



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SOUTHERN CALIFORNIA EDISON COMPANY AND
SAN DIEGO GAS AND ELECTRIC COMPANY

DOCKET NO. 50-206

SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 1

AMENDMENT TO PROVISIONAL OPERATING LICENSE

Amendment No. 54
License No. DPR-13

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Southern California Edison Company and San Diego Gas and Electric Company (the licensees) dated August 27, 1980, and February 2, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public; and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Provisional Operating License No. DPR-13 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 54, are hereby incorporated in the license. Southern California Edison Company shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 7, 1981

ATTACHMENT TO LICENSE AMENDMENT NO. 54

PROVISIONAL OPERATING LICENSE NO. DPR-13

DOCKET NO. 50-206

Revise Appendix A Technical Specifications and Bases and Appendix B Technical Specifications by removing the following pages and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain vertical lines indicating the areas of change.

	<u>PAGES*</u>
Appendix A	69
	70
	70a
	72a
	73
	74
	75
	79
Appendix B	5-1
	5-2
	5-3
	5-4
	5-5

*Pages 45 and 46c are included to clarify the primary and secondary enumeration of Section 4.2, which is necessary as a result of Technical Specification provisions issued by NRC Order dated April 20, 1987.

4.2 SAFETY INJECTION AND CONTAINMENT SPRAY SYSTEM

4.2.1 SAFETY INJECTION AND CONTAINMENT SPRAY SYSTEM PERIODIC TESTING

Applicability

Applies to testing of the Safety Injection System and the Containment Spray System.

Objective:

To verify that the Safety Injection System and the Containment Spray System will respond promptly and properly if required.

Specification:

I. System Test

A. Safety Injection System

- (1) During reactor shutdown at intervals not longer than the normal plant refueling intervals, a "no-flow" system test shall be conducted to demonstrate proper availability of the system. The test shall be performed in accordance with the following procedure:
 - (a) The feedwater, safety injection, charging, condensate, and heater drain pumps shall not be operating. Their respective breakers shall be racked-out to the test position with control power available.
 - (b) The flow path for condensate shall be positively blocked prior to the test.
 - (c) Injection and recirculation system operation shall be initiated by instrumentation and controls installed in the control room.
- (2) The test will be considered satisfactory if control board indication and visual observations indicate all components have operated and sequenced properly. That is, the appropriate pump breakers have opened and closed, and all valves have completed their travel.
- (3) A test of the trisodium phosphate additive shall be conducted to demonstrate the availability of the system. The test shall be performed in accordance with the following procedure:
 - (a) The three (3) storage racks are visually observed to have maintained their integrity.
 - (b) The three (3) racks, each with a storage capacity of 1800 pounds of anhydrous trisodium phosphate additive, are visually observed to be full.

4.2.2 Primary Coolant System Pressure Isolation Valves Testing
(Surveillance Requirement)

Applicability

Operational Modes 1, 2, 3 (Power Operation, Startup, and Hot Shutdown) applies to the operational status of the primary coolant system pressure isolation valves.

