



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTHERN CALIFORNIA EDISON COMPANY

SAN DIEGO GAS & ELECTRIC COMPANY

DOCKET NO. 50-206

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 63
License No. DPR-13

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern California Edison Company and San Diego Gas and Electric Company (the licensees) dated December 24, 1981 as supplemented by letter dated June 10, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Provisional Operating License No. DPR-13 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 63, are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 8, 1982

ATTACHMENT TO LICENSE AMENDMENT NO. 63
PROVISIONAL OPERATING LICENSE NO. DPR-13
DOCKET NO. 50-206

Revised Appendix A Technical Specifications and Bases by removing the following pages and by inserting the enclosed pages. The revised pages contain the captioned amendment number and marginal lines indicating the area of change.

<u>REMOVE</u>	<u>INSERT</u>
39g	39g
39h	39h
Table 3.13.1 (pages 1-6)	Table 3.13.1 (pages 1-4)
--	Table 3.13.2 (pages 1 and 2)
60I	60I
60J	60J
60K	60K-1
--	60K-2
88	88

3.13 SHOCK SUPPRESSORS (SNUBBERS) OPERABILITY

Applicability: Applies to safety related shock suppressors (snubbers) delineated in Tables 3.13.1 and 3.13.2.

Objective: To define operability requirements of snubbers required to protect safety related piping from unrestricted motion when subjected to dynamic loading as might occur during a seismic event or severe transient.

- Specification:
- A. During MODES 1, 2, 3, and 4, all snubbers listed in Tables 3.13.1 and 3.13.2 shall be operable except as noted in 3.13.B through 3.13.D below.
 - B. Within 72 hours of finding one or more snubbers inoperable, replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.14.C on the supported component. Alternatively, declare the supported system inoperable and follow the appropriate Specification for that system.
 - C. If the requirements of A and B above cannot be met, perform the actions required in Section 3.0.
 - D. If a snubber is determined to be inoperable while the reactor is in MODES 5 or 6, the snubber shall be made operable or replaced prior to reactor startup.
 - E. Snubbers may be added to safety related systems without prior License Amendment to Tables 3.13.1 and/or 3.13.2 provided that a revision to Tables 3.13.1 and/or 3.13.2 are included with a subsequent License Amendment request.

Bases:

All snubbers are required to be OPERABLE to ensure that the structural integrity of the reactor coolant system and all other safety related systems is maintained during and following a seismic or other event initiating dynamic loads. Snubbers excluded from this inspection program are those installed on non-safety related systems and then only if their failure, or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

The visual inspection frequency is based upon maintaining a constant level of snubber protection to systems. Therefore, the required inspection interval varies inversely with the observed snubber failures and is determined by the number of inoperable snubbers found during an inspection. Inspections performed before that interval has elapsed may be used as a new reference point to determine the next inspection. However, the results of such early inspections performed before the original required time interval has elapsed (nominal time less 25%) may not be used to lengthen the required inspection interval. Any inspection whose results require a shorter inspection interval will override the previous schedule.

When the cause of the rejection of a snubber is clearly established and remedied for that snubber and for any other snubbers that may be generically susceptible, and verified by inservice functional testing, that snubber may be exempted from being counted as inoperable. Generically susceptible snubbers are those which are of a specific make or model and have the same design features directly related to rejection of the snubber by visual inspection, or are similarly located or exposed to the same environmental conditions such as temperature, radiation, and vibration.

When a snubber is found inoperable, an engineering evaluation is performed, in addition to the determination of the snubber mode of failure, in order to determine if any safety-related component or system has been adversely affected by the inoperability of the snubber. The engineering evaluation shall determine whether or not the snubber mode of failure has imparted a significant effect or degradation on the supported component or system.

To provide assurance of snubber functional reliability, a representative sample of the installed snubbers will be functionally tested during plant shutdowns at refueling outage intervals. Observed failures of these sample snubbers shall require functional testing of additional units. Snubbers of rated capacity greater than 50,000 pounds are exempt from functional testing requirements because of the impracticability of testing such large units.

