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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM



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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM

1.0 PURPOSE

1.1 The purpose of this training program description is to:

1.1.1 Define the prerequisites required for entry into the Reactor Operator/Assistant Control Operator Training Program.

1.1.2

Define the prerequisites required for submitting the candidate for the NRC Reactor Operators license examination.

1.1.3

Define those areas of training required to satisfy the Reactor Operator/Assistant Control Operator Training Program.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

2.0 REFERENCES

- 2.1 Federal Regulations
 - 2.1.1 10CFR19, "Notice Instructions, and Reports to Workers; Inspections," Section 19.12.
 - 2.1.2 10CFR50, "Domestic Licensing of Production and Utilization Facilities," Appendices B, E, and R.
 - 2.1.3 10CFR55, "Operator's Licenses."
- 2.2 USNRC Regulatory Guides
 - 2.2.1 Regulatory Guide 1.8, "Personnel Selection and Training," Revision 1, 1975.
 - 2.2.2 Regulatory Guide 1.114, "Guidance on Being an Operator at the Controls of a Nuclear Power Plant."
- 2.3 USNRC Nuclear Reactor Regulation Guides
 - 2.3.1 NUREG 0094, "A Guide for the Licensing of Facility Operators, Including Senior Operators," July 1976.
 - 2.3.2 NUREG 0737, "Clarification of TMI Action Plan Requirements," Items I.A.2.1, I.A.2.3, I.A.3.1 and II.B.4, October 1980.
- 2.4 USNRC Memoranda
 - 2.4.1 SECY-79-330E, Qualifications of Reactor Operators," July 1979.
 - 2.4.2 SECY-81-84, "Qualifications of Reactor Operators," February 1981, as amended May 1981.
- 2.5 American National Standards
 - 2.5.1 ANSI/ANS 18.1, "Selection and Training of Nuclear Power Plant Personnel," 1971.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

- 2.0 REFERENCES (Continued)
 - 2.6 INPO Guidelines

2.6.1 INPO 82-008, "Control Room Operator, Senior Control Room Operator, and Shift Supervisor Qualification," September 1982.

- 2.7 SONGS Station Orders
 - 2.7.1 S0123-A-126, "Personnel Training."
- 2.8 Nuclear Training Division Training
 - 2.8.1 GEN-2 "General Employee Orientation" Program Descriptions
 - 2.8.2 HPT-5 "Professional Radiation Training"
- 2.9 Unit 1 Tech. Specifications Section 6.4 "Administrative Controls - Training"
- 2.10 Unit(s) 1 or 2 & 3 Operations Qualification Guide for Assistant Control Operator.

2.11 SONGS Units 2 & 3 FSAR Section 13.2 Training

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR

(Continued)

3.0 DEFINITIONS

3.1 Academic Training

Academic training is successfully completed college-level work leading to a recognized degree in a discipline related to the position in question.

3.2 Equivalent

Equivalent experience or training is that as defined by "Nuclear Power Plant Experience," 3.10. Equivalency will be determined on an individual basis by the Supervisor, Operations Training, and the Unit(s) 1 or 2 & 3 Operator Training Administrator and may replace all or part of individual training requirements.

Documentation should be provided any time equivalent training, operational experience or education is used to replace any portion of this training program.

3.3 Exam (Oral/Written)

An evaluation tool used to prove satisfactory progress in any program or course.

3.4 Experience

Applicable work in design, construction, preoperational and startup testing activities, operation, maintenance, onsite activities, or technical services. Observation of others performing work in the above areas is not experience.

3.5 Extra Person On Shift Training

Training conducted on-shift prior to licensing for a minimum of three (3) months (12 weeks or 480 hours). During this training period the candidate participates in the operation of a Nuclear Power Plant performing duties, under instruction of licensed operators, consistent with the licensed position for which the candidate is being trained. (NUREG 0737, I.A.2)

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM

(Continued)

3.0 DEFINITIONS (Continued)

3.6 Licensed Reactor Operator (RO)

Any individual who possesses an operator's license pursuant to Title 10, Code of Federal Regulations, Part 55, "Operating Licenses." An individual performing the functions of a licensed reactor operator will normally be designated as either Control Operator (CO) or Assistant Control Operator (ACO).

3.7 Licensed Senior Reactor Operator (SRO)

Any individual who possesses a Senior Operators License pursuant to Title 10 Code of Federal Regulations, Part 55, "Operators Licenses." Any individual performing the functions of a licensed senior reactor operator will normally be designated as either a Control Room Supervisor (CRS) or a Shift Superintendent (SS).

3.8 Nuclear Plant Equipment Operator (NPEO)

Any individual who carries out the duties and responsibilities identified in the Southern California Edison job specification for job code, "Operator, Nuclear Plant Equipment."

3.9 Nuclear Power Plant

A nuclear power plant is any plant using a nuclear reactor to produce electric power, process steam, or space heating.

3.10 Nuclear Power Plant Experience

Experience acquired in the preoperational and startup testing activities, or operation of nuclear power plants. Experience in design, construction, and operational training (not classroom) may be considered applicable nuclear power plant experience and should be evaluated on a case-by-case basis.

Experience acquired at military, nonstationary, propulsion, or production nuclear plants may qualify as equivalent to nuclear power plant experience on a two-for-one time basis up to a maximum of three years. Two years of such experience equals one year nuclear power plant experience.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM

(Continued)

3.0 DEFINITIONS (Continued)

3.10 (Continued)

Training may qualify as equivalent to nuclear power plant experience if acquired in reactor simulator training programs to a maximum of three month's credit.

On-the-job training may qualify as equivalent to nuclear power plant experience on a one-for-one time basis for up to a maximum of one year's credit.

3.11 Nuclear Reactor

Any assembly of fissionable material which is designed to achieve a controlled, self-sustaining neutron chain reaction.

3.12 NRC Examination for Operator Licenses

Reactor Operator and Senior Reactor Operator licenses may be issued by the NRC to qualified and examined personnel prior to initial core loading of the subject reactor. Examinations for these licenses are termed "Cold" examinations. Examinations administered subsequent to the initial criticality of the reactor are termed "Hot" examinations. The licenses issued based on the results of either hot or cold examinations are equivalent, and are issued for two-year periods. (ANSI N18.1-1971, Sec. 2.2.8)

3.13 On-the-Job Training

On-the-job training is participation in nuclear power plant startup, operation, maintenance, or technical services under the direction of appropriately experienced personnel.

3.14 Related Technical Training

Related technical training is formal training beyond the high school level in technical subjects associated with the position in question, acquired in training schools or programs conducted by the military, industry, utilities, universities, vocational schools, or others. Such training programs should be of a scheduled and planned length and include text material, lectures, and frequent examinations.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

DEFINITIONS (Continued) 3.0

3.15 Simulator Training

The simulator used to meet the training requirements for license candidates and requalification training. A simulator shall have simular operating characteristics to the trainee's own plant.