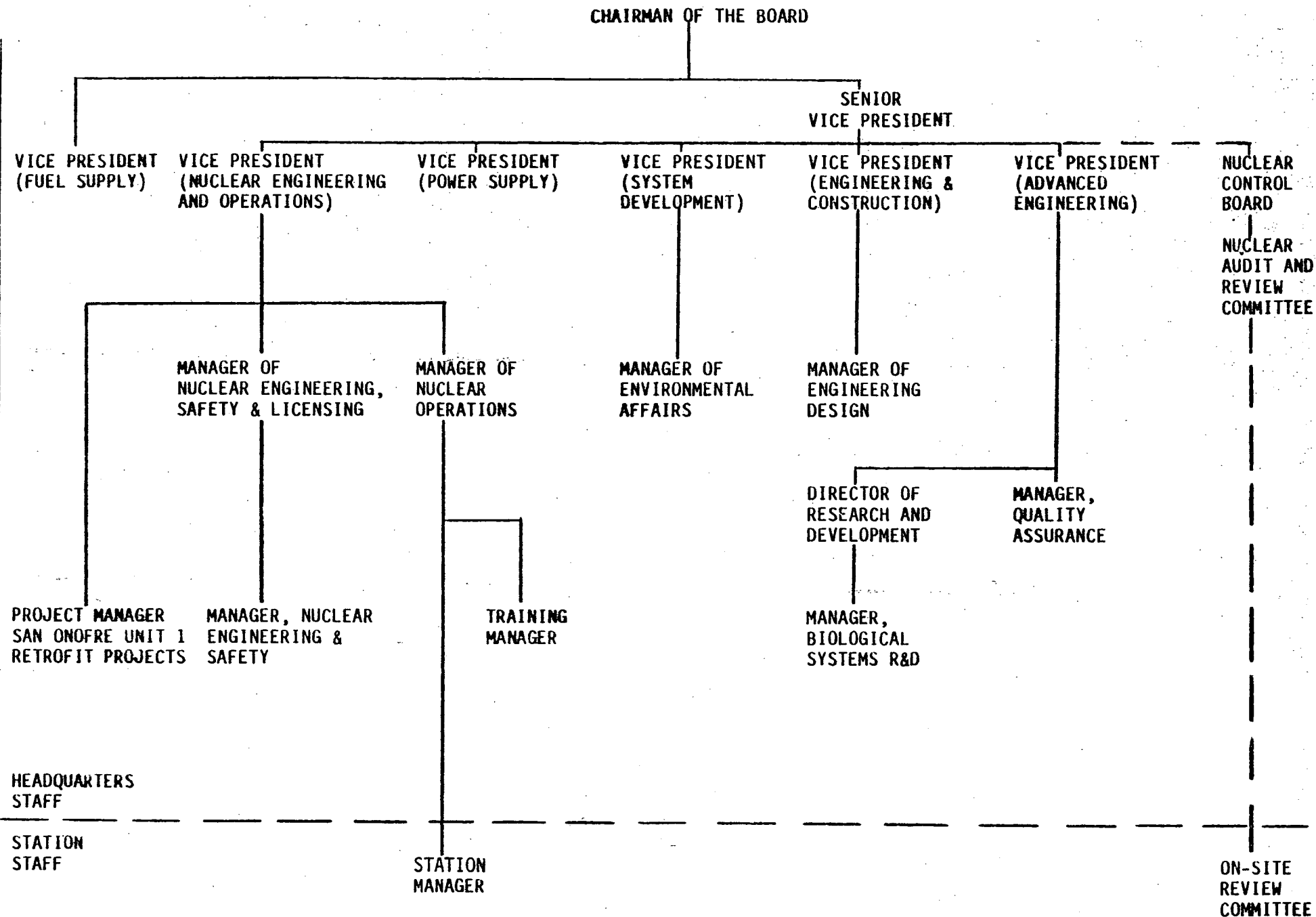
Objective:

To increase the reliability of primary coolant system pressure isolation valves thereby reducing the potential of an inter-system loss of coolant accident.

Specification:

