	S. G. SECONDARY PRESSURE psia	S. G. SECONDARY TEMP. F	PRESSUR- IZER LEVEL %	VESSEL MIXTURE HEIGHT** ft.	S.G. SECONDARY LEVEL** % NR	AUXILIARY FEEDWATER FLOW** gpm
0	Steady state full power	547	476.0	37.5	35.82	30	0
0.1	Reactor at 10% power	670	497.9	35	35.82	28.8	0
2.1	letdown leakage reduced	717	505.4	19.6	35.82	0	0
3.1	manual ADV control	856	525.6	15.9	35.82	0	0
4.7	AFW flow starts	895	530.8	17.0	35.82	0	370
20.8	Pressurizer empty	927	535.0	0	35.82	61	167.8
27	RCS at saturation	927	535.0	0	35.25	65	84.2
45	RV upper head voided	926	535.0	0	29.26	65	93.4
55	Intermediate condition	926	535.0	0	26.80	64	104.1
60	outlet plenum 40% voided	925	535.0	0	25.58	67	54.7
60.3	Charging starts	925	535.0	0	25.54	67	48.0

** Top of the core = 19.82 ft. Top of hot leg = 25.55 ft. Top of Lower head = 29.32 ft.
 * from Fiche ECFUELA J1262

** Actual AFW flow would be as little as 270 gpm at S.G. Pressure of 923 psia. The RETRAN analysis used 370 gpm during the period of S.G. recovery (15 min.) after which flow is manually throttled. Use of actual minimum flows have negligible impact on analysis results. AFW flows used are best estimate values and should not be considered design basis values.

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III. ASSUMPTIONS

A) NUMARC² BASED

1. The reactor is operating at 100% power for 100 days and at normal operating conditions at the time of the blackout.
2. All reactor support systems are within normal operating ranges.
3. The blackout does not occur during a design basis event.
4. Single failure criteria are not considered since SBO is not considered a design basis event.
5. The RCS pump seal leakage (cold) will not be greater than 25 gpm/pump.
6. The coping duration is four (4) hours⁶.
7. The initiating event is a loss of offsite power.

B) OPERATIONS BASED

1. The Nominal pressurizer level at 100% power is 37.5%. This is the expected Pressurizer level at 551.5 F (i.e. based on the reduced Tav_g program - see assumption B.2)²² (confirmed by E-mail from Operations -- see Appendix B). NUMARC permits use of the nominal pressurizer level as the initial condition of the blackout (see assumption A.1 & A.2).
2. The plant is using a Reduced Tav_g program (due to the steam generator tube corrosion and plugging)²⁰.
3. The total unidentified and identified leakage is less than 6 gpm (requirement from Section 3.1.4 of the tech. specs.⁷).

C) EOI^{4,5} BASED

1. Steam generator level will be maintained between 50% and 70% NR during the blackout.

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2. Auxiliary feedwater flow will be provided to the steam generators to maintain the steam generator level between the above limits. Auxiliary feedwater flow is provided through the turbine driven pump (G-10). The operator can throttle flow to the steam generators to maintain the Steam generators within these limits¹⁴.
3. **The DSD can be powered up to restore charging capacity within one hour after the initiation of the blackout⁶.**
4. The number of steam generator tubes plugged will be less than 20%²⁰. The safety analysis for Unit 1 cycle 11 assumes S.G. tube plugging of 20%. Therefore this assumption is used here.
5. The atmospheric dump valves are placed in manual mode to maintain plant subcooling conditions 3 minutes after initiation of the event. This is an expected operator action. However, this assumption will not compromise the conservatism of this calculation. If the manual operation of the ADV's were not credited, decay heat removal will still occur through automatic control of the ADV's and also through the Main Steam Safety Valves (MSSV).
6. The letdown leakage for the first two minutes after the blackout will be less than 90 gpm (i.e. 70 gpm greater than the leakage after blackout initiation). The 90 gpm value is the normal maximum letdown leakage (Section 9.3.4 of Reference 10). The letdown valve closure time is expected to be less than 30 seconds⁸. Therefore the assumption of a 90 gpm letdown leakage for 2 minutes since the initiation of the SBO is conservative.
7. The total letdown system leakage (cold) after the first two minutes is less than 20 gpm. This is the maximum expected leakage from the letdown orifice isolation valves after the letdown system is isolated¹⁸.
8. Control rods are fully inserted within 3 seconds after initiation of the blackout¹².
9. No other leakage resulting in RCS water inventory drain exists.
10. The reactor trip is caused by the instantaneous loss of all AC. This produces the

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loss of the turbine bypass system which causes the initial opening of the ADV's. The ADV's are air operated and have Nitrogen backup. Therefore they will continue to be operable during a blackout.

11. Control room adequacy and battery are assured to function during the entire coping duration¹⁹.
12. The minimum Auxiliary Feedwater flow delivered to all three steam generators at a secondary side pressure of approximately 923 psia, when driven by pump G-10 will be at least 270 gpm. The actual minimum deliverable flow is expected to be between 288 gpm and 300 gpm. Therefore, it is conservative to assume a minimum deliverable flow of 270 gpm.

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IV. DESIGN INPUT

Table IV-1 shows the design inputs used to determine the RCS inventory during the station blackout. The auxiliary feedwater flow that will be automatically delivered to each S.G. by means of the steam driven auxiliary feedwater pump (G10) is shown in Table IV-2. Some of the significant external inputs that require further explanation are described as follows:

- (i) RCS Leakage -- NUMARC 87-00 suggests using the assumption of leakage rates of 25 gpm per pump seal (i.e. total seal leakage of 75 gpm). The Unit 1 technical specification 3.1.4 permits a maximum allowable unidentified and identified leakage of 6 gpm. The letdown leakage limit is assumed to be 90 gpm until the letdown isolation valves are closed (i.e. for the first 2 minutes of the blackout) and subsequently 20 gpm for the remaining duration of the transient. Therefore the total leakage assumed is 171 gpm for the first 2 minutes of the blackout and 101 gpm for the rest of the coping duration. On a loss of all ac power, the letdown system is automatically isolated. This occurs by closure of valves CV-202, CV-203 & CV-204 along with CV-525 and CV-526 (See note in letdown system drawing²¹). The closure time for the letdown isolation valves is less than 30 seconds⁸. For conservatism the closure time of 2 minutes is used in this calculation.
- (ii) Auxiliary Feedwater -- After initiation of the blackout, main feedwater is not available as a decay heat removal system. Auxiliary feedwater however is available since it is driven by the steam driven auxiliary feedwater pump (G-10). AFW flow is automatically initiated when the SG level reaches a setpoint value of 5% NR (Table 5.5-3 of Reference 10). It is then assumed that the delay time for the pump to be available to deliver flow into the S.G.'s is 3 minutes (Section 6.5.2.2.3 of Reference 10). Thereafter once the S.G. level has reached between 50% and 70% NR, the operator manually throttles the AFW flow to maintain this level in the

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Sheet No. **13**

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S.G.'s⁴. The philosophy employed here is to maintain temperature and Pressure control in the Secondary side and consequently in the primary side and thus continue to have this system available for decay heat removal.

- (iii) Charging Flow -- After one hour the charging pump when powered by the DSD restores charging flow to the Reactor. From Table II-2, the RCS pressure at this time will be 1067 psia. The charging flow capable of being delivered at this pressure will be at least 198.5 gpm⁹. However for this calculation a conservative value of 150 gpm charging flow is used*. This minimum charging flow will be maintained from 1 hour to the end of the coping duration (i.e. 4 hours).

* Note that in this analysis it is only important that the charging flow delivered be greater than 101 gpm so that the RCS water that is lost due to the leakage can be made up.

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**TABLE IV-1
DESIGN INPUT FOR STATION BLACKOUT**

A. INITIAL CONDITIONS

#	PARAMETER	VALUE	REFERENCE	COMMENTS
1.	Core Power	1347 MW	10	100% Rated Power
2.	Core Average Temp.	551.5 F	11	At 100% power
3.	RCS Pressure	2100 psia	10	
4.	Pressurizer level	37.5%	--	Nominal level
5.	S.G. Pressure	547 psia	11	at 100% power
6.	S.G. Mass	43,000 lbm	17	per S.G.
7.	S.G. Tubes plugged	20%	11,20	
8.	S.G. Level	30% NR	--	Nominal
9.	Vessel Flow Rate	195,000 gpm	10,20	
10.	Vessel mass flow	73*10 ⁶ lb/h	10	
11.	Core mass flow	69.7*10 ⁶ lb/h	10	
12.	Core bypass flow	4.5%	10	
13.	Heat generated in Fuel	97.4%	10	
14.	No load Ref. Temp.	535 F	11	

B. AFTER BLACKOUT INITIATION

#	PARAMETER	VALUE	REFERENCE	COMMENTS
1.	RCS leakage for initial 2 min.	171 gpm	--	see Section IV discussion
2.	RCS Leakage after initial 2 min.	101 gpm	--	see Section IV discussion
3.	S.G. NR Level	50%-70%	4	Manually maintained
4.	Auxiliary Feedwater Auto. initiation	5% NR	10	see Section IV discussion
5.	Aux. Feedwater Auto initiation delay	3 min.	10	see Section IV discussion
6.	ADV manual initiation time delay	3 min.	4,5	Assumption
7.	ADV relief Capacity	1.468x10 ⁶ lbm/hr	12	
8.	RCP flywheel inertia	45,000 lbm/ft ²	12	
9.	Charging Initiation	1 hour	--	See Section IV discussion
10.	Charging flow	150 gpm	--	See Section IV discussion

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TABLE IV-2
AUXILIARY FEEDWATER FLOW FOR STATION BLACKOUT¹⁶

PUMP DISCHARGE PRESSURE
(Psia)

AFW FLOW*
(Gpm)

0.0
995.0
1030.0
1055.0
1125.0
1155.0
1185.0
1240.0

123.3
123.3
115.0
108.3
88.3
64.3
50.0
33.3

- * The AFW flow shown in this table was derived from Figure 2-1 of Reference 16 and used in the RETRAN case. However, the actual flow expected to be delivered to each S.G. through the steam driven AFW pump G-10 will be at least 90 gpm. This will not significantly affect the results of this calculation since reduced AFW flow has a negligible effect on RCS water inventory change and the reduced flow remains sufficient to meet decay heat removal requirements. The lower initial AFW flow will slightly delay the recovery of S.G. secondary side level relative to the RETRAN results. However, once the S.G. level has reached 50% NR, the operator will throttle the AFW flow to maintain S.G. level. The secondary side results will then be consistent with the RETRAN values. For these reasons the lower AFW flow will not impact the RCS water inventory level nor will it significantly impact the ability of the system to remove core decay heat. AFW flows used are best estimate values and should not be considered design basis values.