3.16 Shall, Should, May

The word "shall" is used to denote a requirement; the word "should" to denote a recommendation; and the word "may" to denote permission, neither a requirement nor a recommendation.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

4.0 PREREQUISITES

4.1 Program Prerequisites

Prior to being accepted into the Reactor Operator Training Program, the applicant shall have met the following program prerequisites:

4.1.1 Education

High school diploma or equivalency certificate. (ANSI-N18.1-1971, Sec. 4.5.1)

4.1.2 Aptitude

Power Operators Selection Service (POSS) test score of 11 or greater.

4.2 NRC Licensing Prerequisites

Before submitting an application to take the NRC Operator Licensing Examination, the Reactor Operator license candidate shall have satisfied the following prerequisites:

4.2.1 Experience

.1 Minimum of 2 years of power plant experience of which at least 1 year shall be nuclear power experience. (ANSI-N18.1-1971, Sec. 4.5.1)

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

4.0 PREREQUISITES (Continued)

.2 Minimum of 6 months at the site for which the license is sought. NUREG-0094, Appendix F, Sec. C.2), (Letter from H. Denton, Mar. 28, 1980, Enclosure 1, Sec. A-1)

.3 Military propulsion plant operating experience and licensed operator experience at another facility may be substituted as stated in paragraph 4.2.1.1.

4.2.2 Training

- .1 Minimum of 3 months' training in the control room as an extra person on shift (Letter from H. Denton, Mar. 28, 1980, Enclosure 1, Sec. A.2.b).
- .2 Training in (1) heat transfer, (2) fluid flow, (3) thermodynamics, (4) use of installed plant systems-to control or mitigate an accident in which the core is severely damaged, and (5) reactor and plant transients (NUREG-0737, Enclosure 1, Sec. A.2.c).
- .3 Total of 500 hours of lectures on subjects listed in ANSI N18.1-1971, Section 5.2.1, related subjects and prerequisite courses (NUREG-0094, Appendix F, Sec. C).
- .4 Manipulation of the controls of the facility during five significant reactivity changes as described in the operator requalification program. Every effort should be made to diversify reactivity changes (NUREG-0094, Appendix F, Sec. C.4). A simulator may be used to meet this requirement.
- .5 Participation in reactor and plant operation at power levels up to at least 20% power operation (NUREG-0094, Appendix F, Sec. C.3). A simulator may be used to meet this requirement.
- •
- .6 Paragraphs 4.2.2.2 and 4.2.2.3 of these training prerequisites can be met by successful completion of the training program described in Section 5.1 of this document.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM

(Continued)

4.0 **PREREQUISITES** (Continued)

4.2.3 Facility Certification

Certifications completed pursuant to sections 55.10(a)(6) and 55.33a(4) and (5) of 10CFR Part 55 shall be signed by the highest level of corporate management for plant operation within the Southern California Edison Company. (NUREG 0737, Sec. A.3)

4.2.4

Medical

Comply with the guidelines for physical condition and general health for control room operators as stated in the applicable portions of ANS 3.4/ANSI N546, "American National Standard Medical Certification and Monitoring of Personnel Requiring Operator Licenses for Nuclear Power Plants." (10CFR 55.10(7)) (NUREG 0094, Appendix D)

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

5.0 PROGRAM DESCRIPTION

5.1 Program Administration

5.1.1 The Reactor Operator/Assistant Control Operator Training Program should follow the program appendices provided in the RO/ACO Program Description. Specifically, items identified as required in section 5.1 shall be completed.

5.1.2 Classroom Training

- .1 Science and Engineering Fundamentals (Appendix C) and Plant Specific Training (Appendix D) shall consist of a combined program length of 500 hours of lectures. (ANSI N18.1-1971, Sec. 5.2.1)(NUREG-0094, App. F, Sec. C)
- .1.1 Approximately four (4) hours per day should be devoted to presentation of material and/or examination. The remaining time each day should be for structured study or review of appropriate material in either laboratory, classroom, or the plant. Instructors should be available during study periods to assist the candidate as necessary.
- .1.2 A waiver of candidate participation in any of the classroom training courses may be granted provided:
 - .1.2.1 An oral and/or written evaluation administered by the Nuclear Training Division that demonstrates knowledge comparable to that required for satisfactory course completion.
 - .1.2.2 Previous equivalent training has been successfully completed as determined by the Supervisor of Operations Training and the unit(s) 1 or 2/3 Operator Training Administrator.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM

(Continued)

5.0 PROGRAM DESCRIPTION (Continued)

- 5.1.3 Extra Person On Shift Training (EPOS) (Appendix E.1)
 - .1 An EPOS training plan should be developed and approved by the Unit(s) 1 or 2 & 3 Operation Superintendent or his designee and the Unit(s) 1 or 2 & 3 Operations Training Administrator for all candidates entering EPOS training. Specifically identified portions of the RO/ACO (On-the-Job Training) Qualification Guide should be completed as a part of the EPOS training.
 - .2 Before beginning actual training, the Unit(s) 1 or 2 & 3 Operations Superintendent or his designee and the Unit(s) 1 or 2 & 3 Operations Training Administrator should meet with all candidates to discuss the plan(s) for completing EPOS training and the on-shift respon-sibilities of the candidates.
 - .3 During the EPOS phase of the training program, the RO/ACO candidate should be assigned to a shift as a control room operator in training in the SONGS Unit(s) 1 or 2 & 3 Operations Department and placed under the administrative control of the Operations Department.
 - .3.1 While in the EPOS phase of the training program, the candidate should make satisfactory progress toward completing the approved EPOS training defined from topics contained in Appendix E.1 "Extra Person On Shift Training."
 - .3.2 The RO/ACO candidate shall have a minimum of three (3) months (12 weeks or 480 hours) training as an extra person on shift.
 - .4 As an extra person on shift, the candidate should be considered in training for the Assistant Control Operator position on SONGS Unit(s) 1 or 2 & 3 and should not be assigned collateral duties.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

5.0 PROGRAM DESCRIPTION (Continued)

- .4.1 Signatures for completed EPOS training and evaluation should be obtained from qualified Unit(s) 1 or 2 & 3 Operations personnel.
- .4.2 The candidate's progress and performance, in completing his/her EPOS training plan, should be reviewed monthly by the unit on-shift trainer. The on-shift trainer should forward a copy of the documentation for the Assistance Control Operator Qualification Program upon completion of EPOS program to the Supervisor of Operations Training.
- .5 Upon completion of the three (3) months EPOS training, the RO/ACO candidate should be assigned to the Nuclear Training Department to complete additional licensing requirements.

5.1.4

- Simulator Training (Appendix E.2)
- .1 The simulator training and attendant classroom training shall total at least 4 (four) weeks of which 50% may be classroom training.
- .2 Simulator team training should be in groups of no more than four (4) persons.
- .3 Simulator training shall include participation in the evolutions specified in Appendix E.2 of this program description. Simulator and facility reactivity manipulations are considered equivalent.
 - .1 Manipulation of the controls during five significant reactivity changes evolutions (NUREG 0094, App. F, Sec. C.4).
 - .2 Participation in reactor and plant operation at power levels up to at least 20% power operation (NUREG 0094, Sec. C.3).