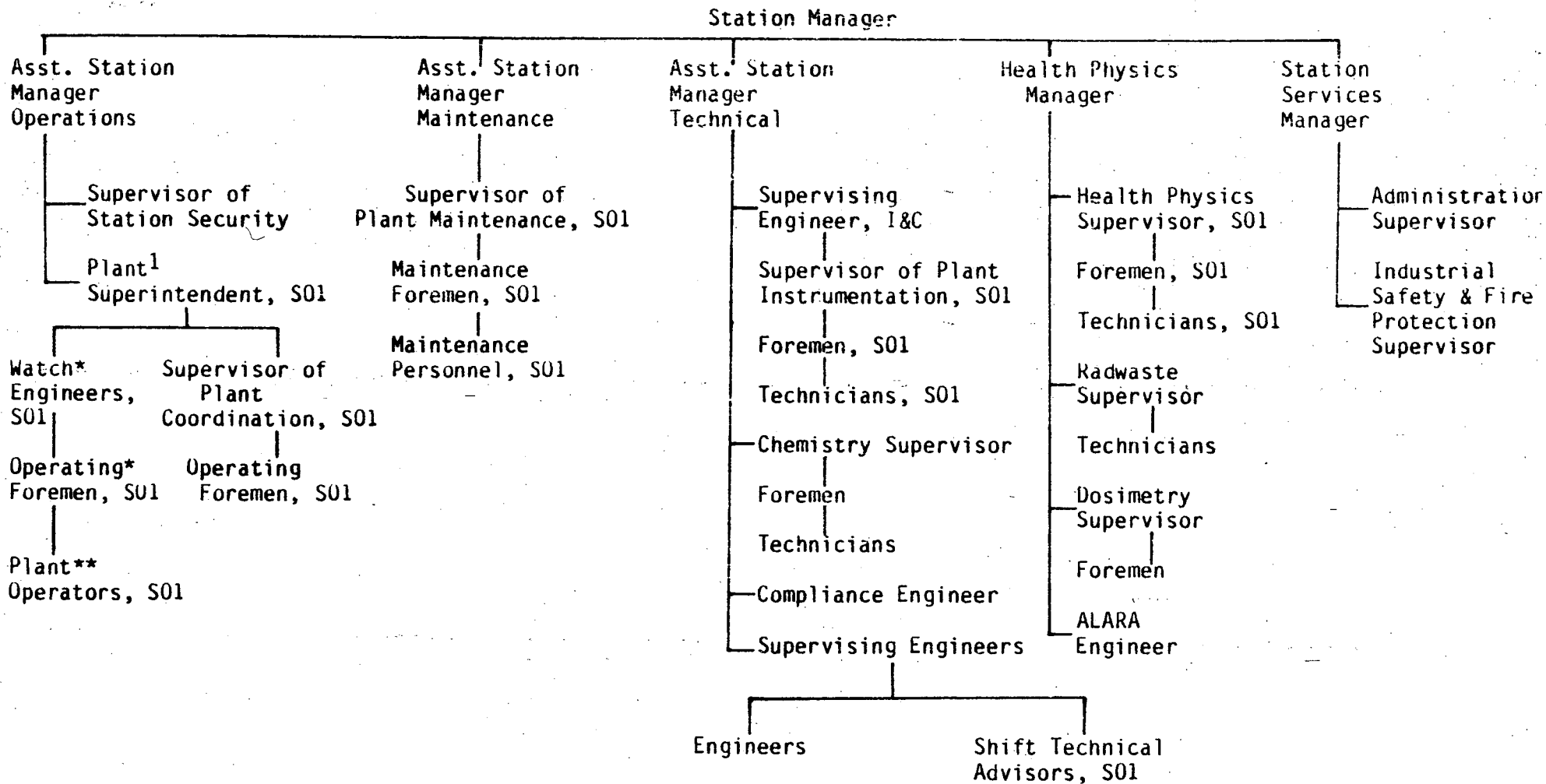
1. Periodic leakage testing ^(a) on each valve listed in Table 3.3.4-1 shall be accomplished every time the plant is placed in the cold shutdown condition for refueling, each time the plant is placed in a cold shutdown condition for 72 hours if testing has not been accomplished in the preceding 9 months, and prior to returning the valve to service after maintenance, repair or replacement work is performed.

(a) To satisfy ALARA requirements, leakage may be measured indirectly (as from the performance of pressure indicators) if accomplished in accordance with approved procedures and supported by computations showing that the method is capable of demonstrating valve compliance with the leakage criteria. The minimum test differential pressure shall not be less than 150 psid.