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V. METHODOLOGY

A. GENERAL

The RCS Inventory Analysis is performed based on the expected operator actions during a blackout as enumerated in EOI S01-1.0-60 and the expected Plant responses described in the Background Document for this EOI⁵. The simulation of the blackout and consequent plant actions and response is modeled using the RETRAN computer code. The results of the computer code will then be verified by a simplified hand calculation.

B. RETRAN CODE MODEL

The basic RETRAN model for Station Blackout was developed for the analysis performed to demonstrate the adequacy of the leakage technical specification (T.S. 3.1.4) and in response to LER 90-004¹⁸. A complete description of the input model used for this LER analysis is documented in Reference 12. The RETRAN nodal diagram describing the control volumes and junctions used in this analysis is shown in figure V-1. The code control systems for auxiliary feedwater flow and steam bypass are described in Figures V-2 and V-3.

The modifications made to the basic model¹² for this analysis are shown on Table V-1. The primary modifications along with the reasons for the changes are described as follows:

1. RCS Leakage -- The base model had the total RCS leakage only from one loop (i.e. volume 18). This approach is acceptable until such a time as the Pressurizer is drained. Thereafter the imbalance in the leakage appears to affect the convergence of the code. Therefore, a new junction (junction 51) was added for leakage from the other loops (i.e. volume 36). The amount of leakage through junction 50 will thus be reduced to one third of the total leakage and through junction 51 will be two thirds of the total leakage (since loops A & C are modeled

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together as one loop). The HOT leakage is determined as follows:

$$L_{hot} = L_{cold} * (d_{cold}/d_{hot})$$

where,

$$L_{hot} = \text{Hot leakage (gpm)}$$

$$L_{cold} = \text{Cold leakage (gpm)}$$

$$d_{hot} = \text{density of water at 535 F} \\ = 47.7 \text{ lbm/ft}^3$$

$$d_{cold} = \text{density of water at 80 F} \\ = 62.6 \text{ lbm/ft}^3$$

The HOT leakage per RCS loop thus becomes 74.67 gpm ($171 * 1.31/3$) for the first 2 minutes of the SBO and 44.1 gpm ($101 * 1.31/3$) thereafter for the coping duration of 4 hours.

2. Charging -- The base model does not account for charging flow into the RCS system one hour after the SBO initiation. Therefore two extra junctions were added to provide this function (junctions 48 and 49). The amount of charging through these junctions will be one third through junction 48 and two thirds through junction 49. The hot charging flow per RCS loop will thus be 65.5 gpm ($150 * 1.31/3$) to be initiated one hour after the initiation of the SBO for the remaining 3 hours of the coping duration. In addition a charging initialization card was included to permit the initiation of charging flow.
3. RCS Flow Rates & Temperatures -- The base model flow rates were recalculated here based on the values from Table 4.4-1 of the FSAR¹⁰. Table V-2 shows the vessel flow rate, core flow rate and the calculation of the vessel inlet and outlet temperatures with the caveat that the Tavg will be maintained at 551.5 F (i.e. the Reduced Tavg program).
4. Bubble Rise Models -- The base model was developed for a RCS leakage of only 27 gpm. The criteria used was Pressurizer drain. Since the criteria here is core

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Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT** Sheet No. 18

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV
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uncovery and the outlet plenum and upper head drain, a bubble rise model is included for the outlet plenum volume 6 (a bubble model in the upper head was in the original model). The model parameters are the same as for the upper head (i.e. Volume 7).

5. Volume overlaps -- Due to the occurrence of bubbles in Volumes 6 & 7, RETRAN was not converging at the time that volume 7 was drained. Therefore, the volume height of volume 6 was adjusted to 9.6 inches to produce an overlap of 0.1 inches (old value was 9.5 inches). For the same reason the volume height of volumes 11, 16, 29 & 34 were also adjusted to 8.605 inches to produce an overlap of 0.1 inches (old value was 8.505 inches).
6. Steam Generator Adjustments -- The mass of water in the secondary side of each S.G. is 43,000 lbm. However, the code model is set up to accept only volumes. Therefore, the volume of the S.G.'s are adjusted to result in this S.G. water mass (i.e. Volume 50 = 1395, Volume 60 = 2906). In addition the initial S.G. liquid level and gain are adjusted to 22.2938 and 0.1044 to obtain an initial S.G. liquid level of 30% NR.
7. AFW control block -- The automatic AFW flow injection is modeled to inject water into the S.G.'s when the S.G. low level reaches 5% NR with a time delay of 3 minutes to account for valve stroking, transit time and time for the pump to get to full speed. The base model erroneously set the trip function for AFW initiation on a parameter for ADV control. This error was conservative in Reference 12 since the AFW would be actuated later than if it was correct. However, for this analysis this error is corrected (Card # 040030).
8. Time step -- In order to reduce the unnecessary time steps and consequently the code execution time, the time steps after the first hour of the transient are changed from 0.05 sec to 0.10 sec.. This will not compromise any of the results or conclusions of the code.

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Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT**

Sheet No. 19

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9. Problem Run Time -- The NUMARC guideline coping duration is 4 hours. However, the charging flow powered by the DSD diesel is restored after one hour. Therefore, the simulation need be performed only for one hour. This simulation is performed for 30 minutes more (i.e. 90 minutes) to simulate some of the recovery process once the charging is restored.

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Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT**

Sheet No. **20**

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TABLE V-1
LIST OF MODIFICATIONS TO BASIC SBO MODEL

No.	Card #	Parameter	Modified value	Base Value	Purpose
1.	010001	No. of minor edits printed	-54	-40	More detail output
2.	010001	No. of trip functions	20	19	Additional trip for charging actuation
3.	010001	Number of bubble Sets	10	9	Addition of set for outlet plenum vol 6
4.	010001	Number of junctions	51	48	Addition of junctions for second leakage path and charging(2)
5.	010002	Number of fill tables	14	13	Addition of fill table for charging
6.	02000x	Minor Edits	--	--	Printing more output variables
7.	0300x0	Edit frequency	--	--	Changes in output edit print frequency
8.	030060	Time steps	.10	.05	less frequent time steps after 1 hr.
9.	040010	Problem run time	4800	7200	Running problem for 1.5 hrs.
10.	040030	Control block	-20	-13	Correction of AFW error
11.	040200	Charging trip	--	--	New charging initialization card
12.	050011	Vessel inlet temp.	527	528	corrected value
13.	050061	Bubble set	10	--	Assigning new bubble model set for vol 6

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Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT**

Sheet No. **21**

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No.	Card #	Parameter	Modified value	Base Value	Purpose
14.	050061	Volume 6 height	9.6	9.5	Changed to get .1" vol 6/7 overlap
15.	050071	Vessel outlet temp.	576	575	corrected value
16.	050111	Volume 11 height	8.605	8.505	Increased to get 0.1 in. vol 11/12 overlap
17.	050161	Volume 16 height	8.605	8.505	Increased to get 0.1 in. vol 15/16 overlap
18.	050291	Volume 29 height	8.605	8.505	Increased to get 0.1 in. vol 29/30 overlap
19.	050341	Volume 34 height	8.605	8.505	Increased to get 0.1 in. vol 33/34 overlap
20.	050501	S.G. "B" Volume	1395	1310	water vol. adjusted to 43,000 lbm
21.	050601	S.G. "A+C" volume	2906	2620	Water vol. adjusted to 86,000 lbm
22.	060101	Bubble set	--	--	Bubble rise model for vol 6 same as vol 7
23.	080011	Junction 1 flow rate	19,361	20,583	Corrected core flow rate
24.	080081	Junction 8 flow rate	6,759	7,110	Corrected Loop "B" flow rate
25.	080091	Junction 9 flow rate	6,759	7,110	Corrected Loop "B" flow rate
26.	080211	Junction 21 flow rate	20,278	21,330	Corrected vessel flow rate
27.	080311	Junction 31 flow rate	13,519	14,220	Corrected Loop "A+C" flow rate
28.	080511	Junction 51	36	--	New RCS leakage path from volume 36

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Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT**

Sheet No. **22**

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV
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No.	Card #	Parameter	Modified value	Base Value	Purpose
29.	080511	Junction 51	2.0	--	Area doubled to double leakage
30.	080512	Junction 51	--	--	continuation card for new junction 51
31.	131301	RCS leakage	-74.67	-35.4	total leakage per RCS loop for first 2 min
32.	131302	RCS leakage	-74.67	-35.4	total leakage per RCS loop for first 2 min
33.	131303	RCS leakage	-44.1	-35.4	total leakage per RCS loop after 2 min
34.	131304	RCS leakage	-44.1	-35.4	total leakage per RCS loop after 2 min
35.	131401	Charging flow	0.0	--	No charging flow at initiation of SBO
36.	131402	Charging flow	0.0	--	No charging flow for one hour of SBO
37.	131403	Charging flow	65.5	--	Charging flow after one hour
38.	131404	Charging flow	65.5	--	Charging flow after one hour
39.	702003	RCS Flow Rate	4.93×10^{-5}	4.69×10^{-5}	Adjust initial gain (1/vessel flow)
40.	702007	Vessel outlet temp.	576.0	575.01	Corrected value
41.	702008	Vessel inlet temperature	527.0	528.02	Corrected value
42.	702009	S.G. Liquid level	22.2938	22.3838	Adjusted to match steady state value
43.	703020	S.G. Level Gain	0.1044	0.1012	Adjusted to get initial 30% NR

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TABLE V-2
CALCULATION OF RCS FLOWS AND TEMPERATURES

The basic equation to calculate heat transfer in the core is:

$$DH_c = Q / M_c \quad \dots \dots \dots \text{Equation 5.1}$$

where,

DH_c = Enthalpy Rise in the Core

Q = Heat generated in core
 $= (4.596 \times 10^9) \times 0.974 = 4.477 \times 10^9 \text{ Btu/hr}$ (Reference 10^9)

M_c = Core mass flow rate
 $= 69.7 \times 10^6 \text{ lbm/hr}$ (Reference 10^6)

Therefore substituting in Equation 5.1,

$$DH_c = 64.2 \text{ Btu/lbm}$$

The enthalpy changes in the core can be stated as:

$$h_{avg} = (h_{ov} + h_{in}) / 2.0 \quad \dots \dots \dots \text{Equation 5.2}$$

and

$$DH_c = h_{oc} - h_{in} \quad \dots \dots \dots \text{Equation 5.3}$$

where,

h_{avg} = average enthalpy in the core
 $= h(P=2100 \text{ psia}, T_{avg}=551.5 \text{ F})$ (This is an approximation)
 $= 549.3 \text{ Btu/lbm}$ (from steam tables)

h_{ov} = Vessel outlet enthalpy

h_{in} = Vessel inlet enthalpy

h_{oc} = Core outlet enthalpy

Therefore combining equations 5.2 and 5.3 and substituting for known quantities,

$$h_{ov} + h_{oc} = (2 \times 549.3) + 64.2$$

$$= 1162.8 \quad \dots \dots \dots \text{Equation 5.4}$$

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From Reference 10* the difference between the vessel temperature rise and the core temperature rise at full power is 2 F (i.e. 49.3 - 47.3)

Therefore, assuming that the core exit temperature and the vessel exit temperature are between 570 and 580 F and interpolating from the steam tables for a pressure of 2100 psia,

$$h_{ov} - h_{oc} = -2.6 \quad \dots \dots \dots \text{Equation 5.5}$$

Combining Equations 5.4 and 5.5 results in,

$$h_{ov} = 1160.2/2 = 580.1$$

Therefore,

$$\begin{aligned} T_{ov} &= \text{Vessel outlet Temperature} \\ &= T(P=2100 \text{ psia}, h_{ov}=580.1 \text{ Btu/lbm}) \\ &= 575.4 \text{ F} \end{aligned}$$

Substituting in Equation 5.2,

$$\begin{aligned} h_{in} &= (2 * 549.3) - 580.1 \\ &= 518.5 \end{aligned}$$

Therefore,

$$\begin{aligned} T_{in} &= \text{Vessel inlet temperature} \\ &= T(P=2100 \text{ psia}, h_{in}=518.5 \text{ Btu/lbm}) \\ &= 526.4 \text{ F} \end{aligned}$$

Subsequently,

$$\begin{aligned} \text{Calculated } T_{avg} &= (575.4 + 526.4) / 2.0 \\ &= 550.9 \text{ F} \end{aligned}$$

Since the actual $T_{avg} = 551.5 \text{ F}$, the values of T_{in} and T_{ov} are adjusted by +0.6 F.