Hydraulic snubbers and mechanical snubbers may each be treated as a different entity for the above surveillance programs.

The service life of a snubber is evaluated via manufacturer input and information through consideration of the snubber service conditions and associated installation and maintenance records (newly installed snubber, seal replaced, spring replaced, in high radiation area, in high temperature area, etc...). The requirement to monitor the snubber service life is included to ensure that the snubbers periodically undergo a performance evaluation in view of their age and operating conditions. These records will provide statistical bases for future consideration of snubber service life. The requirements for the maintenance of records and the snubber service life review are not intended to affect plant operation.

SAFETY RELATED HYDRAULIC SNUBBERS

<u>SNUBBER No.</u>	<u>LOCATION</u>	<u>LINE No.</u>	<u>SNUBBERS IN HIGH RADIATION AREA DURING SHUTDOWN*</u>	<u>SNUBBERS ESPECIALLY DIFFICULT TO REMOVE (YES OR NO)</u>	<u>CATEGORY**</u>	<u>SNUBBERS ACCESSIBLE OR INACCESSIBLE (A OR I)</u>			
1-S-SW-1-1	Main Steam Piping System	1	YES	YES		I			
1-S-SW-1-2			YES	YES		I			
1-S-SW-1-3			YES	YES		I			
1-S-SW-1-4			NO	NO		A			
2-S-SW-1-5			NO	NO		A			
2-S-SW-1-8			NO	NO		A			
2-S-SW-1-9			NO	NO		A			
2-S-SW-1-10			NO	NO		A			
2-S-SW-1-11									
1-SW-2-1			Main Steam Piping System	2		YES	YES		I
1-SW-2-2						YES	YES		I
1-SW-2-3	YES	YES			I				
1-SW-2-4	NO	NO			A				
2-SW-2-5	NO	NO			A				
2-SW-2-8	NO	NO			A				
2-SW-2-9	NO	NO			A				
2-SW-2-10	NO	NO			A				
2-SW-2-11									
1-SW-3-1	Main Steam Piping System	3			YES	YES			I
1-SW-4-1	Main Steam Piping System	4			YES	YES			I
1-SW-5-1	Main Steam Piping System	5	YES	YES		I			

<u>SNUBBER No.</u>	<u>LOCATION</u>	<u>LINE No.</u>	<u>SNUBBERS IN HIGH RADIATION AREA DURING SHUTDOWN*</u>	<u>SNUBBERS ESPECIALLY DIFFICULT TO REMOVE (YES OR NO)</u>	<u>CATEGORY**</u>	<u>SNUBBERS ACCESSIBLE OR INACCESSIBLE (A OR I)</u>
1-S-SW-6, 7-1	Main Steam Piping System	6		YES		I
1-S-SW-6-1			YES		I	
1-S-SW-6-2			YES		I	
1-S-SW-6-3			YES		I	
1-S-SW-6-4			YES		I	
1-S-SW-6-5			YES		I	
1-S-SW-6-6			YES		I	
1-S-SW-7-1	Main Steam Piping System	7		YES		I
1-S-SW-7-2			YES		I	
1-S-SW-7-3			YES		I	
1-S-SW-7-4			YES		I	
1-S-SW-7-5			YES		I	
1-S-SW-7-6			YES		I	
2-9-SW-5	Main Steam Piping System	9		NO		A
6-15-SW-5	Main Steam Piping System	15		NO		A
6-15-SW-6			NO		A	
2-S-SW-17-1	Main Steam Piping System	17		NO		A
2-S-SW-17-2			NO		A	
2-S-SW-17-4			NO		A	
5-S-17-SW-6			NO		A	
5-S-17-SW-9			NO		A	
2-S-SW-18-1	Main Steam Piping System	18		NO		A
2-S-SW-18-2			NO		A	
2-S-SW-18-4			NO		A	
6-S-18-SW-6			NO		A	
6-S-18-SW-8			NO		A	
2-S-50-SW-1	Main Steam Piping System	50		NO		A
2-S-50-SW-2			NO		A	