6.2.1.1 OFFSITE ORGANIZATION

FIGURE 6.2.2.2



¹ At time of appointment to the position, Senior Reactor Operator License Required

* Senior Reactor Operator License Required

** Control and Assistant Control Operators are holders of Reactor Operator Licenses

SENIOR VICE PRESIDENT

VICE PRESIDENT
POWER SUPPLY DEPARTMENT

VICE PRESIDENT
NUCLEAR ENGINEERING & OPERATIONS

MANAGER, POWER
SUPPLY SERVICES

CORPORATE FIRE
PREVENTION COMMITTEE

ADMINISTRATOR
POWER SUPPLY
SERVICES

FIRE CONTROL AND
CONSERVATION
REPRESENTATIVE

FIRE CONTROL
CONSULTANT

MANAGER OF
NUCLEAR OPERATIONS

HEADQUARTERS

FIELD

FIRE EQUIPMENT
INSPECTORS

ASST. STATION
MANAGER
OPERATIONS

ASST. STATION
MANAGER
MAINTENANCE

STATION
MANAGER

STATION
SERVICES
MANAGER

PLANT
SUPERINTENDENT, S01

SUPERVISOR
OF PLANT
MAINTENANCE, S01

INDUSTRIAL SAFETY
& FIRE PROTECTION
SUPERVISOR

PLANT
FIRE
BRIGADE, S01

STATION
FIRE
COMMITTEE

FIGURE 6.2.3.1

FIRE PROTECTION STAFF AND LINE ORGANIZATIONS

6.3 Facility Staff Qualification

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971, "Selection and Training of Personnel for Nuclear Power Plants," for comparable positions. The Health Physics Manager shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, September, 1975."

6.4 Training

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.
- 6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Training Manager and shall meet or exceed the requirements of Section 27 of the National Fire Protection Association Code - 1976."

6.5 REVIEW AND AUDIT

6.5.1 ONSITE REVIEW COMMITTEE (OSRC)

FUNCTION

6.5.1.1 The OSRC shall function to advise the Station Manager on all matters related to nuclear safety.

COMPOSITION:

"6.5.1.2 The OSRC shall be composed of the:

Chairman: Station Manager
Member: Assistant Station Manager, Operations
Member: Assistant Station Manager, Technical
Member: Plant Superintendent, S01
Member: Supervising Engineer, I&C
Member: Health Physics Manager
Member: Chemistry Supervisor
Member: Assistant Station Manager, Maintenance
Member: Engineer
Member: San Diego Gas & Electric Representative"

ALTERNATES

6.5.1.3 Alternate members shall be appointed in writing by the OSRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in OSRC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The OSRC shall meet at least once per calendar month and as convened by the OSRC Chairman.

QUORUM:

"6.5.1.5 A quorum of the OSRC shall consist of the Chairman or his designated alternate and three members, including alternates."

RESPONSIBILITIES

6.5.1.6 In performing their duties with regard to these Appendix A Technical Specifications, the OSRC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.

- c. Review of all proposed changes to the Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and preparation and forwarding of a report covering evaluation and recommendations to prevent recurrence to the Manager of Nuclear Operations and to the Chairman of the Nuclear Audit and Review Committee."
- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Nuclear Audit and Review Committee.
- h. Review of the Plant Security Plan and implementation procedures at least once each two years.
- i. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Nuclear Audit and Review Committee.

AUTHORITY

6.5.1.7 The OSRC shall:

- a. Recommend to the Station Manager written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations to the Station Manager in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide immediate written notification to the Manager of Nuclear Operations and the Chairman of Nuclear Audit and Review Committee of disagreement between the OSRC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above."

RECORDS

"6.5.1.8 The OSRC shall maintain written minutes of each meeting and copies shall be provided to the Manager of Nuclear Operations and Chairman of the Nuclear Audit and Review Committee."

6.5.2 NUCLEAR AUDIT AND REVIEW COMMITTEE (NARC)

FUNCTION

6.5.2.1 The NARC provides independent review and audit of designated activities in the area of nuclear safety (see Appendix A, DPR-13) and environmental impact (see Appendix B, DPR-13).