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

5.0 PROGRAM DESCRIPTION (Continued)

- .3 Each candidate shall perform a reactor startup during the Simulator Training Program.
- 5.1.4.4 SONGS Unit(s) 1 or 2 & 3 procedures and technical specifications should be used whenever possible and appropriate during simulator training.
 - .5 Documentation of simulator training shall be provided by the simulator facility.
- 5.1.5 Pre-license Review

A review course should be conducted to ensure candidate preparedness for the NRC licensing examinations.

- .1 The course may include coverage of any or all of the items from the previously conducted science/ engineering fundamentals and plant-specific training courses.
- .2 The review course should include training in recent plant system and procedure modifications impacting the duties and responsibilities of the licensed operator.
- .3 The review course should include Simulator Training as described in Section 5.1.4.

5.1.6 Examinations

The following criteria should be used as a basis for determining an acceptable level of performance during all phases of training for RO and SRO candidates.

- .1 Written examinations.
- .1.1 Science and Engineering Fundamentals, Systems Training and Pre-License Review

.1.1.1 Greater than 70% in each section or retested in that section.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

5.0 **PROGRAM DESCRIPTION** (Continued)

- .1.1.2 Greater than 80% overall or a complete retest.
- .1.1.3 Retest scores greater than 70% in each section and greater than 80% overall.
- .1.1.4 Failure of an overall examination requires administrative review by the Unit License Coordinator and implementation of a Remedial Program approved by the Unit Training Administrator.
- .1.1.5 Failure of a retake examination or section of a retake examination requires administrative review by the Unit Training Administrator, and could be grounds for removal from the current licensing program.

.1.2 Pre-License Audit Examination.

- .1.2.1 An NRC type written audit examination should be administered at approximately the 12th week of the Pre-License Review Program. The examination will be written and evaluated in accordance with NUREG-1021, Guidelines for Operator Written Examinations.
- 1.2.2 A grade of less than 70% in any section or less than 80% overall shall require an administrative review by the Units 2/3 Training Administrator and could be grounds for removal from the current licensing program.
- 5.1.6.2 Oral Examinations

.2.1 Pre-License Review

A minimum of one Oral/Plant Walk-Through Examinations shall be conducted for each license candidate during the Pre-License Review.

An oral examination shall be graded as "Satisfactory", "Unsatisfactory", or "Marginal" based upon the candidates responses to questions during an oral examination or plant walk-through.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

5.0 PROGRAM DESCRIPTION (Continued)

- .2.1.1 "Unsatisfactory" evaluations require a retest.
- .2.1.2 An "Unsatisfactory" evaluation on a retest shall require an administrative review by the Unit Licesnse Coordinator and implementation of a Remedial Program approved by the Unit Training Administrator.
- .2.1.3 An Oral/Plant Walk-Through Examination shall be included as a portion of the Pre-License Audit Examination process.
 - .3.1 An evaluation of a Pre-License Audit Oral/Plant Walk-Through marked as "Unsatisfactory" shall require an administrative review by the Unit Training Administrator and could be grounds for removal from the current licensing program.
- 5.1.6.3 Simulator Evaluations
- <u>NOTE</u>: Simulator Evaluations shall be conducted during simulator operations training and during the Pre-License Audit Examination as applicable.
 - 3.1 Subjective periodic evaluations by the simulator instructors shall be completed for each candidate. The evaluations should address such areas as teamwork, communication, control board technique and dexterity, procedural knowledge, and plant system knowledge.
 - 3.1.1 Periodic evaluation by the simulator instructors shall be "Satisfactory", "Unsatisfactory", or "Marginal" in each category described above and either "Satisfactory", "Unsatisfactory", or "Marginal" overall.
 - 3.1.2 Two consecuative periodic overall "Unsatisfactory" or Marginal" evaluations shall require an administrative review by the Unit Training Administrator, and implementation of a Remedial Program approved by the Unit Training Administrator.

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

5.0 PROGRAM DESCRIPTION (Continued)

3.2 A simulator operational demonstration should be included as a portion of the Pre-License Audit Examination Process as applicable.

3.2.1 An evaluation of a Pre-License Simulator Audit marked as "Unsatisfactory" shall require an administrative review by the Unit Training Administrator and could be grounds for removal from the current licensing program.

5.1.6.4 REVIEW BOARD

- .1 Approximately 2 weeks prior to the NRC Examination, a Review Board should be convened to determine the final acceptability of candidates to participate in the NRC Examination.
 - 4.1.1 The Review Board shall examine the training documentation of each candidate to determine the acceptability of each candidate to participate in the NRC Examination.
 - 4.1.2 The Review Board should consist of the following members:
 - 2.1 *Unit License Coordinator.
 - 2.2 *Unit Operations Training Administrator.
 - 2.3 *Supervisor of Operations Training.
 - 2.4 (1) Unit Operations Superintendent.
 - 2.5 (1) STA Supervisor.

* Minimum quorum required.

(1) As applicable to the individual license candidate.



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	.3 Transient/Accident Analysis (Appendix E.3)	55.21(e)		0094 0737			
	.4 Core Damage Recognition/ Mitigation (Appendix E.4)	55	N18.1	0094 0737 0578	Sec. II	 REQUIDE 1.8 SECY 81-84 	
	.5 Integrated Plant Operations (Appendix E.5)	55.22(f) 55.22(g)				 	
	.6 Emergency Plan (Appendix E.6)	55.23(k)					
	.7 Industry and Facility Experiences (Appendix E.7)	55.23(e) 55.23(f) 55.23(1)			 		

*55.21(b),(c),(d),(f),(h),(i),(k);
55.22(a),(b),(c),(h),(i);
55.23(a),(b),(g),(h),(i)

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REACTOR OPERATOR/ASSISTANT CONTROL OPERATOR TRAINING PROGRAM (Continued)

	•			REQU	IREMENT SOU	RCES	·····
5.0	PROGRAM DESCR	RIPTION (Continued)	10 CFR	ANSI	NUREG	IDENTON LTR	OTHER
	•	perations Training Courses		 	 	 	-
	.1 E (xtra-Person-On-Shift Training Appendix E.1)	. •	 	 1021ES109 0737 Encl. I		
	.2 S (imulator Training Appendix E.2)	55	3.5	 0737 	1	 REGUIDE 11.49 SECY 81-84 SECY 330E
	.3 C	ontrol Room Communications Appendix E.3)			 		
	5.2.5 P	re-License Review					I I
	5.2.6 S	ite Certification Test					
•	5.2.7 Jo (/	ob Qualification Training Appendix F)					REGUIDE 1.8
			-		- -		.
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APPENDICES

- A. General Employee Orientation
- B. Initial Plant Familiarization
- C. Science and Engineering Fundamentals Courses
 - C.1 Power Plant Overview
 - C.2 Mathematics
 - C.3 Classical Physics
 - C.4 Chemistry Fundamentals
 - C.5 Fluids/Heat Transfer/Thermodynamics
 - C.6 Reactor Thermodynamics
 - C.7 Mechanical Fundamentals
 - C.8 Electrical Science
 - C.9 Nuclear Physics
 - C.10 Reactor Theory
 - C.11 Radiation Science
 - C.12 Radiochemistry
 - C.13 Materials Science
 - C.14 Instrumentation and Control Fundamentals

D. Plant-Specific Training Courses

- D.1 Administrative Requirements
- D.2 Plant Systems
- D.3 Transient/Accident Analysis
- D.4 Core Damage Recognition/Mitigation
- D.5 Integrated Plant Operations
- D.6 Emergency Plan
- D.7 Industry and Facility Experiences
- E. Operations Training Courses
 - E.1 Extra Person On Shift Training
 - E.2 Simulator Training
 - E.3 Control Room Operator Skills Training
- F. Job Qualification Training

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APPENDIX A

GENERAL EMPLOYEE ORIENTATION

- A. General Employee Orientation should include training in policies, practices, procedures and employee responsibilities related to:
 - 1. Station security
 - 2. Industrial safety

3. Health physics/radiation protection

4. Station emergencies (including radiological emergency plan)

5. Quality assurance/Quality control

- 6. Station administration
- 7. First aid
- 8. Station fire protection program
- 9. Station communications
- B. This training should be conducted in accordance with Training Program Description GEN-2.
- C. This section may be waived if previously completed.