<u>SNUBBER No.</u>	<u>LOCATION</u>	<u>LINE No.</u>	<u>SNUBBERS IN HIGH RADIATION AREA DURING SHUTDOWN*</u>	<u>SNUBBERS ESPECIALLY DIFFICULT TO REMOVE (YES OR NO)</u>	<u>CATEGORY**</u>	<u>SNUBBERS ACCESSIBLE OR INACCESSIBLE (A OR I)</u>
2-S-51-SW-1 2-S-51-SW-2	Main Steam	51		NO NO		A A
1-S-SW-391-1 1-S-SW-391-2 1-S-SW-391-3 1-S-SW-391-4	Feedwater Piping System	391		YES YES YES YES		I I I I
1-S-SW-392-1 1-S-SW-392-2 1-S-SW-392-3 1-S-SW-392-4 1-S-SW-392-5	Feedwater Piping System	392		YES YES YES YES YES		I I I I I
1-S-SW-393-1 1-S-SW-393-2 1-S-SW-393-3 1-S-SW-393-4	Feedwater Piping System	392		YES YES YES YES		I I I I
1-SW-734-2	Containment Spray System	734		YES		I
T-8-PS-27	Refueling Water Piping System	734 & 747		NO		A
1-S-735-SW-1	Refueling Water Piping System	735		YES	2	I
1-SC-602R-6	Pressurizer Relief Piping System	5035		YES		I

<u>SNUBBER No.</u>	<u>LOCATION</u>	<u>LINE No.</u>	<u>SNUBBERS IN HIGH RADIATION AREA DURING SHUTDOWN*</u>	<u>SNUBBERS ESPECIALLY DIFFICULT TO REMOVE (YES OR NO)</u>	<u>CATEGORY**</u>	<u>SNUBBERS ACCESSIBLE OR INACCESSIBLE (A OR I)</u>
5-SW-151R-10	Safety Injection	6002		NO	2	A
5-SW-151R-11	Piping System			NO	2	A
6-151R-13	Safety Injection	6003		NO	2	A
6-151R-14	Piping System			NO	2	A
5-S-1501R-40	Safety Injection	6004		NO		A
5-S-1501R-41				NO		A
5-SS-1501R-9				NO		A
6-1501-R-42	Safety Injection	6005		NO		A
6-1501-R-43	Piping System			NO		A
1-SS-1501R-4	Safety Injection	6006		YES		I
1-SS-1501R-5	Piping System			YES		I
1-SS-1501R-3				YES		I
1-SS-1501R-6	Safety Injection	6007		YES		I
1-SS-1501R-7	Piping System			YES		I
1-SS-1501R-1	Safety Injection	6008		YES		I
1-SS-1501R-2	Piping System			YES		I
E-1A	Steam Generators			YES	2	I
E-1B				YES	2	I
E-1C				YES	2	I

* Modifications to this column due to changes in high radiation areas may be made without prior License Amendment provided that a revision to Table 3.13.2 is included with the next License Amendment Request.

** CATEGORY - 1. First snubber away from each reactor vessel nozzle
2. Snubber within 5 feet of heavy equipment.
3. Snubber within 10 feet of the discharge from a safety relief valve.

TABLE 3.13.2

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SAFETY RELATED MECHANICAL SNUBBERS

<u>SNUBBER No.</u>	<u>LOCATION</u>	<u>LINE No.</u>	<u>SNUBBERS IN HIGH RADIATION AREA DURING SHUTDOWN*</u>	<u>SNUBBERS ESPECIALLY DIFFICULT TO REMOVE (YES OR NO)</u>	<u>CATEGORY**</u>	<u>SNUBBERS ACCESSIBLE OR INACCESSIBLE (A OR I)</u>
5-318-SW-10 5-318-SW-11	Safety Injection Piping System	318		NO NO		A A
1-6019-SS-2 1-734-SS-003	Safety Injection Sphere Spray	734 734		NO YES		A I
SA-16-1 SA-16-2	No. 1 D/G Intake	1540 1542		NO NO		A A
SA-16-3 SA-16-4 SA-16-5 SA-16-6	No. 1 D/G Exhaust	1544		NO NO NO NO	3	A A A A
SA-17-1 SA-17-2	No. 2 D/G Intake	1547 1549		NO NO		A A
SA-17-3 SA-17-4 SA-17-5 SA-17-6	No. 2 D/G Exhaust	1551		NO NO NO NO	3 3	A A A A
SA-1	Safety Injection Piping System	3122		NO		A
5-SS-151R-11 14-PS-2A 6-SW-151R-15 14-PS-2B,	Safety Injection Piping System	6002 6003		NO NO NO NO		A A A A