COMPOSITION:

"6.5.2.2 The NARC shall be composed of the:

Manager of Engineering Design
Manager of Environmental Affairs
Manager of Nuclear Engineering, Safety & Licensing
Manager, Quality Assurance
Manager of Nuclear Operations
Manager, Nuclear Engineering & Safety
Manager, Biological Systems Research and Development
San Diego Gas & Electric Representative

Chairmanship shall be designated by the Nuclear Control Board."

ALTERNATES

6.5.2.3 Alternate members shall be appointed in writing by the NARC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in NARC activities at any one time.

CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the NARC Chairman.

MEETING FREQUENCY

6.5.2.5 The NARC shall meet at least once per six months.

- b. Review and approve recommended changes to the Technical Specifications.
- c. Submit proposed changes to the Technical Specifications to the Commission.
- d. Maintain management control with respect to nuclear safety.

6.6

DELETED

6.7

SAFETY LIMIT VIOLATION

6.7.1

The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36 (c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported to the Commission, the Manager of Nuclear Operations and to the NARC Chairman immediately."
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the OSRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures, and (3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the Commission, the NARC and the Manager of Nuclear Operations, within 14 days of the violation."

6.8

PROCEDURES

6.8.1

Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Sections 5.1 and 5.3 of ANSI N18.7-1976, Administrative Controls for Nuclear Power Plants; Appendix "A" of USNRC Regulatory Guide 1.33, Rev. 1, Quality Assurance Program Requirements (Operation); and Paragraph 2.2.1 of Fire Protection Program Review, STP APCS 9.5-1, San Onofre Nuclear Generating Station, Unit 1, March, 1977; except as provided in 6.8.2 and 6.8.3 below.

6.8.2

Each procedure and administrative policy of 6.8.1 above and changes thereto shall be reviewed by the OSRC and approved by the Station Manager prior to implementation.

5.0

ADMINISTRATIVE CONTROLS

5.1

Responsibility

The Southern California Edison Company shall be solely responsible for the operation of San Onofre Nuclear Generating Station Unit 1. Management responsibility at the station and corporate levels shall be as specified in Section 5.3: and shall ensure that these environmental technical specifications are fully implemented and that continued protection to the environment shall be provided at all times. Responsibility shall be exercised at the corporate level to provide that the group, individual or organization assigned to audit or otherwise verify that an activity has been correctly performed is independent of the individual, group or organization directly responsible for performing the specific activity. Where other organizations are delegated responsibility for executing portions of these environmental technical specifications, the Southern California Edison Company shall retain responsibility for all such activities. A detailed description of the program utilized to audit or otherwise verify that an activity has been correctly performed shall be developed and submitted to the Directorate of Licensing for review within 4 months of the effective date of these environmental technical specifications. The audit and verification program shall be established and conducted in accordance with this program description.

5.2

Organization

5.2.1

The corporate organization, which provides the line responsibility for the operation of the Station and the staff responsibilities in support of these operations, is depicted in Figure 5.2-1.

5.2.2

The Station organization utilized in the operation and maintenance of the Station is depicted in Figure 5.2-2.

"5.2.3

The Health Physics Manager and the Chemistry Supervisor shall be the Onsite Review Committee members responsible for environmental issues. These members shall be qualified in environmental matters."

5.3

Review and Audit

Review and audit of Station operations shall be provided by the Onsite Review Committee, the Nuclear Audit and Review Committee, the Nuclear Control Board, and the Quality Assurance Organization.