Therefore the values input into RETRAN are as follows:

$$T_{in} = 527 \text{ F}$$

$$T_{ov} = 576 \text{ F}$$

$$\begin{aligned} h_{in} &= h(p=2100 \text{ psia}, T=527 \text{ F}) \\ &= 519.3 \text{ Btu/lbm} \end{aligned}$$

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Sheet No. **25**

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$$h_{ov} = h(p=2100 \text{ psia}, T=576 \text{ F})$$

$$= 580.8 \text{ Btu/lbm}$$

$$M_c = \text{Core mass flow rate}$$

$$= (69.7 * 10^6 \text{ lbm/hr}) / (3600 \text{ Sec/hr})$$

$$= 19,361 \text{ lbm/sec}$$

$$M_v = \text{Vessel mass flow rate}$$

$$= (73 * 10^6 \text{ lbm/hr}) / (3600 \text{ Sec/hr}) \dots \dots \text{ (Reference 10*)}$$

$$= 20,278 \text{ lbm/sec}$$

$$= 6,759 \text{ lbm/sec per RCS loop}$$

* from Table 4.4-1

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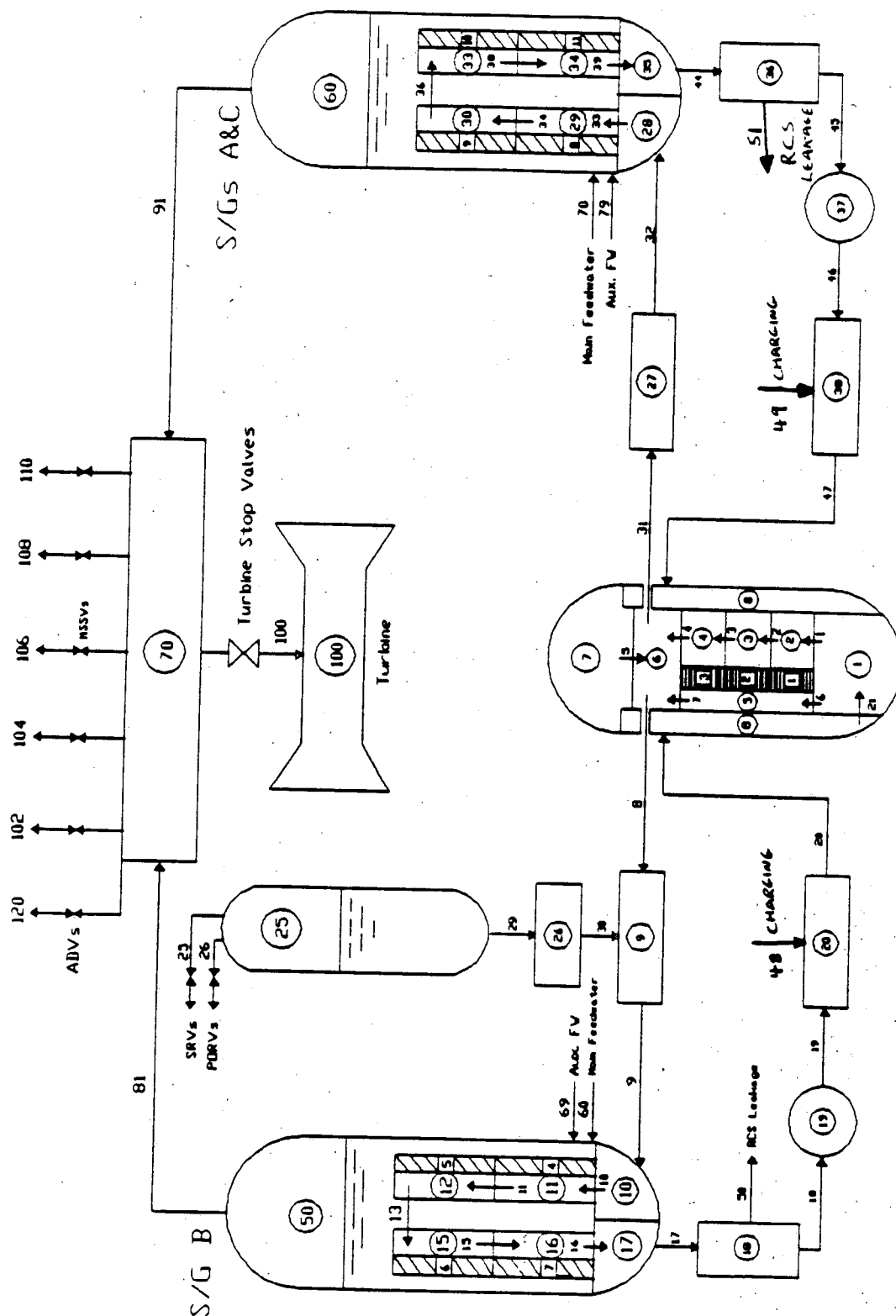
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Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT**

Sheet No. **26**

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Fig.V-1 SONGS 1 RETRAN Nodal Diagram For SBO



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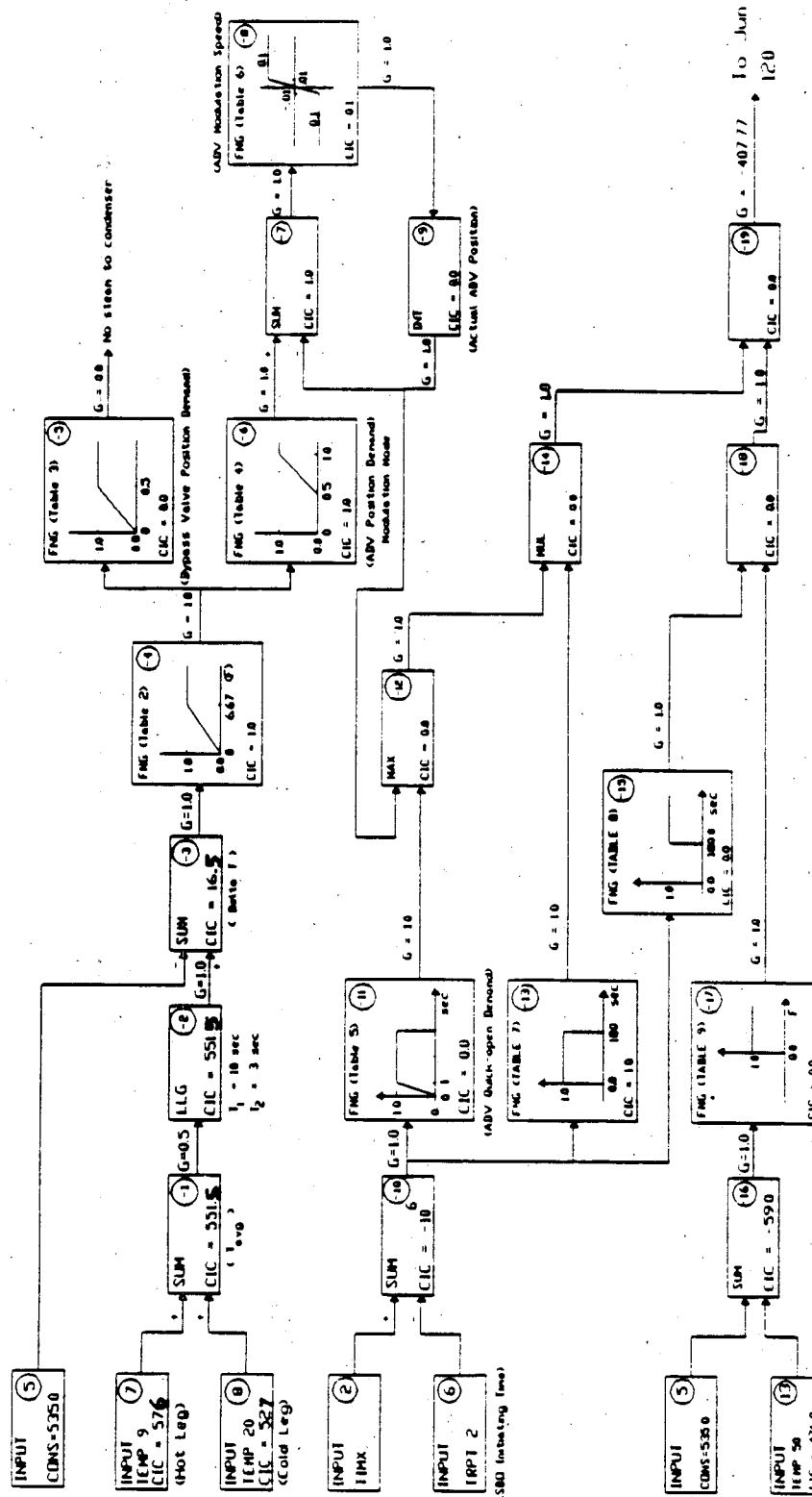
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Fig.V2 SONGS 1 Steam Dump and Bypass Control System Model
For Station Blackout with Reduced Tavg Program