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APPENDIX B

INITIAL PLANT FAMILIARIZATION

- A. Initial plant familiarization should include both formal classroom presentations and in-plant tours.
 - 1. The classroom portion should introduce the candidate to the basic primary and secondary plant systems, major components and their functions. A list of commonly used acronyms and industry terms should be provided to enhance the candidate's ability to interact with other plant personnel.
 - Both instructor guided and self guided tours should be conducted to ensure the candidate can locate major plant areas and specific equipment. "Initial Plant Familiarization Checklists" should be provided for this purpose.
 - 3. At the completion of the familiarization period a written test may be administered.

4. This section may be waived if previously completed.

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APPENDIX C.1

SCIENCE AND ENGINEERING FUNDAMENTALS

POWER PLANT OVERVIEW

A. Training in this area should include:

- 1. Nuclear Energy
 - a. Power Production
 - b. Fission
 - c. Myths
 - d. Risks/Benefits
- 2. The Nuclear Power Plant
 - a. Reactor Components
 - b. BWR
 - c. PWR
 - d. Other Designs

3. Construction and Licensing

- a. NRC
- b. Plant Licensing
- c. Operator Licensing
- d. Fuel Load
- 4. Plant Operations
 - a. Organization
 - b. Quality Assurance
 - c. Training
- B. All, or portions, of this section may be waived based on equivalent training, education or experience.

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APPENDIX C.2

SCIENCE AND ENGINEERING FUNDAMENTALS

MATHEMATICS

- A. Training in this area should include:
 - 1. Arithmetic
 - a. Whole numbers
 - b. Fractions
 - c. Decimals
 - d. Arithmetic Operations
 - 2. Exponents
 - 3. Scientific Notation
 - 4. Common Logarithms
 - 5. Natural Logarithms

6. Algebra

- a. Functions
- b. Linear Equations
- c. Quadratic Equations
- d. Solutions of Equations
- 7. Unit Analysis
- 8. Trigonometry
 - a. Functions
 - b. Tables
 - c. Vectors

9. Graphs

- a. Cartesian coordinates
- b. Polar coordinates
- c. Seim-Log
- d. Log-Log
- e. Nomographs
- 10. Dynamic Situation Analysis
 - a. Slope of a line
 - b. Area under a line

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APPENDIX C.2

SCIENCE AND ENGINEERING FUNDAMENTALS

MATHEMATICS

- 11. Statistics
 - a. Averages
 - b. Weighted Averages
 - c. Deviations

12. Probability

- a. Random Events
- b. Nuclear Applications
- B. All, or portions, of this section may be waived based on equivalent training, education or experience.

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APPENDIX C.3

SCIENCE AND ENGINEERING FUNDAMENTALS

CLASSICAL PHYSICS

- A. Training in this area should include:
 - 1. Unit Analysis
 - a. Units of measurement
 - 1) Time
 - 2) Distance
 - 3) Mass
 - 4) Metric vs. English
 - b. Combining Fundamental Units
 - c. Unit Conversion

2. Element of Mechanics

- a. Forces
 - 1) Gravitational Forces
 - 2) Friction
- b. Motion
 - 1) Linear
 - 2) Newton's Laws
 - 3) Rotational
- c. Momentum
 - 1) Linear
 - 2) Angular
- d. Work and Energy
- e. Power

3. Electrostatics

- a. Electric Charges
- b. Coulombs Law
- c. Electric Field
- d. Electric Potential
- e. Electron Volt
- 4. Electromagnetic Radiation
 - a. Wave propagation.
 - b. Quantum theory.

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APPENDIX C.3 (cont.)

SCIENCE AND ENGINEERING FUNDAMENTALS

CLASSICAL PHYSICS

B. All, or portions, of this section may be waived based on equivalent training, education or experience.

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APPENDIX C.4

SCIENCE AND ENGINEERING FUNDAMENTALS

CHEMISTRY FUNDAMENTALS

- A. Training in this area should include:
 - 1. Physical/Chemical Properties
 - 2. Elements/Compounds/Mixtures
 - 3. Atomic Structure
 - 4. Ionization/Valence
 - 5. Periodic Table
 - 6. Bonding
 - 7. Reactions
 - 8. Catalysis
 - 9. Acids and Bases
 - 10. pH
 - 11. Conductivity
 - 12. Solubility
 - 13. Types of impurities
 - a. Dissolved
 - b. Suspended

14. Sources of Impurities

- a. Erosion
- b. Corrosion
- c. Carryover
- d. Makeup
- 15. Corrosion Types
 - a. General
 - b. Localized
 - c. Galvanic
 - d. Crevice
 - f. Chloride stress
 - g. Caustic stress

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APPENDIX C.4 (cont.)

SCIENCE AND ENGINEERING FUNDAMENTALS

CHEMISTRY FUNDAMENTALS

16. Corrosion of Plant Materials

- a. Carbon steel
- b. Stainless steel
- c. Zircalloy
 - 1) Hydriding
 - 2) Metal-Water

17. Effects of Corrosion and Corrosion Products

- 18. Impurity Control/Removal
 - a. Filtration
 - 1) Cartridge
 - 2) Deep Bed
 - 3) Precoat

b. Ion Exchange

- 1) Theory
- 2) Single Bed
- 3) Mixed Bed
- 4) Regeneration

c. Blowdown

19. Chemistry Control

- a. Soluble
- b. Volatile

20. Routine Analyses

a. Types

b. Significance

21. Technical Specification Requirements

B. All, or portions, of this section may be waived based on equivalent training, education or experience.

TRAINING PROGRAM DESCRIPTION OPCW2 REVISION 0 PAGE 35 OF 67 DATE: JANUARY 2, 1985 APPENDIX C.5

SCIENCE AND ENGINEERING FUNDAMENTALS

FLUIDS/HEAT TRANSFER/THERMODYNAMICS

- A. Training topics shall cover:
 - 1. Basic Properties of Fluids and Matter
 - 2. Fluid Statics
 - 3. Fluid Dynamics
 - 4. Heat Transfer by Conduction, Convection, and Radiation
 - 5. Change of Phase-Boiling

Training for these topics should include:

- 1. Properties of Fluids
- 2. Gas Laws
- 3. Hydrostatics
- 4. Units of Fluid Flow
- 5. Laminar/Turbulent Flow
- 6. Continuity of Flow
- 7. Bernoulli's Principle
- 8. Fluid Friction/Head Loss
- 9. Flow Measurement

10. Two Phase Flow

a. Water Hammer b. Erosion

- D. EPOSTON
- 11. Pumps
 - a. Types
 - b. Theory of Operation
 - c. Pump Laws
 - d. NPSH
 - e. Head/Losses
 - f. Performance Curves
 - g. Runout

12. Other Forms of Energy

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APPENDIX C.5 (cont.)