5.3.1

The membership of the Onsite Review Committee and the responsibilities and authorities of the Committee with respect to these environmental technical specifications are as follows:

a. Membership

- (1) Station Manager (Chairman)
- (2) Assistant Station Manager, Operations
- (3) Assistant Station Manager, Technical
- (4) Plant Superintendent, SO1
- (5) Supervising Engineer, I&C
- (6) Health Physics Manager
- (7) Chemistry Supervisor
- (8) Assistant Station Manager, Maintenance
- (9) Engineer
- (10) San Diego Gas & Electric Representative

(Correction)

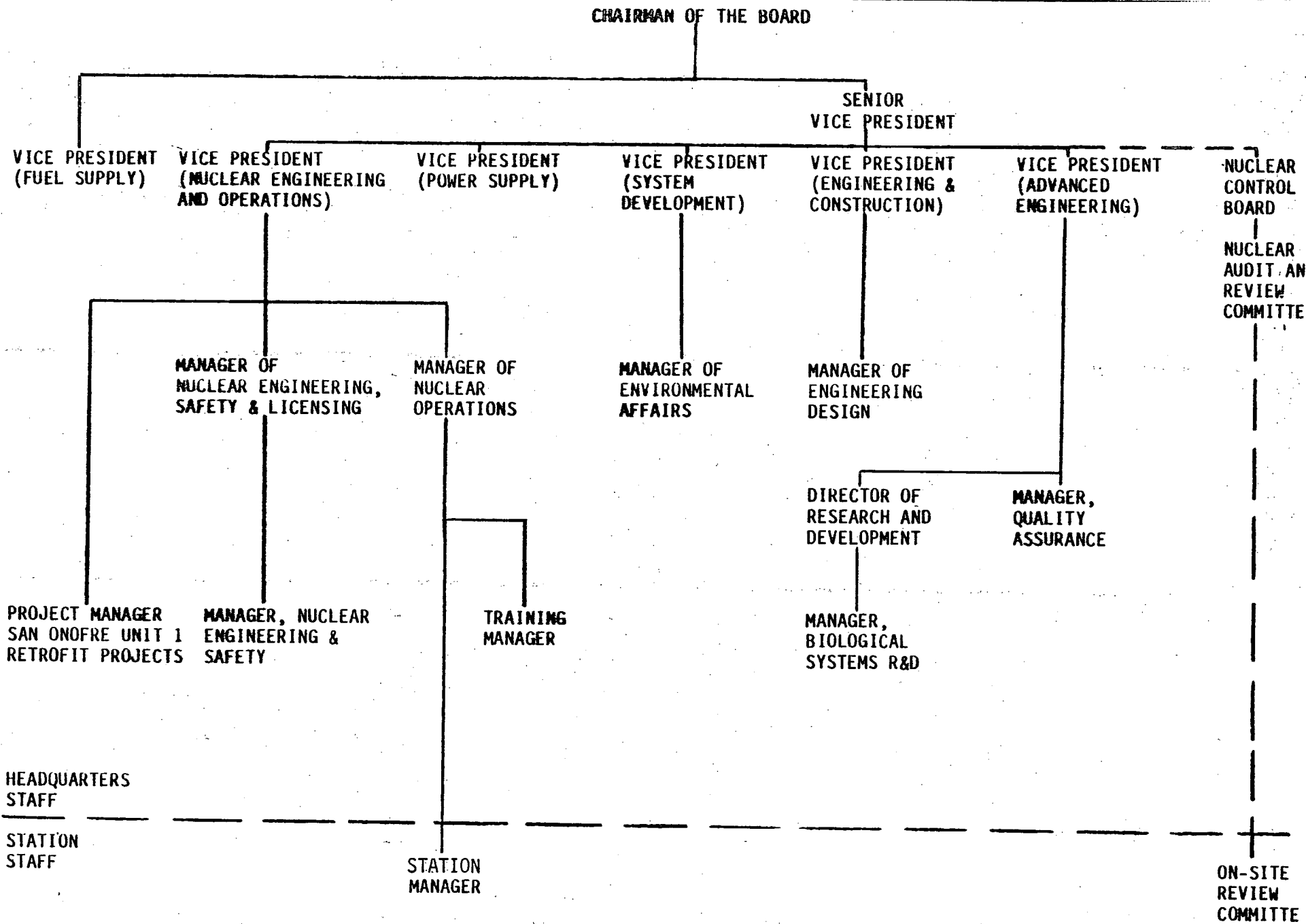


FIGURE 5.2-1 SOUTHERN CALIFORNIA EDISON COMPANY CORPORATE ORGANIZATION

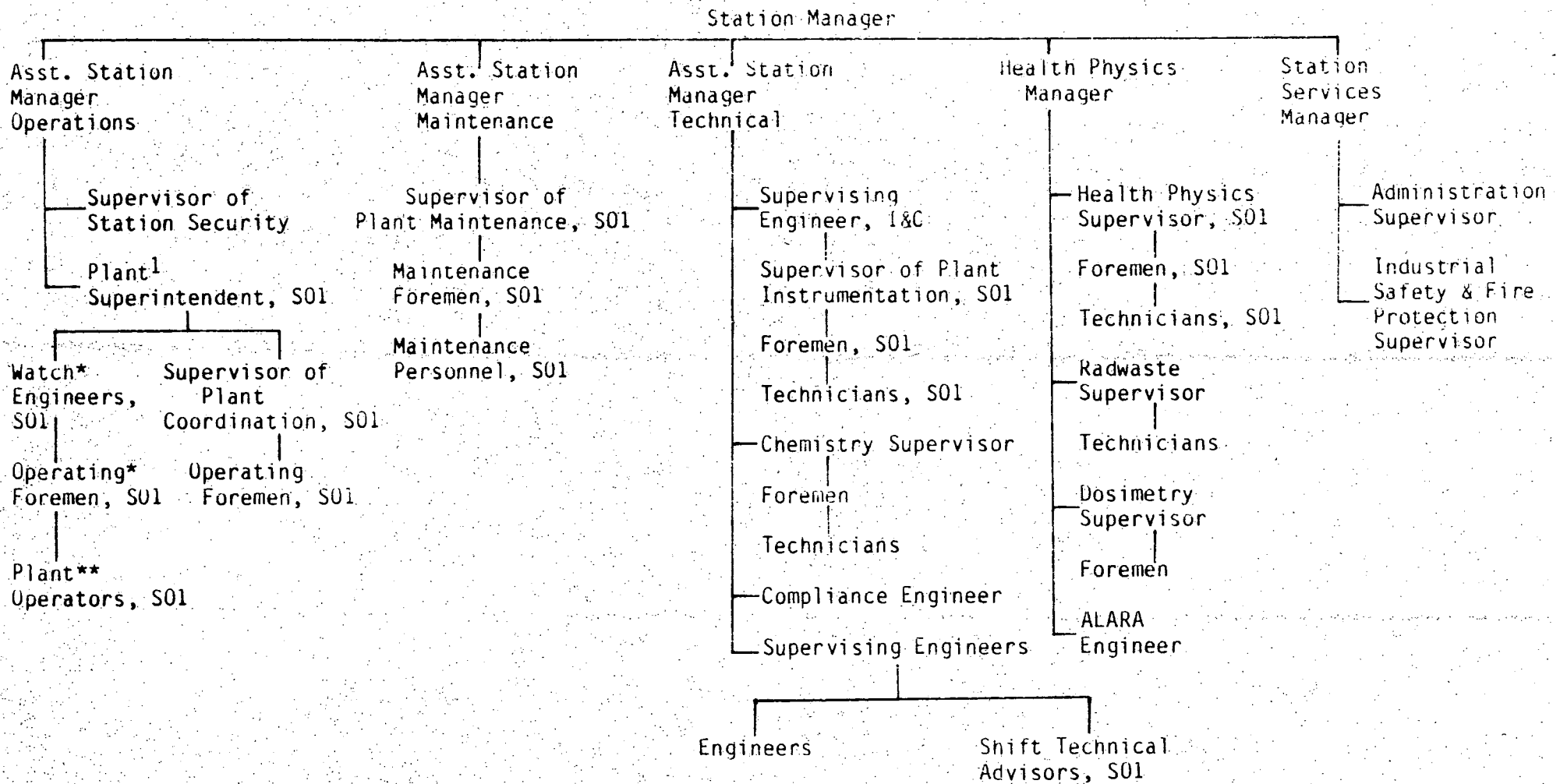


FIGURE 5.2-2 SAN ONOFRE NUCLEAR GENERATING STATION (UNIT 1) ORGANIZATION

- 1 At time of appointment to the position, Senior Reactor Operator License Required
- * Senior Reactor Operator License Required
- ** Control and Assistant Control Operators are holders of Reactor Operator Licenses

b. Responsibilities and Authority

- (1) Determine potential environmental hazards by review and evaluation of facility activities
- (2) Analyze and review Environmental Technical Specification violations. Submit reports of such incidents to the Nuclear Audit and Review Committee.
- (3) Analyze and review reportable environmental occurrences or other facility incidents which relate to the environmental impact of the station and its operation. Submit reports of such incidents to the Nuclear Audit and Review Committee.
- (4) Review and approve facility procedures, tests or experiments and changes thereto which relate to the environmental impact of the station and its operation.