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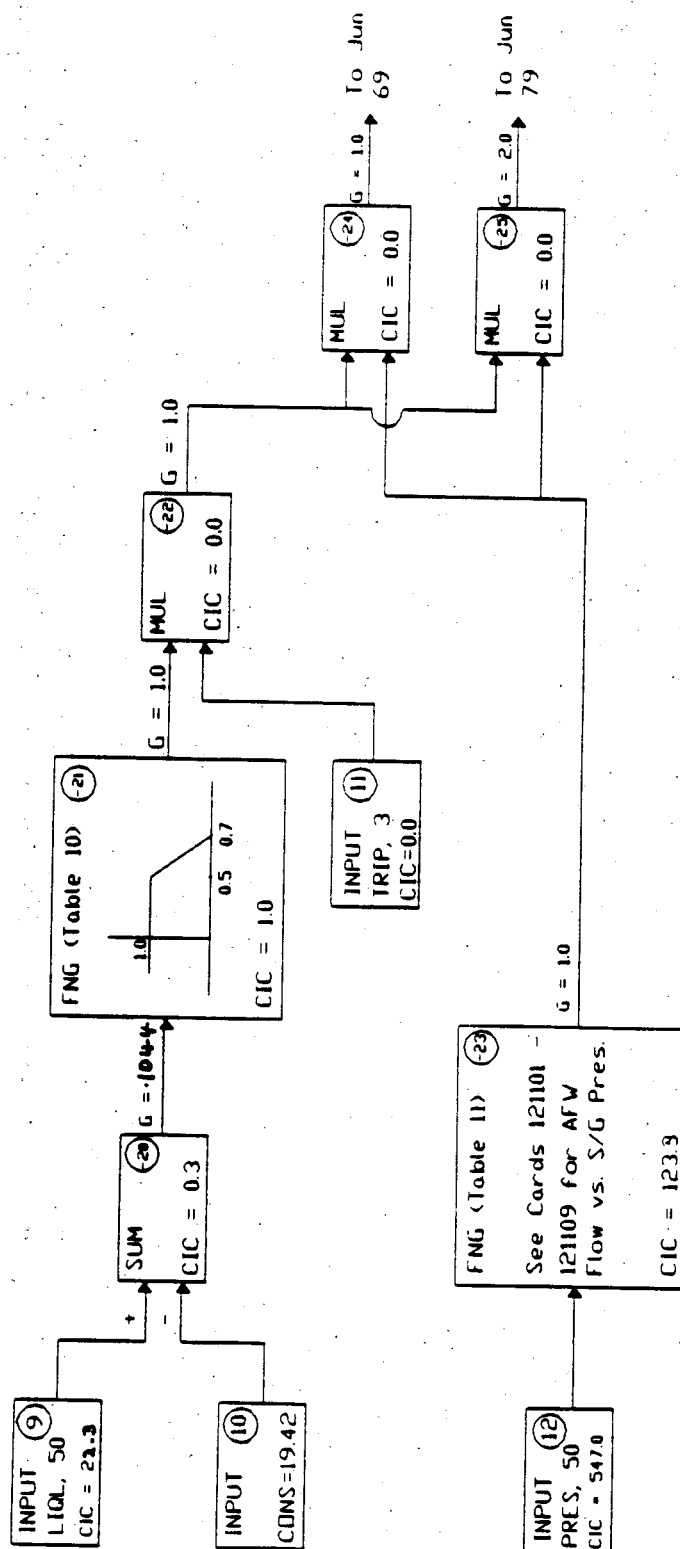
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**Fig. V-3 SONGS 1 AFW Flow Control For Station Blackout Event
With Reduced Tavg Program**



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VI. REFERENCES

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3. NUMARC 87-00, "Guidelines and Technical Bases for NUMARC initiatives addressing Station Blackout at Light Water Reactors", November 1987.
4. EOI S01-1.0-60, Rev. 9, "Loss of all AC power", February 13, 1991.
5. EOI S01-1.0-60.1, Rev. 2, "Background Document for Loss of all AC power", April 10, 1990.
6. Letter for M.O. Medford (SCE) to NRC "Docket Nos. 50-206, 50-361, 50-362; Response to 10 CFR 50.63, Loss of all alternating current power; San Onofre Nuclear Generating Station Units 1, 2 and 3", April 17, 1989.
7. SONGS Unit 1 Technical Specifications, Amendment 143.
8. MMP 1-3645.005N, Revision 0 (See sheet 437).
9. DC-3460, "CRS and Charging System Performance", P. Biba, January 9, 1991 (Attachment 46)
10. SONGS Unit 1 Updated Final Safety Analysis Report (UFSAR)
11. PFC # 1-89-003, "Unit 1 Cycle 10 Core Reload", January 23, 1989 (Attachment J).
12. DC-3533, "SONGS 1 RCS Leakage During Station Blackout", Y.P.Ting, March 8, 1991.
13. Intentionally Blank
14. Songs Unit 1 drawing 5178220-16.
15. Crane Reference Manual "Flow of Fluids through valves, fittings and pipe", 1981 (See chapter 1)
16. DC-2836, Supplement B, "AFW Flow Venturi Sizing", May 21, 1990.
17. Letter from H.C. Calton (Westinghouse) to L.K. Carlisle (SCE), "Southern California Edison Company, San Onofre Nuclear Generating Station Unit 1, Main Steamline

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△	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	△					
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- Break M & E Releases for SONGS-1", NS-OPS-OPL-I-90-613, October 26, 1990.
18. LER 90-004, "Potential for Reactor System Leakage greater than Technical Specification Basis, San Onofre Nuclear Generating Station, Unit 1", April 25, 1990.
 19. Letter from F.R. Nandy (SCE) to NRC, "Supplemental Response to 10CFR 50.63, Loss of All Alternating Current Power, Station Blackout (TAC No. 68599/600), San Onofre Nuclear Generating Station Units 1,2 and 3", May 1, 1990.
 20. Letter from M.F. Muenks (Westinghouse) to P.D. Myers(SCE), "Southern California Edison San Onofre Unit No. 1 Cycle 11 Reload Safety Evaluation Report", 90SC-G-0025, 9/26/90.
 21. Songs Unit 1 drawing 5178130-13.
 22. SONGS Unit 1 Systems Descriptions, SD-S01-280-3-3, Revision 2 (See Figure 3).

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V11. NOMENCLATURE

ADV	Atmospheric dump Valve
AFW	Auxiliary Feedwater Flow
AOV	Air Operated Valves
DSD	Dedicated Shutdown Diesel
EOI	Emergency Operating Instruction
LER	Licensing Event Report
MOV	Motor Operated Valves
NR	Narrow Range (SG level)
NRC	Nuclear Regulatory commission
NUMARC	Nuclear Management and Resource Council
RCP	Reactor Coolant Pump
RCS	Reactor Coolant System
RV	Reactor Vessel
SBO	Station Blackout
SG	Steam Generator

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VIII. CALCULATIONS

A. HAND VERIFICATION

In order to verify the accuracy of the RETRAN code results a simplified hand calculation will be performed. The purpose of this calculation is strictly to verify that the results are in the realm of reasonableness. The RCS inventory calculated during the station blackout is described in Table VIII-1. As shown on this Table the results of the hand verification calculation compare quite favorably with the results of RETRAN. Therefore it is concluded that the RETRAN results are acceptable.

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△	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	△					
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TABLE VIII-1
VERIFICATION OF RCS INVENTORY BY HAND ESTIMATE

NO.	CONTROL VOLUME NAME	RETRAN VOLUME NO.	LIQUID MASS ^a lbm	LEAKAGE ^b		DRAIN TIME (min.) ^d		RETRAN DRAIN TIME ^e min.
				INITIAL ^c lb/sec	REGULAR lb/sec	CONTROL VOLUME	CUMULATIVE SYSTEM	
1.	Pressur- izer	25	18,447	24.2	14.2	20.2	20.2	20.8
2.	Surge Line	26	927	0	14.0	1.1	21.3	22.0
3.	R.V. Upper head	7	17,261	0	13.9	20.7	42.0	45.0
4.	R.V. Outlet Plenum ^f	6	13,105	0	13.9	15.8	57.8	60.0

(a) from RETRAN model at steady state (i.e. at time = 0)

(b) from Table II-2

(c) for first 2 minutes of SBO only

(d) $t = \{ [M - (L_i * c * k)] / (L_r * k) \} + c$

where,

t = Drain time (minutes)

M = Liquid mass

L_i = Initial leakage Rate (i.e. for first 2 minutes of SBO)

L_r = Regular leakage Rate (i.e. after first 2 minutes of SBO)

k = conversion constant from seconds to minutes (i.e. 60.0)

c = initial leakage time in minutes (i.e. 2.0 if initial leakage is greater than 0 or 0.0 otherwise)

(e) from Table II-1

(f) ONLY UNTIL TOP OF HOT LEG (i.e. 39.7% of outlet plenum)

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Project or DCP/MMP _____ Calc. No. **DC - 3601**

CCN CONVERSION:
CCN NO. CCN -

Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT**

Sheet No. **34**

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	R E V
△	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	△					
△					△					

APPENDIX A

LISTING OF RETRAN CODE MODEL

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Sheet No. 35

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓					✓				
✓					✓					✓				
✓					✓					✓				

REF. : (1) RELOAD SAFETY EVALUATION, SAN ONOFRE NUCLEAR GENERATING
STATION, UNIT 1 CYCLE 10.
(2) ACCIDENT ANALYSIS BASIS DOCUMENTATION: MODULE 11, STEAMLINE
RUPTURE FOR SONGS1, WCAP12567, MAY, 1990
(3) SD-SQ1-280, REVISION 2, FIG. 3.

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Sheet No. 36Sheet No. 36

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

*** THIS CARD IS A REPLACEMENT CARD.

*** THIS CARD IS A REPLACEMENT CARD.

020001	PNRM,	0	PRES,	25	MIXL,	25	MIXL,	26	MIXL,	7	TEMP,	9
020002	TEIP,	20	PRES,	60	SPED,	1	SPED,	2	AVEX,	6	AVEX,	7
020003	AVEX,	26	AVEX,	11	AVEX,	12	AVEX,	15	AVEX,	16	WP**,	50
020004	WP**,	60	WP**,	70	WP**,	79	WP**,	81	WP**,	91	WP**,	100
020004	WP**,	51	AVEX,	9	WP**,	79	WP**,	81	WP**,	91	WP**,	100

CALCULATION SHEET

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REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓					✓				
✓					✓					✓				