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Table 3.13.2 (Cont'd)

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SNUBBER No.	LOCATION	LINE No.	SNUBBERS IN HIGH RADIATION AREA DURING SHUTDOWN*	SNUBBERS ESPECIALLY DIFFICULT TO REMOVE	CATEGORY**	SNUBBERS ACCESSIBLE OR INACCESSIBLE
				(YES OR NO)		(A OR I)
2-SC-1501R-03	Safety Injection	6005				A
6-SC-1501R-02	Piping System					A
1-SS-1501-11		6006				A
1-SS-1501-R-14				YES	1	I
1-SS-1501-R-10		6008		YES		I
1-SS-1501-R-13				YES	1	I
1-SS-1501-12	Safety Injection	6007		YES	1	I
1-SS-1501-R-15	Piping System			YES		I
1-6018-SS-006	Safety Injection	6018		YES		I
	Piping System			YES		I
1-6019-SS-006	Safety Injection	6019		YES		I
	Piping System					I

* Modifications to this column due to changes in high radiation areas may be made without prior License Amendment provided that a revision to Table 3.13.2 is included with the next License Amendment Request.

** CATEGORY - 1. First snubber away from each reactor vessel nozzle
 2. Snubber within 5 feet of heavy equipment.
 3. Snubber within 10 feet of the discharge from a safety relief valve.

4.14 SHOCK SUPPRESSORS (SNUBBERS) SURVEILLANCE

Applicability: Applies to surveillance of safety related snubbers delineated in Tables 3.13.1 and 3.13.2

Objective: To ensure the operability of safety related snubbers.

Specification: Each snubber listed in Tables 3.13.1 and 3.13.2 shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 3.13.

A. Visual Inspections

Visual inspections shall be performed in accordance with the following schedule:

<u>No. of Snubbers Found Inoperable During Inspection or During Inspection Interval</u>	<u>Subsequent Visual Inspection Period*</u>
0	18 months + 135 days
1	12 months + 90 days
2	6 months + 45 days
3, 4	124 days + 31 days
5, 6, 7	62 days + 16 days
8 or more	31 days + 8 days

The snubbers may be categorized into two groups: Those accessible and those inaccessible during normal reactor operation. Each group may be inspected independently in accordance with the above schedule. Further, the subgroups - mechanical and hydraulic - may be inspected independently.

All hydraulic snubbers, with seal material not fabricated from ethylene propylene or other materials demonstrated compatible with the operating environment, shall be visually inspected for operability at intervals of roughly half the duration that the seal material has been demonstrated to remain operable or the Visual Inspection Period, if less. For materials of uncertain compatibility with the environment, the seals shall be visually inspected every 31 + 8 days. (As of mid-1981, there were no such seals.)

*The inspection interval shall not be lengthened more than one step at a time.

B. Visual Inspection Acceptance Criteria

Visual inspections shall verify (1) that there are no visible indications of damage or impaired OPERABILITY, (2) attachments to those foundation or supporting structure are secure, and (3) in those locations where snubber movement can be manually induced without disconnecting the snubber, that the snubber has freedom of movement and is not frozen up. Snubbers which appear inoperable as a result of visual inspections may be determined OPERABLE for the purpose of establishing the next visual inspection interval, providing that (1) the cause of the rejection is clearly established and remedied for that particular snubber and for other snubbers that may be generically susceptible; and (2) the affected snubber is functionally tested in the as found condition and determined OPERABLE per Specifications 4.14.D or 4.14.E, as applicable.