SCIENCE AND ENGINEERING FUNDAMENTALS

FLUIDS/HEAT TRANSFER/THERMODYNAMICS

- 13. Heat
 - a. Heat capacity
 - b. Sensible
 - c. Latent
 - d. Saturation
 - e. Superheat
 - f Subcooling
- 14. Thermodynamic Properties of Fluids
- 15. Representations of Properties
 - a. Pvs.V b. Pvs.T c. Tvs.h
 - d. Pvs.h

16. First Law of Thermodynamics

a. Conservation

b. Conversion

17. Second Law of Thermodynamics

- a. Efficiency
- b. Entropy
- 18. TS Diagram/Processes
 - a. Isobaric
 - b. Isothermal
 - c. Adiabatic
 - d. Isenthalpic
 - e. Isentropic

19. HS (Mollier) Diagram/Processes

- a. Isobaric
- b. Isothermal
- c. Adiabatic
- d. Isenthalpic
- e. Isentropic

20. Steam Tables

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APPENDIX C.5 (cont.)

SCIENCE AND ENGINEERING FUNDAMENTALS

FLUIDS/HEAT TRANSFER/THERMODYNAMICS

- 21. Heat Transfer Methods
 - a. Conduction
 - b. Convection
 - c. Radiation
- 22. Heat Exchangers
 - a. Direct/Surface
 - b. Flow Classifications
 - c. Regenerative/Non regenerative
 - d. Components
 - e. Theory of Operation
 - f. Applications
 - g. Calculations

23. Steam Generators

- a. Types
- b. Components
- c. Boiling Heat Transfer
- d. Circulation
- e. Recirculation
- f. Operating Characteristics

24. Condensers

- a. Types
- b. Operation
- c. Efficiency: Effects and Factors
- d. Cooling Systems
- 25. Steam Turbines
 - a. Components
 - b. Types
 - c. Compounding
 - d. Flow Classifications
 - e. Energy Conversion
 - f. Auxiliary/Support Systems
 - g. Hazards
- 26. Plant Cycles
 - a. Carnot
 - b. Ideal Rankine
 - c. Actual Plant
 - d. Efficiency Determination/comparison

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APPENDIX C.5 (cont.)

SCIENCE AND ENGINEERING FUNDAMENTALS

FLUIDS/HEAT TRANSFER/THERMODYNAMICS

- 27. PWR Operation
 - a. Steady state
 - 1) secondary pressure vs. primary temperature
 - 2) primary vs. secondary mass flow rates
 - b. Transients
 - 1) up-power
 - 2) down-power
 - 3) PRZR insurge
 - 4) PRZR outsurge
 - 5) effects on S.G. level
- B. All, or portions, of this section may be waived based on equivalent training, education or experience.

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APPENDIX C.6

SCIENCE AND ENGINEERING FUNDAMENTALS REACTOR THERMODYNAMICS

- A. Training topics shall cover:
 - 1. Burnout and Flow Ductability
 - 2. Reactor Heat Transfer Limits

Training for these topics should include:

- 1. Reactor Heat Generation
 - a. Calorimetric
 - b. Heat Flux
 - c. Linear Heat Rate
 - d. Hot Channel factors
 - e. Peaking Factors
 - f. Power Profiles
 - 1) axial
 - 2) radial
 - 3) poison effects
 - 4) burnup effects

g. Indications

2. Fuel Heat Transfer

a. Fuel temperature/heat flux profiles

-]) axial
- 2) radial
- b. Power transients
- c. Fuel densification
- d. Fuel pellet swelling
- e. Clad creep
- f. Pellet/Clad interactions
- g. Corrosion
- h. Crud/fouling
- i. Gap conductance
- 3. Reactor Thermal-Hydraulics

a. Channel temperature/enthalpy profiles

- 1) axial 2) radial
- b. Boiling
- c. DNB/CHF
- d. DNBR

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APPENDIX C.6

SCIENCE AND ENGINEERING FUNDAMENTALS

REACTOR THERMODYNAMICS

- e. DNBR variations with:
 - 1) RCS flow
 - 2) RCS pressure
 - 3). RCS.temperature
 - 4) Power
- f. Forced convection flow
 - 1) conditions
 - 2) requirements
 - 3) pump operating limits

g. Natural circulation

- 1) theory
- 2) recognition
- 3) limitations
- 4) two phase flow
- 5) vapor binding

4. Limitations

- a. Operational limits
 - 1) total power
 - 2) power distribution
- b. Safety limits
 - 1) technical specifications
 - 2) bases
 - 3) transient challenges
- B. All, or portions, of this section may be waived based on previous equivalent training, education or experience.

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APPENDIX C.7

SCIENCE AND ENGINEERING FUNDAMENTALS.

MECHANICAL FUNDAMENTALS

- A. Training in this area should include:
 - 1. Lubrication
 - a. Lubricant types/characteristics
 - b. Systems
 - 1) Splash
 - 2) Drip
 - 3) Forced
 - 4) Gravity
 - c. Hazards
 - 2. Bearings
 - a. Types
 - b. Applications
 - 3. Valves
 - a. Types
 - b. Characteristics
 - c. Applications
 - 4. Valve Operators
 - a. Manual
 - b. Remote/Automatic
 - 5. Piping Auxiliaries
 - a. Traps
 - b. Strainers/Filters
 - c. Hangers/Snubbers
 - 6. Reading Piping Diagrams (P&ID's)
- B. All, or portions, of this section may be waived based on equivalent training, education or experience.

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APPENDIX C.8

SCIENCE AND ENGINEERING FUNDAMENTALS

ELECTRICAL SCIENCE

- A. Training in this area should include:
 - 1. Atomic Structure
 - 2. Voltage
 - 3. Magnetism/Current Flow
 - 4. Resistance/Conductance
 - 5. Electrical Laws
 - a. Coulomb's Law b. Ohm's Law

6. DC Fundamentals

a. Series Circuitsb. Parallel Circuits

c. Power

I OWEI

7. AC Fundamentals

- a. Effective Current/Voltage
- b. Inductance/Inductive Reactance
- c. Capacitance/capacitive Reactance
- d. Impedance
- e. I/E Phase Relationships

f. Power

8. Buses, Bus Ducts and Disconnects

9. Circuit Breakers and Switchgear

10. AC Generators

11. Excitation Systems

12. Transformers

13. Three Phase Power

14. Power Plant Motors

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APPENDIX C.8 (cont.)