*** THIS CARD IS A REPLACEMENT CARD.
*** THIS CARD IS A REPLACEMENT CARD.

```

020005 MP**, 20 COUT, 1 COUT, 3 COUT, -1 COUT, -2 COUT, -19
020006 COUT, -20 COUT, -24 COUT, -25 PRES, 50 LIQL, 50 MIXL, 50
020007 MP**, 69 MP**, 102 MIXL, 12 MIXL, 15 * $$ VFN REPLACED $$
020007 MP**, 69 MP**, 102 MIXL, 12 MIXL, 15 TEMP, 26 MIXL, 11
*
* $$$ VFN NEW MINOR EDIT CARDS ADDED $$$
*
020008 TEMP, 12 TEMP, 50 MIXL, 10 MIXL, 17 PRES, 4 TEMP, 4
020009 MIXL, 4 AVEEX, 4 TEMP, 6 MIXL, 6 PRES, 6 TEMP, 11
020009 MIXL, 4 AVEEX, 4 TEMP, 6 MIXL, 6 MP**, 48 MP**, 49
*
* COUT, 1 = NORMALIZED PZR LEVEL,
* COUT, 3 = NORMALIZED RCS FLOW,
* COUT, -2 = TAVG,
* COUT, -19 = ADV RELIEVING CAPACITY,
* COUT, -20 = S/G NR LEVEL,
* COUT, -24 = AUXFW FLOW TO S/G A,
* COUT, -25 = AUXFW FLOW TO S/G B&C.
*
***** TIME-STEP ALGORITHM CONSTANTS - 030001 *****
*
* FC3 FC4 FC5 FC6 FC7 FC8 FC9
*030001 0.0 0.0 10.0 10.0 0.0 0.0 0.0
*
***** TIME STEPS DATA - 03XXX0 *****
*
* $$$ VFN CHANGES MADE $$$
* BETWEEN 1000 AND 3600 SECS
* -- NO CHANGES TO EXECUTION TIME STEPS
* -- MAJOR EDIT PRINTED EVERY 100 SECS (WAS 200 SECS)
* BETWEEN 3600 SECS AND END OF EXECUTION
* -- EXECUTION TIME STEPS EVERY 10 SECS (WAS 5 SECS)
* -- MAJOR EDIT PRINTED EVERY 400 SECS (WAS 500 SECS)
* $$$ VFN CHANGES MADE $$$
*
* NMIN NMAJ NOMP NCHK DELTM DTMIN TLAST
*030010 100 5 9000 0 0.01 0.0 10.0
*030020 100 25 9000 0 0.01 0.0 100.0
*030030 100 50 9000 0 0.02 0.0 500.0
*030040 250 20 9000 0 0.02 0.0 1000.0
*030050 200 20 9000 0 0.05 0.0 3000.0
*030060 400 25 9000 0 0.05 0.0 1.0E+6
*030050 200 10 9000 0 0.05 0.0 3600.0
*030060 200 20 9000 0 0.10 0.0 1.0E+6
*
***** TRIP CONTROLS - 04XXX0 *****
*
* TRIP SIG
*4XXX0 ID ID IX1 IX2 SETPT DELAY
*040010 1 1 0 0 7200.0 0.0 * PROBLEM RUNNING TIME

```

CALCULATION SHEET

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Sheet No. 38

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

```

179 040010 1 1 0 0 4800.0 0.0 * PROBLEM RUNNING TIME
180 *
181 040020 2 1 0 0 0.01 0.0 * SBO OCCURS/RCPS TRIP
182 *
183 040030 3 -14 -13 0 0.05 180.0 * STEAM-DRIVEN AFM PUMP
184 040030 3 -14 -20 0 0.05 180.0 * STEAM-DRIVEN AFM PUMP
185 * OPERATION
186 040040 4 1 0 0 0.0 0.0 * MFM OPERATION
187 *
188 ----- PRIMARY PORVS AND SRVS -----
189 *
190 040050 5 4 25 0 2215.0 0.0 * PORV SETPOINT (2 VALVES)
191 040060 -5 -4 25 0 2190.0 0.0 * RESET (ASSUMPTION)
192 *
193 PORV SETPOINTS = 2200/2215 PSIA (PAGE 17, SD-S01-280)
194 *
195 040070 6 4 25 0 2525.0 0.0 * SRV SETPOINT (2 VALVES)
196 040080 -6 -4 25 0 2450.0 0.0 * RESET (ASSUMPTION)
197 *
198 PZR SRV SETPOINTS = 2500/2525 PSIA (PAGE 19, SD-S01-280)
199 *
200 *
201 ----- SECONDARY SAFETY VALVES -----
202 *
203 040090 7 4 70 0 1000.0 0.0 * MSSV #1 SETPOINT
204 040100 -7 -4 70 0 950.0 0.0 * RESET (ASSUMPTION)
205 *
206 040110 8 4 70 0 1015.0 0.0 * MSSV #2 SETPOINT
207 040120 -8 -4 70 0 965.0 0.0 * RESET (ASSUMPTION)
208 *
209 040130 9 4 70 0 1035.0 0.0 * MSSV #3 SETPOINT
210 040140 -9 -4 70 0 985.0 0.0 * RESET (ASSUMPTION)
211 *
212 040150 10 4 70 0 1045.0 0.0 * MSSV #4 SETPOINT
213 040160 -10 -4 70 0 995.0 0.0 * RESET (ASSUMPTION)
214 *
215 040170 11 4 70 0 1050.0 0.0 * MSSV #5 SETPOINT
216 040180 -11 -4 70 0 1000.0 0.0 * RESET (ASSUMPTION)
217 *
218 040190 25 1 0 0 0.01 0.0 * LEAKAGE INITIALIZATION
219 *
220 * $$$ VFN CHANGES $$$
221 * NEW TRIP SEQUENCE ADDED FOR CHARGING ACTUATION
222 * $$$ VFN CHANGES $$$
223 *
224 040200 26 1 0 0 0.01 0.0 * CHARGING INITIALIZATION
225 *
226 *
227 *
228 ***** VOLUME DATA - 05XXXX (Y=1) *****
229 *
230 * $$$ VFN CHANGES MADE $$$
231 * IN VOLUME 6 (LOWER REACTOR VESSEL HEAD)
232 * -- BUBBLE RISE MODEL INCLUDED (HAD NO B.R. MODEL)
233 * -- INCLUDED OVERLAP OF 0.1 INCHES BETWEEN VOLUME 6 & 7
234 * (HAD NO OVERLAP)
235 * INLET AND OUTLET TEMPERATURES CORRECTED
236 * -- VOLUME 1 TEMPERATURE 527 F (WAS 528 F)
237 * -- VOLUME 9 TEMPERATURE 576 F (WAS 575 F)
238 * IN VOLUMES 11, 16, 29, 34

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CALCULATION SHEET

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REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

*** THIS CARD IS A REPLACEMENT CARD.

\$\$\$ VFN CHANGES MADE \$\$\$

	1	2	3	4	5	6	7	8	9	10
	Y	IB	IR	P	H	X OR	VOL	ZVOL	ZM	FLOWL
						XM				FLOWA
245	*									
246	*	Y	IB	IR	P	H	X OR	VOL	ZVOL	ZM
247	*									
248	050011	0	0	0.0	520.1	0.0	758.7	9.82	9.82	9.82
249	*				(528.0 F)					1.0E+9
250	050011	0	0	0.0	519.3	0.0	758.7	9.82	9.82	9.82
251	*				(527.0 F)					1.0E+9
252	050021	0	0	0.0	0.0	0.0	117.0	3.33	3.33	3.33
253	050031	0	0	0.0	0.0	0.0	117.0	3.34	3.34	3.34
254	050041	0	0	0.0	0.0	0.0	117.0	3.33	3.33	3.33
255	050051	0	0	0.0	0.0	0.0	135.5	10.0	10.0	10.0
256	050061	0	0	0.0	0.0	0.0	734.0	9.5	9.5	9.5
257	050061	10	0	0.0	0.0	0.0	734.0	9.6	9.6	9.6
258	050071	4	0	0.0	579.9	0.0	383.1	6.5	6.5	6.5
259	*				(575.0 F)					1.0E+6
260	050071	4	0	0.0	580.8	0.0	383.1	6.5	6.5	6.5
261	*				(576.0 F)					1.0E+6
262	050081	0	0	0.0	0.0	0.0	393.0	23.25	23.25	18.33
263	050091	0	0	0.0	0.0	0.0	76.7	2.3	2.3	2.3
264	050101	0	0	0.0	0.0	0.0	103.7	7.5	7.5	7.5
265	*									1.0E+6
266	*50111	0	0	0.0	0.0	0.0	73.95	8.505	8.505	8.505
267	*50121	6	0	0.0	0.0	0.0	73.95	8.505	8.505	8.505
268	*50151	7	0	0.0	0.0	0.0	73.95	8.505	8.505	8.505
269	*50161	0	0	0.0	0.0	0.0	73.95	8.505	8.505	8.505
270	*									7.2
271	050111	0	0	0.0	0.0	0.0	59.25	8.505	8.505	8.505
272	050111	0	0	0.0	0.0	0.0	59.25	8.605	8.605	8.605
273	050121	0	0	0.0	0.0	0.0	59.25	8.505	8.505	8.505
274	050151	0	0	0.0	0.0	0.0	59.25	8.505	8.505	8.505
275	050161	0	0	0.0	0.0	0.0	59.25	8.505	8.505	8.505
276	050161	0	0	0.0	0.0	0.0	59.25	8.605	8.605	8.605
277	*									5.8
278	*									(20% TUBE PLUGGING)
279	050171	0	0	0.0	0.0	0.0	103.3	7.5	7.5	7.5
280	050181	0	0	0.0	0.0	0.0	106.5	2.3	2.3	2.3
281	050191	0	0	0.0	0.0	0.0	133.0	5.0	5.0	5.0
282	050201	0	0	0.0	0.0	0.0	53.5	2.3	2.3	2.3
283	050251	1	0	2100.0	0.0	0.0	1300.0	37.20	13.95	37.20
284	*									34.91
285	*									PZR NOMINAL WATER LEVEL = 37.2 FT * 37.5% = 13.95
286	*									
287	050261	5	0	0.0	0.0	0.0	24.5	5.13	5.13	5.13
288	050271	0	0	2105.53	0.0	0.0	153.4	2.30	2.30	2.30
289	050281	0	0	0.0						

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Sheet No. 40

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	R
✓	V. F. Nazarethy	6/21/91	W. Alhassani	6/21/91	✓					E
✓					✓					V
✓					✓					↑

*** THIS CARD IS A REPLACEMENT CARD.
*** THIS CARD IS A REPLACEMENT CARD.

*** THIS CARD IS A REPLACEMENT CARD.
*** THIS CARD IS A REPLACEMENT CARD.

(20% TUBE PLUGGING)

VOLUME DATA - 05XXXY (Y=2)

11	12	13	14	15	16
DIAMV	ELEV	INEQ	VRAIN	VLHTC	MESH

0.0	-24.40	0	0.0	0.0	0
0.0454	-14.58	0	0.0	0.0	0
0.0454	-11.25	0	0.0	0.0	0
0.0454	-7.91	0	0.0	0.0	0
0.0	-14.58	0	0.0	0.0	0
0.0	-4.58	0	0.0	0.0	0
0.0	4.92	0	0.0	0.0	0
0.0	-18.33	0	0.0	0.0	0
2.3	-1.15	0	0.0	0.0	0
0.0	-1.15	0	0.0	0.0	0
0.0533	6.35	0	0.0	0.0	0
0.0533	14.855	0	0.0	0.0	0
0.0533	14.855	0	0.0	0.0	0
0.0533	6.35	0	0.0	0.0	0
0.0	-1.15	0	0.0	0.0	0
2.3	-1.15	0	0.0	0.0	0
0.0	-1.15	0	0.0	0.0	0
2.3	-1.15	0	0.0	0.0	0
6.67	6.18	1	4.0	0.0	0
6.67	6.18	0	0.0	0.0	0

CALCULATION SHEET

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Sheet No. 41

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓					✓				
✓					✓					✓				

TIME	PRES	TEMP	AVG.	MIXL
------	------	------	------	------

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Sheet No. 42

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

*** THIS CARD IS A REPLACEMENT CARD.
*** THIS CARD IS A REPLACEMENT CARD.

N

2

***** JUNCTION DATA - 08XXXXY (Y=1) *****

\$\$\$ VFN CHANGES MADE \$\$\$

RCS FLOW RATES CORRECTED (NO ADJUSTMENTS MADE TO MATCH
VENDOR THOT DATA)

-- JUNCTION 1 WP 19,361 LBM/SEC (WAS 20583)

-- JUNCTION 8 WP 6,759 LBM/SEC (WAS 7,110)