C. Functional Tests

At least once per refueling cycle shutdown, a representative sample (10% of the total, the next larger integer number, of each type of snubber in use in the plant) shall be functionally tested either in place or in a bench test. For each snubber that does not meet the functional test acceptance criteria of Specification 4.14.D or 4.14.E, an additional 10% of that type of snubber shall be functionally tested.

The representative sample selected for functional testing shall include the various configurations, operating environments and the range of size and capacity of snubbers, exempting snubbers of capacity greater than 50,000 pounds, i.e., E-1A, E-1B and E-1C. At least 25% of the snubbers in the representative sample shall include snubbers from the following three categories:

1. The first snubber away from each reactor vessel nozzle
2. Snubbers within five feet of heavy equipment (valve, pump, turbine, motor, etc.)
3. Snubbers within ten feet of the discharge from a safety relief valve

Snubbers identified in Table 3.13.1 and 3.13.2 as "Especially Difficult to Remove" or in "High Radiation Zones During Shutdown" shall also be included in the representative sample. Tables 3.13.1 and 3.13.2 may be used jointly or separately as the basis for the sampling plan.

In addition to the regular sample, snubbers which failed the previous functional test shall be retested during the next test period. If a spare snubber has been installed in place of a failed snubber, then both the failed snubber (if it is repaired and installed in another position) and the spare snubber shall be retested. Test results of these snubbers may not be included for the re-sampling.

If any snubber selected for functional testing either fails to lockup or fails to move, i.e., frozen in place, the cause will be evaluated and if caused by manufacturer or design deficiency, all snubbers of the same design subject to the same defect shall be functionally tested. This testing requirement shall be independent of the requirements stated above for snubbers not meeting the functional test acceptance criteria.

For the snubber(s) found inoperable, an engineering evaluation shall be performed on the components which are supported by the snubber(s). The purpose of this engineering evaluation shall be to determine if the components supported by the snubber(s) were adversely affected by the inoperability of the snubber(s) in order to ensure that the supported component remains capable of meeting the designed service.

D. Hydraulic Snubbers Functional Test Acceptance Criteria

The hydraulic snubber functional test shall verify that:

1. Activation (restraining action) is achieved within the specified range of velocity or acceleration in both tension and compression.
2. Snubber bleed, or release rate, where required, is within the specified range in compression or tension. For snubbers specifically required to not displace under continuous load, the ability of the snubber to withstand load without displacement shall be verified.

E. Mechanical Snubbers Functional Test Acceptance Criteria

The mechanical snubber functional test shall verify, after visual examination for damage, that the snubbers have freedom of movement by performing a manual test over the range of stroke in both compression and tension. (Per IE Bulletin 81-01)

F. Snubber Service Life Monitoring

A record of the service life of each snubber, the date at which the designated service life commences and the installation and maintenance records on which the designated service is based shall be maintained as required by Specification 6.10.2.k.

Concurrent with the first inservice visual inspection and at least once per refueling outage thereafter, the installation and maintenance records for each snubber listed in Tables 3.13.1 and 3.13.2 shall be reviewed to verify that the indicated service life has not been exceeded or will not be exceeded prior to the next scheduled snubber service life review. If the indicated service life will be exceeded prior to the next scheduled snubber service life review, the snubber service life shall be reevaluated or the snubber shall be replaced or reconditioned so as to extend its service life beyond the date of the next scheduled service life review. This reevaluation, replacement or reconditioning shall be indicated in the records.

Basis:

Refer to the basis given in 3.13.

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- f. Records of in-service inspections performed pursuant to these Technical Specifications.
- g. Records of Quality Assurance activities as required by the QA Manual.
- h. Records of reviews performed for changes made to procedures or equipment or reviews or tests and experiments pursuant to 10 CFR 50.59.
- i. Records of meetings of the OSRC and the NARC.
- j. Records for Environmental Qualification which are covered under the provisions of paragraph 6.12.
- k. Records of the service lives of all hydraulic and mechanical snubbers listed in Tables 3.13.1 and 3.13.2, including the date at which the service life commences and associated installation and maintenance records.

6.10.3 The following records shall be retained for two years.

- a. Records of facility radiation and contamination surveys.
- b. Records of training of facility personnel.