- (5) Review and approve proposed changes to the facility which relate to the environmental impact of the station and its operation.
- (6) When items reviewed under paragraphs (4) and (5) above involve a change in the Environmental Technical Specifications, or a change which would significantly alter the environmental impact of the facility, it shall be submitted to the Nuclear Audit and Review Committee.
- (7) Review proposed changes to the Environmental Technical Specifications and submit to the Nuclear Audit and Review Committee.
- (8) Review station-based environmental surveillance programs for compliance with these environmental technical specifications.

c. Meeting Frequency: Monthly and as Required

- d. Quorum: The Chairman or his designated alternate plus three members, including alternates. For environmental matters, either the Health Physics Manager or the Chemistry Supervisor or the respective designated alternate shall be present."
- e. Alternates: Alternate members shall be appointed in writing by The Onsite Review Committee Chairman to serve on a temporary basis; however, no more than two alternates shall participate in Onsite Review Committee actions at any time.

The membership of the Nuclear Audit and Review Committee and the responsibilities and authorities of the Committee with respect to these environmental technical specifications are as follows:

a. Membership (Chairmen designated by the Nuclear Control Board)

- (1) Manager of Engineering Design
- (2) Manager of Environmental Affairs
- (3) Manager of Nuclear Engineering, Safety & Licensing