```
-- JUNCTION 9 WP 6,759 LBM/SEC (HAS 7,110)
```

-- JUNCTION 21 WP 20,278 LBM/SEC (NAS 21,330)

-- JUNCTION 31 WP 13,519 LBM/SEC (HAS 14,220)

NEW LEAKAGE JUNCTION (JUNCTION 51) INCLUDED FOR VOLUME 36 IN ORDER TO BETTER REPRESENT THE ACTUAL LEAKAGE IN THE PLANT

THEREFORE :

-- JUNCTION 50 : ONE THIRD OF TOTAL LEAKAGE

-- JUNCTION 51 : TWO THIRDS OF TOTAL LEAKAGE

NEW JUNCTIONS (JUNCTIONS 48 & 49) INCLUDED FOR VOLUMES 20 AND 38 TO REPRESENT CHARGING MAKEUP

THEREFORE :

-- JUNCTION 48 : ONE THIRD OF TOTAL CHARGING

-- JUNCTION 49 : TWO THIRDS OF TOTAL CHARGING

\$\$\$ VFN CHANGES MADE \$\$\$

	1	2	3	4	5	6	7	8	9	10
* XXXY	VI	VO	IP	IV	WP	AJUN	ZJUN	INERTA	FJUNF	FJUNR

080011	1	2	0	0	20583.0	46.60	-14.58	0.034	0.45	1.0
080011	1	2	0	0	19361.0	46.60	-14.58	0.034	0.45	1.0

ASSUMING CORE BYPASS FLOW = 4.5% OF TOTAL CORE FLOW.

080021	2	3	0	0	0.0	46.60	-11.25	0.068	0.0	0.0
080031	3	4	0	0	0.0	46.60	-7.91	0.068	0.0	0.0
080041	4	6	0	0	0.0	48.60	-4.58	0.034	1.0	0.45
080051	7	6	0	0	0.0	50.00	4.92	0.100	0.0	0.0
080061	1	5	0	0	0.0	13.55	-14.58	0.357	0.45	1.0
080071	5	6	0	0	0.0	13.55	-4.58	0.357	-1.0	0.45
080081	6	9	0	0	7110.0	4.15	0.0	2.241	-1.0	1.0
080091	9	10	0	0	7110.0	4.15	0.0	2.241	0.2	0.2
080081	6	9	0	0	6759.0	4.15	0.0	2.241	-1.0	1.0
080091	9	10	0	0	6759.0	4.15	0.0	2.241	0.2	0.2

THE CORE FLOWRATE HAS BEEN ADJUSTED TO MATCH THE THOT WITH
VENDOR'S RESULT.

080101	10	11	0	0	0.0	7.2	6.35	0.788	0.45	1.0
080111	11	12	0	0	0.0	7.2	14.855	1.182	0.0	0.0
080131	12	15	0	0	0.0	7.2	20.50	1.182	0.2	0.2
080151	15	16	0	0	0.0	7.2	14.855	1.182	0.0	0.0

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Sheet No. 43

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

*** THIS CARD IS A REPLACEMENT CARD.

479	080161	16	17	0	0	0.0	7.2	6.35	0.788	1.0	0.45
480	080171	17	18	0	0	0.0	4.15	0.0	2.961	0.2	0.2
481	080181	18	19	-1	0	0.0	4.15	0.0	2.961	0.0	7.5
482	080191	19	20	1	0	0.0	4.15	0.0	1.578	0.0	7.5
483	080201	20	8	0	0	0.0	4.15	0.0	1.578	0.2	0.45
484	080211	8	1	0	0	21330.0	16.9	-18.33	1.376	0.0	0.0
485	080211	8	1	0	0	20278.0	16.9	-18.33	1.376	0.0	0.0
486	*										
487	080251	0	25	5	0	0.0	1.0	43.3	0.0	0.0	0.0
488	080261	0	25	6	0	0.0	1.0	43.3	0.0	0.0	0.0
489	*										
490	080291	25	26	0	0	0.0	0.55	6.18	40.818	1.0	1.0
491	080301	26	9	0	0	0.0	0.55	1.15	40.818	1.0	1.0
492	080311	6	27	0	0	14220.0	8.30	0.0	1.129	-1.0	1.0
493	080311	6	27	0	0	13519.0	8.30	0.0	1.129	-1.0	1.0
494	080321	27	28	0	0	0.0	8.30	0.0	1.129	0.2	0.2
495	080331	28	29	0	0	0.0	14.4	6.35	0.394	0.45	1.0
496	080341	29	30	0	0	0.0	14.4	14.855	0.788	0.0	0.0
497	080361	30	33	0	0	0.0	14.4	20.50	0.788	0.2	0.2
498	080381	33	34	0	0	0.0	14.4	14.855	0.788	0.0	0.0
499	080391	34	35	0	0	0.0	14.4	6.35	0.394	0.2	0.2
500	080441	35	36	0	0	0.0	8.3	0.0	1.480	0.0	0.0
501	080451	36	37	-2	0	0.0	8.3	0.0	1.480	0.0	7.5
502	080461	37	38	2	0	0.0	8.3	0.0	0.789	0.2	7.5
503	080471	38	8	0	0	0.0	8.3	0.0	0.789	-1.0	0.45
504	*										
505	080481	0	20	14	0	0.0	1.0	0.0	0.0	0.0	0.0
506	080491	0	38	14	0	0.0	2.0	0.0	0.0	0.0	0.0
507	*										
508	080501	0	18	13	0	0.0	1.0	0.0	0.0	0.0	0.0
509	080511	0	36	13	0	0.0	2.0	0.0	0.0	0.0	0.0
510	*										
511	080601	0	50	1	0	0.0	1.0	6.4	0.0	0.0	0.0
512	080691	0	50	3	0	0.0	1.0	6.4	0.0	0.0	0.0
513	*										
514	080701	0	60	2	0	0.0	1.0	6.4	0.0	0.0	0.0
515	080791	0	60	4	0	0.0	1.0	6.4	0.0	0.0	0.0
516	*										
517	080811	50	70	0	0	528.3	4.76	43.35	21.01	-1.0	-1.0
518	080911	60	70	0	0	1056.6	4.76	43.35	21.01	-1.0	-1.0
519	*										
520	081001	70	100	0	1	0.0	9.52	43.35	21.01	-1.0	-1.0
521	*										
522	081021	0	70	7	0	0.0	1.0	60.0	0.0	0.0	

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Sheet No. 44

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				
✓					✓				

39	*											
40	080012	0	-1	0	0	0.0445	1.0	0	0	0.0	2	0
41	080022	0	-1	0	0	0.0445	1.0	0	0	0.0	3	0
42	080032	0	-1	0	0	0.0445	1.0	0	0	0.0	3	0
43	080042	0	-1	0	0	0.0445	1.0	0	0	0.0	1	0
44	080052	0	-1	0	0	8.00	1.0	0	0	0.0	0	0
45	080062	0	-1	0	0	4.22	1.0	0	0	0.0	0	0
46	080072	0	-1	0	0	4.22	1.0	0	0	0.0	0	0
47	080082	1	-1	0	0	2.30	1.0	0	0	0.0	0	0
48	080092	1	-1	0	0	2.30	1.0	0	0	0.0	0	0
49	080102	0	-1	0	0	0.0533	1.0	0	0	0.0	2	0
50	080112	0	-1	0	0	0.0533	1.0	0	0	0.0	3	0
51	080132	1	-1	0	0	0.0533	1.0	0	0	0.0	3	0
52	080152	0	-1	0	0	0.0533	1.0	0	0	0.0	3	0
53	080162	0	-1	0	0	0.0533	1.0	0	0	0.0	1	0
54	080172	1	-1	0	0	2.30	1.0	0	0	0.0	0	0
55	080182	1	-1	0	0	2.30	1.0	0	0	0.0	0	0
56	080192	1	-1	0	0	2.30	1.0	0	0	0.0	0	0
57	080202	1	-1	0	0	2.30	1.0	0	0	0.0	0	0
58	080212	1	-1	0	0	4.64	1.0	0	0	0.0	0	0
59	*											
60	080252	0	-1	0	0	1.0	1.0	0	0	180.0	0	1
61	080262	0	-1	0	0	1.0	1.0	0	0	180.0	0	0
62	*											
63	080292	1	-1	0	0	0.84	1.0	0	0	0.0	0	0
64	080302	0	-1	0	3	0.84	1.0	0	0	0.0	0	0
65	080312	1	-1	0	0	3.25	1.0	0	0	0.0	0	0
66	080322	1	-1	0	0	3.25	1.0	0	0	0.0	0	0
67	080332	0	-1	0	0	0.0533	1.0	0	0	0.0	2	0
68	080342	0	-1	0	0	0.0533	1.0	0	0	0.0	3	0
69	080362	1	-1	0	0	0.0533	1.0	0	0	0.0	3	0
70	080382	0	-1	0	0	0.0533	1.0	0	0	0.0	3	0
71	080392	0	-1	0	0	0.0533	1.0	0	0	0.0	1	0
72	080442	1	-1	0	0	3.25	1.0	0	0	0.0	0	0
73	080452	1	-1	0	0	3.25	1.0	0	0	0.0	0	0
74	080462	1	-1	0	0	3.25	1.0	0	0	0.0	0	0
75	080472	1	-1	0	0	3.25	1.0	0	0	0.0	0	0
76	*											
77	080482	0	-1	0	0	1.0	1.0	0	0	0.0	0	0
78	080492	0	-1	0	0	1.0	1.0	0	0	0.0	0	0
79	*											
80	080502	0	-1	0	0	1.0	1.0	0	0	180.0	0	0
81	080512	0	-1	0	0	1.0	1.0	0	0	180.0	0	0
82	*											
83	080602	0	-1	0	0	1.0						

CALCULATION SHEET

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Sheet No. 45

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓					✓				
✓					✓					✓				

$$\begin{aligned} \text{HYDRAULIC TORQUE} &= \text{OVERALL EFFICIENCY (P. 1-8, REF.)} \\ &\quad * \text{PUMP MOTOR TORQUE} \\ &= 0.78 * 1.78\text{E}+4 \end{aligned}$$

CALCULATION SHEET

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Sheet No. 46

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassadi	6/21/91	✓				
✓					✓				

VALVE POSI. "BYPASS" VALVE * ----- TABLE 3 -----

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Sheet No. 47

Sheet No. 47

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	R E V
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓					✓
✓					✓					✓

719	* XXYY	N	DEMAND	POSI. DEMAND	
720	*				
721	120301	-5	-1.0E+6	0.0	
722	120302		0.0	0.0	
723	120303		0.5	1.0	
724	120304		1.0	1.0	
725	120305		1.0E+6	1.0	
726	*				
727	*				
728	*		VALVE POSI.	"ADV"	* ----- TABLE 4 -----
729	*	N	DEMAND	POSI. DEMAND	
730	*				
731	120401	-4	-1.0E+6	0.0	
732	120402		0.5	0.0	
733	120403		1.0	1.0	
734	120404		1.0E+6	1.0	
735	*				
736	*				
737	*		TIME	VALVE	* ----- TABLE 5 -----
738	*	N	(SEC)	POSITION	* FOR QUICK-OPEN MODE
739	*				* ASSUMING FULL OPENED WITHIN
740	120501	-6	-1.0E+6	0.0	* 1.0 SECOND AND LASTS FOR 10
741	120502		0.0	0.0	* SECONDS.
742	120503		1.0	1.0	
743	120504		10.0	1.0	
744	120505		10.1	0.0	
745	120506		1.0E+6	0.0	
746	*				
747	*				
748	*		VALVE POSI.	VALVE	* ----- TABLE 6 -----
749	*	N	ERROR	SPEED	* VALVE SPEED FOR MODULATION
750	*				* MODULATION SPEED = 10 SEC
751	120601	-5	-1.0E+6	-0.1	
752	120602		-1.0E-6	-0.1	
753	120603		0.0	0.0	
754	120604		1.0E-6	0.1	
755	120605		1.0E+6	0.1	
756	*				
757	*				
758	*		TIME	OUTPUT	* ----- TABLE 7 -----
759	*	N	(SEC)		*
760	*				* ADVS ARE IN AUTOMATIC MODE.
761	120701	-4	-1.0E+6	1.0	
762	120702		180.0	1.0	
763	120703		180.1	0.0	
764	120704		1.0E+6	0.0	
765	*				
766	*				
767	*		TIME	OUTPUT	* ----- TABLE 8 -----
768	*	N	(SEC)		*
769	*				* ADVS ARE MANUALLY REGULATED.
770	120801	-4	-1.0E+6	0.0	
771	120802		180.0	0.0	
772	120803		180.1	1.0	
773	120804		1.0E+6	1.0	
774	*				
775	*				
776	*		DELTA T	OUTPUT	* ----- TABLE 9 -----
777	*	N	(F)		
778	*				

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Sheet No. 48

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓					✓				
✓					✓					✓				
✓					✓					✓				

79	120901	-4	-1.0E+6	0.0	
