



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS & ELECTRIC COMPANY

DOCKET NO. 50-206

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 65
License No. DPR-13.

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern California Edison Company and San Diego Gas and Electric Company (the licensees) dated May 7, 1981, as modified March 10, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment of this license amendment and Paragraph 3.B of Provisional Operating License No. DPR-13 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 65, are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective upon issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 22, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 65

PROVISIONAL OPERATING LICENSE NO. DPR-13

DOCKET NO. 50-206

Revised Appendix A Technical Specifications and Bases by inserting the enclosed new pages.

Pages

44l

44m

4.1.9 Auxiliary Feedwater Pumps and Valves

Applicability: Applies to the motor driven auxiliary feedwater pump, the turbine driven auxiliary feedwater pump, and auxiliary feedwater valves for MODES 1, 2 and 3.

Objective: To ensure the reliability of the auxiliary feedwater system.

- Specification:
- A. Each auxiliary feedwater pump shall be demonstrated OPERABLE by testing each pump in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50.55a(g), except where specific written relief has been granted by the NRC pursuant to 10 CFR 50.55a(g)(6)(i).
 - B. At least once per 31 days when the auxiliary feedwater system is required to be operable, an inspection shall be made to verify that normally open manual valves in the auxiliary feedwater system suction piping and the emergency path from the auxiliary feedwater pumps to the steam generators that could interrupt all AFW flow are locked open.
 - C. Each auxiliary feedwater pump shall be demonstrated OPERABLE at least once per 18 months during shutdown by:
 - 1. Verifying that each automatic valve in the flow path actuates to its correct position upon receipt of each auxiliary feedwater actuation test signal.
 - 2. Verifying that each auxiliary feedwater pump starts as designed automatically upon receipt of each auxiliary feedwater actuation test signal.
 - D. At least once per 18 months, all normally closed manual valves in the alternate auxiliary feedwater system suction line shall be demonstrated operable.
 - E. When the reactor coolant system pressure remains less than 500 psig for a period longer than thirty (30) days, a flow test shall be performed to verify the emergency flow path from the condensate storage tank to each steam generator, using the motor driven auxiliary feedwater pump prior to increasing reactor coolant system pressure above 500 psig. The flow test shall be conducted with the auxiliary feedwater system valves in their emergency alignment. As soon as steam becomes available, the steam driven auxiliary feedwater pump shall be similarly tested.

Basis:

The OPERABILITY of the auxiliary feedwater system ensures that the Reactor Coolant System can be cooled down to less than 350°F from normal operating conditions in the event of a total loss of offsite power.

The electric driven auxiliary feedwater pump and the steam driven auxiliary feedwater pump are both capable of delivering a total feedwater flow of 165 gpm at a pressure of 1015 psig to the entrance of the steam generators. This capacity is sufficient to ensure that adequate feedwater flow is available to remove decay heat and reduce the Reactor Coolant System temperature to less than 350°F when the residual Heat Removal System may be placed into operation.

References:

- (1) NRC letter dated July 2, 1980 from D. G. Eisenhut to all pressurized water reactor licensees.