- (4) Manager, Nuclear Engineering & Safety
- (5) Manager, Biological Systems Research and Development
- (6) Manager, Quality Assurance
- (7) Manager of Nuclear Operations
- (8) San Diego Gas & Electric Representative"

b. Responsibilities

- (1) Review and investigate reports concerning violations of the Environmental Technical Specifications. Where investigation indicates, evaluate and prepare recommendations to prevent their recurrence.
- (2) Review proposed changes to the Environmental Technical Specifications. Coordinate environmental technical specifications with the safety technical specifications to avoid conflicts and maintain consistency. Submit approved proposed changes to the Environmental Technical Specifications to the Nuclear Control Board for transmittal.
- (3) Review and approve proposed changes to the facility and its procedures or proposed tests or experiments which are forwarded by the Onsite Review Committee when they involve a change in the Environmental Technical Specifications, or a change which would significantly alter the environmental impact of the facility. Submit such proposed changes when required with appropriate environmental analysis to the Nuclear Control Board for transmittal.
- (4) Review all environmental surveillance programs for compliance with these environmental technical specifications.
- (5) Review all design changes related to changes in the Station which relate to the environmental impact of the station and its operation.
- (6) Review overall facility activities as they relate to environmental impact by:
 - (a) formal facility review annually and as directed by the Committee Chairman.