80	120902		0.0	0.0	
81	120903		0.1	1.0	
82	120904		1.0E+6	1.0	
83	*				
84	*				
85	*		S/G		
86	*	N	NR LEVEL	OUTPUT	* ----- TABLE 10 -----
87	*				
88	121001	-4	-1.0E+6	1.0	* OPERATOR ACTION ON S/G
89	121002		0.50	1.0	* LEVEL CONTROL
90	121003		0.70	0.0	* (ASSUMPTION)
91	121004		1.0E+6	0.0	
92	*				
93	*				
94	*		S/G PRES.	AUXFW	* ----- TABLE 11 -----
95	*	N	(PSIA)	(GPM/SQ.FT)	
96	*				
97	121101	-9	0.0	123.3	* REFERENCE:
98	121102		995.0	123.3	* FIG. 21, DESIGN CALC. NO.
99	121103		1030.0	115.0	* DC-2836, SHEET R5-6A.
100	121104		1055.0	108.3	
101	121105		1125.0	88.3	
102	121106		1155.0	78.3	
103	121107		1185.0	64.3	
104	121108		1215.0	50.0	
105	121109		1240.0	33.3	
106	*				
107	*				
108	*				
109	***** FILL TABLE - 13XXYY *****				
110	*				
111	*				
112	*				
113	*				
114	*				
115	*				
116	* XXYY	N	TRIP ID (TIME) JY	TIME (S)	FLUX (LB/SEC-FT**2)
117	*				H PSIA
118	130101	-3	4 0 0	0.0	528.3 394.0 700.0
119	130102			1.0	0.0 394.0 700.0
120	130103			1.0E+6	0.0 394.0 700.0
121	*				
122	* NOTE: TOTAL MFM FLOWRATE AT FULL POWER IS 5.706E+6 LBM/HR (SEE P.20-1, SONGS1 STATION MANUAL).				
123	*				
124	*				
125	*				
126	*				
127	*				
128	*				
129	* XXYY	N	TRIP ID (TIME) JY	TIME (S)	FLUX (LB/SEC-FT**2)
130	*				H PSIA
131	130201	-3	4 0 0	0.0	1056.6 394.0 700.0
132	130202			1.0	0.0 394.0 700.0
133	130203			1.0E+6	0.0 394.0 700.0
134	*				
135	*				
136	*				
137	*				
138	*				

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REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

TRIP	JX	FLUX	H	PSIA				
XXYY	N	ID (PSIA)	JY	PRES (LBM/FT**2-S)				
130701	-4	7	1	0.0	0.0	1.0	1.0	* --JUN 102--
130702				1000.0	0.0	1.0	1.0	* VALVES WITH
130703				1030.0	-332.6	1.0	1.0	* SETPOINT =
130704				1.0E+6	-332.6	1.0	1.0	* 1000.0 PSIA
					(2 VALVES)			
130801	-4	8	1	0.0	0.0	1.0	1.0	* --JUN 104--
130802				1015.0	0.0	1.0	1.0	* VALVES WITH
130803				1045.5	-337.6	1.0	1.0	* SETPOINT =
130804				1.0E+6	-337.6	1.0	1.0	* 1015.0 PSIA

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Project or DCP/MMP _____ Calc. No. **DC - 3601**CCN CONVERSION:
CCN NO. CCN --Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT**Sheet No. **50**

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	R E V
△	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	△					
△					△					

*** THIS CARD IS A REPLACEMENT CARD.
*** THIS CARD IS A REPLACEMENT CARD.

899 * --JUN 106--
900 130901 * 9 1 0 0.0 0.0 1.0 1.0 * VALVES WITH
901 130902 1035.0 0.0 1.0 1.0 * SETPOINT =
902 130903 1066.0 -344.3 1.0 1.0 * 1035.0 PSIA
903 130904 1.0E+6 -344.3 1.0 1.0
904 *
905 * --JUN 108--
906 131001 * 10 1 0 0.0 0.0 1.0 1.0 * VALVES WITH
907 131002 1045.0 0.0 1.0 1.0 * SETPOINT =
908 131003 1076.0 -347.6 1.0 1.0 * 1045.0 PSIA
909 131004 1.0E+6 -347.6 1.0 1.0
910 *
911 * --JUN 110--
912 131101 * 11 1 0 0.0 0.0 1.0 1.0 * VALVES WITH
913 131102 1050.0 0.0 1.0 1.0 * SETPOINT =
914 131103 1082.0 -349.3 1.0 1.0 * 1050.0 PSIA
915 131104 1.0E+6 -349.3 1.0 1.0
916 *
917 *
918 *
919 *
920 *
921 *
922 *
923 *
924 *
925 *
926 *
927 *
928 *
929 *
930 *
931 *
932 *
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936 *
937 *
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939 *
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941 *
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943 *
944 *
945 *
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947 *
948 *
949 *
950 *
951 *
952 *
953 *
954 *
955 *
956 *

TABLE 12 (ADV - JUN 120)

TRIP		JX-CNTL	DUPPLY	CNTL BLK	H		PSIA
N	ID	BLK ID	JY	VALUE (LBH/S-FT**2)	H	PSIA	
1	1000	-19	0	0.0	0.0	1.0	1.0

TABLE 13 (LEAKAGE - JUN 50)

\$\$\$ VFN CHANGES MADE \$\$\$
FOR FIRST 2 MINUTES OF SBO LEAKAGE IS 74.67 GPM/S.G.
FOR REMAINING TIME OF SBO LEAKAGE IS 44.1 GPM/S.G.
\$\$\$ VFN CHANGES MADE \$\$\$

TRIP		JX	JY	TIME (S)	FLUX (GPM/FT**2)	H	PSIA
N	ID						
-2	25	0	1	0.0	-35.4	1.0	1.0
-4	25	0	1	1.0E+6	-35.4	1.0	1.0
				0.0	-74.67	1.0	1.0
				120.0	-74.67	1.0	1.0
				120.1	-44.10	1.0	1.0
				1.0E+6	-44.10	1.0	1.0

TABLE 14 (CHARGING - JUN 48 & 49)

\$\$\$ VFN CHANGES MADE \$\$\$
FOR FIRST ONE HOUR OF SBO CHARGING IS 0.0 GPM/LOOP
FOR REMAINING TIME OF SBO CHARGING IS 65.5 GPM/LOOP
\$\$\$ VFN CHANGES MADE \$\$\$

TRIP		JX	JY	TIME (S)	FLUX (GPM/FT**2)	H	PSIA
N	ID						
-4	26	0	1	0.0	0.0	1.0	1.0
				3600.0	0.0	1.0	1.0

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Sheet No. 51

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

REF. (1): FIG. 4-30, A PARAMETRIC STUDY OF AN ATWS IN A W FOUR-
LOOP PLANT, NSAC-91, NOVEMBER 1985.

Sheet No. 52

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

SCRAM TABLE

DENSITY REACTIVITY TABLE

*142000	-9	31.84	-4.37	* -4.0 PCM/F
*142001		33.40	-3.26	
*142002		34.96	-2.36	
*142003		36.52	-1.63	
*142004		38.08	-1.11	
*142005		39.64	-0.69	
*142006		41.20	-0.34	
*142007		42.76	-0.17	
*142008		44.33	0.0	
*				
*				
*142000	-9	31.84	-5.67	* -8.0 PCM/F
*142001		33.40	-4.4	
*142002		34.96	-3.3	
*142003		36.52	-2.37	
*142004		38.08	-1.6	
*142005		39.64	-1.05	
*142006		41.20	-0.61	
*142007		42.76	-0.28	
*142008		44.33	0.0	
*				
*				
*				
142000	-9	31.84	-3.02	* 0.0 PCM/F
142001		33.40	-2.15	
142002		34.96	-1.43	
142003		36.52	-0.91	
142004		38.08	-0.50	
142005		39.64	-0.28	
142006		41.20	-0.11	

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Sheet No. 53Sheet No. 53

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassan	3/21/91	✓					✓				
✓					✓					✓				

09	*	IVSL	IVSR	IGOM	CELEV	IMCL	IMCR	ASUL	ASUR	VOLS			
10	*	M1	M2	M3	M4	M5	M6	M7	M8	M9			
11	* XXXY												
12	150011	0	2	1	0.0	2	2	0.0	10407.3	91.5	* CORE		
13	150021	0	3	1	0.0	2	2	0.0	10407.3	91.5			
14	150031	0	4	1	0.0	2	2	0.0	10407.3	91.5			
15	*												
16	150041	11	50	2	4.2525	2	2	4727.5	5540.0	25.95	* S/G #8 TUBES		
17	150051	12	50	2	12.7575	2	2	4727.5	5540.0	25.95			
18	150061	15	50	2	12.7575	2	2	4727.5	5540.0	25.95			
19	150071	16	50	2	4.2525	2	2	4727.5	5540.0	25.95			
20	*												
21	150081	29	60	2	4.2525	2	2	9454.8	11080.0	51.75	* S/G A&C TUBES		
22	150091	30	60	2	12.7575	2	2	9454.8	11080.0	51.75			
23	150101	33	60	2	12.7575	2	2	9454.8	11080.0	51.75			
24	150111	34	60	2	4.2525	2	2	9454.8	11080.0	51.75			
25	*												
26	*		HDML		HDML		DHEL		DHER	CHNL	CHNR	IHXQF	NTUBES
27	*		M10		M11		M12		M13	M14	M15	M16	M17
28	* XXXY												
29	150012	0.0		0.0454	0.0		0.0454	0.00	3.33	0			
30	150022	0.0		0.0454	0.0		0.0454	0.00	3.34	0			
31	150032	0.0		0.0454	0.0		0.0454	0.00	3.33	0			
32	*												
33	150042	0.0533		0.0918	0.0533		0.0980	8.505	8.505	1			
34	150052	0.0533		0.0918	0.0533		0.0980	8.505	8.505	1			
35	150062	0.0533		0.0918	0.0533		0.0980	8.505	8.505	1			
36	150072	0.0533		0.0918	0.0533		0.0980	8.505	8.505	1			
37	*												
38	150082	0.0533		0.0918	0.0533		0.0980	8.505	8.505	2			

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REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

***** CONDUCTOR GEOMETRY DATA - 17XXYY *****

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Sheet No. 55

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓					✓				
✓	V. F. Nazareth		W. Alhassani		✓					✓				

----- STAINLESS STEEL -----

Sheet No. 56

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazarein	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

316	190401	-8						
317	*							
318	*	F	CP	F	CP	F	CP	

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Sheet No. 57

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓				
✓					✓				

200401	-5					
**						
**	F	COEF	F	COEF	F	COEF
**						
200402	200.0	7.10E-6	600.0	7.70E-6	1000.0	8.10E-6
200403	1400.0	8.70E-6	1600.0	9.00E-6		

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Sheet No. 58

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE
✓	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	✓					✓					✓				
✓					✓					✓					✓				
✓					✓					✓					✓				