SCIENCE AND ENGINEERING FUNDAMENTALS

ELECTRICAL SCIENCE

- 15. Protective Relaying
- 16. Motor Control
- 17. Storage Batteries
- 18. Inverters
- 19. Power System Operation
 - a. Unit operation
 - b. Synchronization
 - c. Parallel operation

20. Semiconductor Fundamentals

- a. Semi conductors
- b. Diodes
- c. Special devices
- 21. Logic
 - a. Concepts
 - b. Bistables

22. Computer Literacy

23. Hazards and Safety Precautions

24. Electrical and Electronic Prints and Diagrams

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APPENDIX C.9

SCIENCE AND ENGINEERING FUNDAMENTALS

NUCLEAR PHYSICS

A. Training in this area should include:

- 1. Atomic/Nuclear Structure
- 2. Chart of the Nuclides

3. Binding Energy

4. Mass/Energy Conversion

- 5. Radiation
 - a. Types
 - b. Origins
 - c. Interactions

6. Radioactive Decay

7. Induced Nuclear Reactions

8. Fission

9. Microscopic Cross Section

10. Macroscopic Cross Section

11. Neutron Interactions with Reactor Materials

12. Neutron Moderation and Diffusion

APPENDIX C.10

SCIENCE AND ENGINEERING FUNDAMENTALS

REACTOR THEORY

A. Training in this area should include:

1. Neutron Multiplication

2. Multiplication Factors

3. Flux Distribution

4. Neutron Kinetics

5. Reactor Kinetics

6. Reactor Control

7. Coefficients

8. Reactivity Variations

9. Fission Product Poisons

10. Core Characteristics

11. Subcritical Multiplication

12. Power Operations

13. Reactor Shutdown

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APPENDIX C.11

SCIENCE AND ENGINEERING FUNDAMENTALS

RADIATION SCIENCE

A. Training in this area should include:

1. Fundamentals of Health Physics

2. Biological Effects of Ionizing Radiation

3. Principles of Radiation Detection

4. Standards and Regulations

5. ALARA Concepts and Principles

6. Equilibrium

7. Chart of the Nuclides

8. Health Physics Instrumentation

9. Respiratory Protection

10. Contamination and Decontamination

11. Radioactive Waste Management

12. Radiation Protection and Shielding

13. Radioactive Material Classification

14. Area Classification

15. Warning Signs and Equipment

16. Health Physics Workshop

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APPENDIX C.12

SCIENCE AND ENGINEERING FUNDAMENTALS

RADIOCHEMISTRY

A. Training in this area should include:

1. Nuclear Reactions

2. Fission Product Activity

3. Activated Corrosion/Wear Products

4. Activated RCS Impurities

5. Indications of Fuel Damage

6. Fission Product Concentrations

7. Parent-Daughter Relationships

8. Measurement Techniques

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APPENDIX C.13

SCIENCE AND ENGINEERING FUNDAMENTALS

MATERIALS SCIENCE

A. Training in this area should include:

1. States of Matter

2. Structure of the Solid State

3. Properties of the Solid State

- 4. Metallic Structure
- 5. Elastic Deformation
- 6. Plastic Deformation
- 7. Fracture Mechanics

8. Corrosion Effects

- 9. Pressurized Thermal Shock
- B. All, or portions, of this section may be waived based on equivalent training, education or experience.

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APPENDIX C.14

SCIENCE AND ENGINEERING FUNDAMENTALS

INSTRUMENTATION AND CONTROL FUNDAMENTALS

- A. Training in this area should include:
 - 1. Pressure Measurement/Devices
 - 2. Temperature Measurement/Devices
 - 3. Level Measurement/Devices
 - 4. Flow Measurement/Devices
 - 5. Nuclear Instrumentation
 - a. In-Core
 - b. Ex-Core
 - 1) Proportional
 - 2) Compensated Ion Chamber
 - 3) Uncompensated Ion Chamber
 - 4) Fission Chamber
 - 6. Basic Control Loops
 - a. Analog
 - b. Proportional
 - c. Rate
 - d. Gain
 - e. Feed Forward/Back
 - f. Combinations
 - 7. Logic Diagrams
- B. All, or portions, of this section may be waived based on equivalent training, education or experience.

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APPENDIX D.1

PLANT-SPECIFIC TRAINING COURSES

ADMINISTRATIVE REQUIREMENTS

A. Training in this area shall include an overview of the requirements of regulations, orders, policies and procedures, including the Station Administrative Operating Instructions.

B. This section may be waived if previously completed.

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APPENDIX D.2

PLANT-SPECIFIC TRAINING COURSES

PLANT SYSTEMS

- A. Training shall be plant-specific and include, as applicable:
 - 1. Component types/design features
 - 2. Component and control locations
 - 3. Line-ups and flow paths

4. Power supplies

5. Inter-relationships with other systems

6. Automatic features

7. Instrumentation and controls

8. Major parameters

9. Limits, precautions of setpoints

10. Procedures

- a. Normal
- b. Abnormal
- c. Emergency
- d. Surveillance

11. Transient function/response

12. Design bases

13. Technical Specification Requirements

- B. The following lessons should be presented prior to the start of systems training:
 - 1. Introduction to Technical Specifications.
 - 2. Introduction to Piping and Instrumentation Drawings (P & ID's).
- C. The following systems should be presented at a knowledge level consistant with that required for system operation for Units 2 & 3:
 - 1. 200kv Electrical
 - 2. 6.9/4.1kv Electrical
 - 3. 480v Electrical



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APPENDIX D.2 (cont.)

PLANT-SPECIFIC. TRAINING COURSES

PLANT SYSTEMS

4. Non-IE 250/125v DC and 120v AC 5. Class IE 120v AC/125v DC 6. Emergency Diesel Generator 7. Misc. Power Supplies 8. Main Circulating Water 9. Saltwater Cooling 10. Domestic and Service Water 11. Demineralized Water, Makeup and Transfer 12. Nuclear Service Water 13. Condensate and Feedwater 14. Feedwater Pump Turbine 15. Auxiliary Feedwater 16. Steam Generator 17. Feedwater Regulating System 18. Blowdown Processing 19. Main and Reheat Steam 20. Auxiliary Steam/Auxiliary Boiler 21. Heater Drains and Vents 22. Turbine Generator Construction 23. Turbine Mechanical Systems 24. Turbine Gland/Valve Steam Sealing 25. Turbine Lube Oil 26. Generator Seal Oil 27. Generator Gas Cooling 28. Generator Stator Cooling 29. Condenser Air Removal Turbine Supervisory and Protection System 30. 31. Turbine Controls 32. Main Turbine Unitized Actuators 33. Main Generator and 22kv System 34. Turbine Plant Cooling Water 35. Auxiliary Gas (Nitrogen & Hydrogen) 36. Ventilation System 37. Fire Computer Main Condensate and Feedwater Chemical Addition System 38. 39. Auxiliary Feedwater Chemical Addition 40. Fire Detection System 41. Fuel Fabrication/Receipt, Storage 42 Reactor Vessel Internals 43. Spent Fuel Pool Cooling 44. Fuel Handling Building Ventilation 45. Refueling 46. Reactor Coolant System 47. Loose Parts Monitoring System

48. Nuclear Plant Sampling System

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APPENDIX D.2 (cont.)

PLANT-SPECIFIC TRAINING COURSES

PLANT SYSTEMS

49. Chemical and Volume Control System

50. Pressurizer Pressure and Level Control System

51. Control Element Drive Mechanism Control System

52. Reactor Regulating System

53. Steam Bypass Control System

54. Gaseous Radwaste

55. Coolant Radwaste

56. Misc. Liquid Waste System

57. Area & Effluent Rad Monitor System

58. Wide Range Gas Monitor

59. Process Radiation Monitoring System

60. Ventilation Systems

61. Engineered Safety Features

62. Containment Design

63. Containment Air Handling Systems

64. Shutdown Cooling Systems

65. Excore Nuclear Instrumentation

66. Plant Protection System

67. CEAC's and CPC's

68. COLSS

69. Fixed and Moveable Incore NI's

70. Subcooled Margin Monitor

71. Critical Functions Monitoring Systems/QSPDS

72. Plant Computer

73. Component Cooling Water System

74. Post Accident Sampling System

75. Compressed Air Systems

76. Secondary Plant Sampling System

77. Essential Plant Parameters Display

D. The following systems should be presented at a knowledge level consistant with that required for system operation for Unit 1:

1. Steam Generator

2. Main and Auxiliary Steam System

3. Main Steam Dump & Control

4. Feedwater and Condensate

5. Secondary Make-up System

6. Turbine Cycle Vents and Drains

7. Reheaters

8: Reheater Steam Dump

9. Main Circulating Water

10. Turbine Plant Cooling Water

11. Steam Generator Level Control System

12. Auxiliary Feedwater

13. Turbine Mechanical

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APPENDIX D.2 (cont.)

PLANT-SPECIFIC TRAINING COURSES

PLANT SYSTEMS

14. Turbine Controls and Oil Systems

15. Turbine Cycle Sampling

16. Turbine Cycle Chemical Addition

17. Service and Domestic Water

18. Evaporators

19. Screen Wash System

20. Salt Water Cooling System

21. Fire Protection System

22. Instrument and Service Air

23. Lubricating Oil Storage and Transfer

24. Waste Water System

25. Condenser Air Removal

26. Turbine Gland Seal and Exhaust

27. Diesel Generators

28. Diesel Generator Support Systems

29. Main Generator Excitation & Control

30. 220KV System

31. Main Generator Protection

32. Generator Hydrogen

33. Generator Hydrogen Seal Oil

34. 4KV and 480KV Systems

35. Maintained 120VAC

36. Battery and DC Systems

37. Main and Auxiliary Transformers

38. Station Lighting

39. Ventilation

40. Reactor Vessel and Internals

41. Reactor Coolant System

42. Reactor Coolant Pump

43. Pressurizer, Pressure and Level Control

44. CVCS Charging and Letdown

45. CVCS Seal Injection

46. CVCS Boric Acid System

47. Component Cooling Water

48. Safety Injection System

49. Sequencer

50. Containment Spray Hydrazine Injection System

51. Residual Heat Removal

52. Gaseous Radwaste & CVI

53. Liquid Radwaste

54. Spent Fuel Pit and Cooling System

55. Rod Control and Position Indication

56. REactor Control and Protection

57. Nuclear Instrumentation

58. Post Accident Containment Monitoring Systems

59. Primary Make-up Systems

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APPENDIX D.2 (cont.)

PLANT-SPECIFIC TRAINING COURSES

PLANT SYSTEMS

- 60. Refueling and Fuel Handling
- 61. Reactor Coolant System Instrumentation
- 62. Reactor Plant Sampling
- 63. Radiation Monitoring System
- 64. Post-Accident Radiation Monitoring
- 65. Post-Accident Sampling System
- 66. Wide Range Gas Monitor
- 67. Annunicators
- 68. Communication Systems
- 69. Seismic Monitoring
- 70. Weather Station
- 71. Cathodic Protection
- 72. Liquid/Gas N₂
- E. At the completion of presentation of the electrical distribution systems, the candidate should be provided with a tour of the Alhambra Energy Control Center.
- F. If the candidate has previously completed the NPEO training program, presentation of those systems presented at a level consistent with that required for system operation may be waived.

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APPENDIX D.3

PLANT-SPECIFIC TRAINING COURSES

TRANSIENT/ACCIDENT ANALYSIS

A. Training in Transient/Accident Analysis shall be provided. Topics in this area should include:

1. FSAR Overview '

a. Transient analysis

b. Accident analysis (including design basis accident)

2. Facility License/Technical Specifications

a. Overview

b. Safety Limits

c. LCO's

3. Thermal-hydraulic Operating Envelope

4. Accident Identification and Trending

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APPENDIX D.4

PLANT-SPECIFIC TRAINING COURSES

CORE DAMAGE RECOGNITION/MITIGATION

- A. Training in Core Damage Recognition/Mitigation shall be provided. Topics in this area should include:
 - 1. Core Cooling Mechanics
 - 2. Potentially Damaging Conditions
 - 3. Gas/Steam Binding
 - 4. Recognizing Core Damage
 - 5. Recriticality Considerations
 - 6. Hydrogen Generation/Hazards
 - 7. Monitoring of Critical Parameters
 - 8. Radiation Hazards and Monitor Response
 - 9. Cooling Mode Selection/Operation

Α.

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APPENDIX D.5

PLANT-SPECIFIC TRAINING COURSES

INTEGRATED PLANT OPERATIONS

Training should include an overview of the following major integrated plant procedures, including expected plant response and emphasizing the duties and responsibilities of the licensed operator.

1. Plant startup from cold shutdown to hot standby

2. Plant startup from hot standby to minimum load

3. Plant operation from minimum load to full power

4. Plant shutdown from full power to hot standby

5. Plant shutdown from hot standby

6. Reactor trip or safety injection

7. Shutdown from outside the control room

8. Loss of reactor coolant

9. Loss of secondary coolant

10. Steam generator tube rupture

11. Loss of secondary heat sink

12. Malfunction of instrument air system

13. Loss of condenser vacuum

14. Earthquake

15. Fire in 4KV room turbine lube oil reservoir area

16. Loss of offsite power/station blackout

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APPENDIX D.6

PLANT-SPECIFIC TRAINING COURSES

EMERGENCY PLAN

- A. Training should include:
 - 1. Emergency plan overview
 - 2. Classification of emergencies
 - 3. Dose projection source term/dose assessment
 - 4. First aid
- B. The first aid portion of this section may be waived based on equivalent training, education or experience.