```

*** VFN CHANGES ***
INLET AND OUTLET TEMPERATURES CORRECTED
  -- INLET TEMP 527 F (WAS 528 F)
  -- OUTLET TEMP 576 F (WAS 575 F)
SG LEVEL CHANGED TO GET INITIAL LEVEL AT 30% NR
  -- LEVEL 22.2938 FT (WAS 22.3838)
  -- GAIN (IN BLOCK -20) 0.1044 (WAS 0.1012)
RCS FLOW RATE GAIN ADJUSTED TO RESULT IN STEADY STATE

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NES&L DEPARTMENT

Calc. No. DC - 3601

PAGE ____ OF ____

CCN NO. CCN

Sheet No. 59

RECEIVED

*** THIS CARD IS A REPLACEMENT CARD.
*** THIS CARD IS A REPLACEMENT CARD.
*** THIS CARD IS A REPLACEMENT CARD.

*** THIS CARD IS A REPLACEMENT CARD.

\$\$\$ VFN CHANGES \$\$\$

XXX	IDC	SYMBOL	IREG	CGAIN	CIC
-----	-----	--------	------	-------	-----

1444	702001	1	MIXL,	25	2.6882E-2	0.375	* PZR LEVEL (FRACTION)
1445	702002	2	TIMX,	0	1.0	0.0	* SYSTEM TIME
1446	702003	3	WP**,	21	4.6882E-5	1.0	* NORM. RCS FLOWRATE
1447	702003	3	WP**,	21	4.9315E-5	1.0	* NORM. RCS FLOWRATE
1448	702004	4	CONS,	0	-1.0	-1.0	*
1449	702005	5	CONS,	0	535.0	535.0	* NO LOAD T(REF)
1450	702006	6	TRPT,	2	1.0	0.0	* TRIP TIME FOR TRIP 2.
1451	702007	7	TEMP,	9	1.0	575.01	* HOT LEG TEMP.
1452	702008	8	TEMP,	20	1.0	528.02	* COLD LEG TEMP.
1453	702009	9	LIQL,	50	1.0	22.3838	* S/G LIQUID LEVEL
1454	702007	7	TEMP,	9	1.0	576.0	* HOT LEG TEMP.
1455	702008	8	TEMP,	20	1.0	527.0	* COLD LEG TEMP.
1456	702009	9	LIQL,	50	1.0	22.2938	* S/G LIQUID LEVEL
1457	702010	10	CONS,	0	19.42	19.42	* LOWER NR LEVEL TAP
1458	702011	11	TRIP,	3	1.0	0.0	
1459	702012	12	PRES,	50	1.0	547.0	
1460	702013	13	TEMP,	50	1.0	476.053	

CONTROL BLOCK DEFINITIONS

466	*	1	2	3	4	5	6	7	8	9	10
467	* XXX	IDC	ITYPE	INC1	INC2	GAIN	CP1	CP2	CIC	MIN	MAX
468	*										
469	703001	-1	SUM,	7	8	0.5	1.0	1.0	551.5		
470	703002	-2	LLG,	-1	0	1.0	10.0	3.0	551.5		
471	703003	-3	SUM,	5	-2	1.0	-1.0	1.0	16.5	* DELTA T	
472	703004	-4	FNG,	-3	2	1.0	0.0	0.0	1.0		
473	703005	-5	FNG,	-4	3	0.0	0.0	0.0	0.0	* BYPASS	
474	703006	-6	FNG,	-4	4	1.0	0.0	0.0	1.0		
475	703007	-7	SUM,	-6	-9	1.0	1.0	-1.0	1.0		
476	703008	-8	FNG,	-7	6	1.0	0.0	0.0	0.1		
477	703009	-9	INT,	-8	0	1.0	0.0	0.0	0.0	0.0	1.0
478	703010	-10	SUM,	2	6	1.0	1.0	-1.0	-1.0E+6		
479	703011	-11	FNG,	-10	5	1.0	0.0	0.0	0.0	0.0	1.0
480	703012	-12	MAX,	-9	-11	1.0	0.0	0.0	0.0	0.0	1.0
481	703013	-13	FNG,	-10	7	1.0	0.0	0.0	1.0	0.0	1.0
482	703014	-14	MUL,	-12	-13	1.0	0.0	0.0	0.0	0.0	1.0
483	703015	-15	MUL,	-10	8	1.0	0.0	0.0	0.0	0.0	1.0
484	703016	-16	SUM,	5	13	1.0	-1.0	1.0	-58.947		
485	703017	-17	FNG,	-16	9	1.0	0.0	0.0	0.0	0.0	1.0
486	703018	-18	MUL,	-15	-17	1.0	0.0	0.0	0.0	0.0	1.0
487	703019	-19	SUM,	-14	-18	-407.77	1.0	1.0	0.0		

SEE P.14 OF SD-S01-190 FOR THE ADV RELIEVING CAPACITY.

AUXFW CNTL

*93	703020	-20	SUM,	9	10	0.1012	1.0	-1.0	0.3	0.0	1.0
*94	703020	-20	SUM,	9	10	0.1044	1.0	-1.0	0.3	0.0	1.0
*95	703021	-21	FNG,	-20	10	1.0	0.0	0.0	1.0	0.0	1.0
*96	703022	-22	MUL,	11	-21	1.0	0.0	0.0	0.0	0.0	1.0
*97	703023	-23	FNG,	12	11	1.0	0.0	0.0	123.3		
*98	703024	-24	MUL,	-22	-23	1.0	0.0	0.0	0.0	* TO J 69	

NES&L DEPARTMENT CALCULATION SHEET

ICCN NO./ PRELIM. CCN NO.	PAGE ____ OF ____
CCN CONVERSION: CCN NO. CCN --	

Project or DCP/MMP _____ Calc. No. **DC - 3601**

Subject **SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT** Sheet No. **60**

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV
△	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	△					↓
△					△					

99 703025 -25 MJL, -22 -23 2.0 0.0 0.0 0.0 * TO J 79
 00 *
 01 *
 02 ***** END OF DATA *****
 03

NES&L DEPARTMENT CALCULATION SHEET

ICCN NO./ PRELIM. CCN NO.	PAGE ____ OF ____
CCN CONVERSION: CCN NO. CCN --	

Project or DCP/MMP _____ Calc. No. DC - 3601

Subject SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT Sheet No. 61

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV
△	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	△					
△					△					

APPENDIX B

PRESSURIZER PROGRAM LEVEL

NES&L DEPARTMENT
CALCULATION SHEET

ICCN NO./ PRELIM. CCN NO.	PAGE ____ OF ____
CCN CONVERSION: CCN NO. CCN -	

Project or DCP/MMP _____ Calc. No. DC - 3601

Subject SONGS 1 RCS INVENTORY ANALYSIS FOR STATION BLACKOUT Sheet No. 62

REV	ORIGINATOR	DATE	IRE	DATE	REV	ORIGINATOR	DATE	IRE	DATE	REV
△	V. F. Nazareth	6/21/91	W. Alhassani	6/21/91	△					
△					△					

[205] From: JOHN CUSTER at WEST3 6/20/91 1:41PM (859 bytes: 14 ln)
 To: MIKE MCDEVITT at NESL3
 Subject: Pressurizer Program level

----- Message Contents -----

Mike -

Although Pressurizer program level is not specifically addressed in Operations procedures, this information is common operator required knowledge. It is addressed in our training and system description and is set by I&C as required by engineering. Presently the program level for our lower Tave program (level is programmed to Tave) is 25% level from 0 to 15% power and ramps linearly from 25% to 37.5% level from 15% power to 100% power.

At our present normal full Rx power level of 92% PZR program level is approximately 36.5%.

John Custer 89271 6/20/91