APPENDIX D.7

PLANT-SPECIFIC TRAINING COURSES

INDUSTRY/FACILITY EXPERIENCE

- A. Operating experiences to be covered in this training should be designated by the Unit Operator Training Administrator.
- B. Training should include a review of applicable industry and facility operating experiences.
- C. Potential sources of items to be included in this training should include, but not be limited to:
 - 1. Licensee Event Report (LER's)
 - a. On-site
 - b. Other plant sites
 - 2. Station Incident Reports (SIR's)

3. INPO/NSAC, Significant Operating Experience Reports (SOER's)

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APPENDIX E.1

OPERATIONS TRAINING COURSES

EXTRA PERSON ON SHIFT TRAINING (EPOS)

- A plan for the training, to be included in the EPOS phase of the Α. program, should be developed by the Units 2 & 3 Operations Superintendent, or his designee, and the Units 2 & 3 Operations Training Administrator.
- EPOS training should be selected from the following list of topics: Β.
 - 1. Administrative Requirements
 - 1.0 Station Operations
 - 2.0 Use of Procedures
 - 3.0 Duties and Responsibilities
 - 4.0 Notification and Reporting of Significant Events
 - 5.0 Equipment Control Implementation
 - 6.0 Control of System Alignments7.0 Equipment Deficiency Status

 - 8.0 Temporary Facilities Modification Control
 - 9.0 Operations Shift Relief
 - 10.0 Acknowledgement of Information

11.0 Containment Sphere Integrity and Access

2. Primary Plant Systems

13.0 Reactor Coolant System

14.0 Pressurizer and Pressurizer Relief Tank

- 15.0 Reactor Coolant Pump Seal Water System
- 16.0 Safety Injection System
- 17.0 Letdown System

18.0 Volume Control and Charging System

19.0 Boric Acid System

20.0 Residual Heat Removal System

21.0 Reactor Cycle Sampling System

22.0 Containment Spray and Recirculation System

23.0 Post Accident Sampling System

3. Radwaste System

24.0 Radwaste Liquid Collection System

25.0 Radwaste Liquid Processing and Release System

26.0 Radwaste Gas System

27.0 Radwaste Cryogenic Gas Treatment System

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APPENDIX E.1 (Continued)

OPERATIONS TRAINING COURSES

EXTRA PERSON ON SHIFT TRAINING (EPOS)

4. Secondary Plant Systems

28.0 Main Steam System

29.0 Condensate System

30.0 Feedwater System

31.0 Auxiliary Feedwater System

32.0 Feedwater Heaters

33.0 Condenser Air Removal System

34.0 Flash Evaporator System

5. Main Turbine/Generator

35.0 Main Turbine 36.0 Turbine Lube Oil System 37.0 Main Generator and Excitation Systems 38.0 Generator Seal Oil System

6. Electrical Systems

39.0 Diesel Generators
40.0 220kv Buses, 18kv Buses, and Main Transformer
41.0 4kv Buses
42.0 480 Volt Buses
43.0 Vital Buses, Utility Buses and Regulated Buses
44.0 DC Buses 1 & 2
45.0 Plant Lighting System

7. Instrumentation and Control Systems

46.0 Reactor Control and Protection System
47.0 Control Rod Drive System
48.0 Nuclear Instrumentation System
49.0 Reactor Plant Instrumentation
50.0 Auxiliary Control Panel
51.0 Radiation Monitoring System

8. Water Systems

52.0 Circulating Water System 53.0 Component Cooling Water System 54.0 Saltwater Cooling System 55.0 Turbine Plant Cooling Water System 56.0 Spent Fuel Pit Cooling System 57.0 Service and Domestic Water System 58.0 Fire Systems 59.0 Primary Plant Makeup Water System



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APPENDIX E.1 (Continued)

OPERATIONS TRAINING COURSES

EXTRA PERSON ON SHIFT TRAINING (EPOS)

9. Miscellaneous Systems

60.0 Instrument and Service Air System

61.0 Containment Ventilation System

62.0 Miscellaneous Ventilation System

10. Integrated Plant Operations

64.0 Integrated Plant Operations

65.0 Plant Startup from Cold Shutdown to Hot Standby

66.0 Plant Startup from Hot Standby to Minimum Load

.67.0 Plant Operation from Minimum Load to Full Power

68.0 Plant Shutdown from Full Power to Hot Standby

69.0 Plant Shutdown from Hot Standby to Cold Shutdown

70.0 Natural Circulation Operations

71.0 Power Operations

72.0 Critical Safety Function Status Trees

73.0 Emergency Operating Instructions

C. Training topics, listed above, should be maintained current with those listed in the "Table of Contents," Units 2 & 3 Reactor Operator/Assistant Control Operator Qualification Guide, Rev. 0.

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APPENDIX E.2

OPERATIONS TRAINING COURSES

SIMULATOR TRAINING

As described in Section 5.1.4 of this program description, reactivity manipulations and plant evolutions shall be performed at the plant or on an appropriate simulator during initial operator training. Items indicated with an asterisk (*) may be used to meet the requirement. (10CFR55, App. A.3.a)(NUREG 0737, Attachment 4)

*1. Reactor Startups to a point where heatup rate is established.

*2. Manual control of steam generators and/or feedwater during startup and shutdown.

*3. Manual Rod Control for power changes greater than 10%.

- 4. LOCA with significant S/G leakage.
- 5. LOCA inside primary containment.
- 6. LOCA outside primary containment.
- 7. Large LOCA with leak-rate determination.
- 8. Small LOCA with leak-rate determination.
- 9. Saturated RCS response.
- 10. Loss of coolant flow/natural circulation.
- 11. Loss of all feedwater (normal and emergency).
- *12. Plant Shutdown.
- *13. Boration and/or dilution during power operation.
- *14. Any reactor power change of 10% or greater where load change is performed with load limit control.
- *15. Reactor trip.
- 16. Turbine or generator trip.
- 17. Main Steam line break (inside or outside containment).
- 18. Loss of normal feedwater.

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APPENDIX E.2 (Continued) OPERATIONS TRAINING COURSES SIMULATOR TRAINING

- 19. Loss of shutdown cooling (RHR).
- 20. Nuclear Instrumentation System Failure.
- 21. Loss of Reactor Protection System Channel.
- *22. Misaligned Control Rod(s) or dropped rod(s).
- 23. Inability to drive control rods or stuck rod.
- 24. Automatic Control Malfunctions which effect core reactivity.
- *25. Conditions requiring emergency boration.
- 26. RCS or CVCS pressure control system failure.
- 27. Loss of Offsite Power.
- 28. Loss of Vital bus.
- 29. Loss of condenser vacuum.
- 30. Loss of Saltwater Cooling.
- 31. Loss of Component Cooling Water (system or component).
- 32. Loss of Instrument Air.
- 33. Fuel Cladding failure or high activity in reactor coolant.

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APPENDIX E.3

OPERATIONS TRAINING COURSES

CONTROL ROOM OPERATOR SKILLS TRAINING

- A. Training should include the fundamentals of effective speaking, listening and writing.
- B. Completion of the following lessons administered by the Nuclear Training Division satisfies this requirement.

1. Problem Solving

- 2. Communication
 - Face-to-Face
 - Phone
 - Radio
- 3. Tailboarding

4. Procedure Use

- Normal
- Emergency
- Multiple
- 5. Plant Manipulations
- 6. Plant Monitoring
- 7. Annunicator Response
- 8. Log Book Keeping
- C. All, or portions of this section may be waived based on equivalent training or experience.

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APPENDIX F

JOB QUALIFICATION TRAINING

Requirements as identified in the "Units 2 & 3 Assistant Control Operator Qualification Guide."