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ACCESSION NBR:8209160162 DUC,DATE: 82/09/14 NOTARIZED: NO DOCKET # FACIL:50-361 San Onofre Nuclear Station, Unit 2, Southern Californ 05000361 AUTH.NAME AUTHOR AFFILIATION BASKIN,K.P. Southern California Edison Co. RECIP.NAME RECIPIENT AFFILIATION MIRAGLIA,F. Licensing Branch 3

SUBJECT: Supple info in 820911 ltr requesting approval of Proposed Change NPF=10=42 to License NPF=10, changing implementation post=accident sampling sys to 830101, Interim post=accident sampling capability info encl.

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K. P. BASKIN MANAGER OF NUCLEAR ENGINEERING, SAFETY, AND LICENSING

September 14, 1982

Director, Office of Nuclear Reactor Regulation Attention: Mr. Frank Miraglia, Branch Chief Licensing Branch No. 3 U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Gentlemen:

8209160162 820914 PDR ADUCK 05000361

Subject: Docket No. 50-361 San Onofre Nuclear Generating Station Unit 2

By letter dated September 11, 1982 Southern California Edison Company (SCE) requested NRC approval of Proposed Change NPF-10-42 to Facility Operating License NPF-10 for San Onofre Nuclear Generating Station, Unit 2. Proposed Change NPF-10-42 changes the implementation date of the Post Accident Sampling System from "prior to exceeding five (5) percent power" to January 1, 1983. The purpose of this letter is to supplement the information provided by the September 11, 1982 letter.

The primary purposes of the PASS are to (1) assess core conditions and (2) assess containment hydrogen level following occurrence of a significant accident. The PASS functions which provide the bulk of the information needed to assess core conditions are the Reactor Coolant System (RCS) liquid indications. Although SCE will not declare the RCS-liquid functions of PASS operable until the remainder of the system has been modified, in the interim SCE will maintain the RCS functions available for service as much as is possible consistent with the need to take those functions out of service to work on the remainder of the system. One criterion of planning the subject modifications will be maximizing availability of the RCS liquid sample function. It is estimated that these functions will be available about seventy-five percent of the time. Indication of containment atmosphere hydrogen levels is provided by the hydrogen monitors which are fully operable.

The secondary functions of PASS (containment atmosphere, containment sump and RCS-gas) are addressed in Enclosure 1. In addition, as modifications of these PASS functions are completed, they will be maintained available as much as possible consistent with the completion of other PASS modifications.



TELEPHONE

(213) 572-1401

Mr. Frank Miraglia

As stated in the September 11, 1982 letter, approval of Proposed Change NPF-10-42 is needed urgently. If you have any questions concerning the enclosed information, please contact me.

Very truly yours,

2P Basken

cc: H. Rood, To be opened by addressee only

ENCLOSURE 1

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INTERIM POST ACCIDENT SAMPLING CAPABILITY

		PASS FUNCTION	STATUS	INTERIM CAPABILITY		
I.	Con	Containment Atmosphere				
	Α.	Hydrogen	Not Available	Hydrogen will be detected and measured by means of the Seismic I, Quality Class II, NUREG-0588 qualified IE powered containment hydrogen monitors. These instruments provide redundant, channelized continuous readout of hydrogen in containment in concentrations from O to 10%. One channel is recorded. Presence of hydrogen in the containment atmosphere will be indicative of core degradation, thus providing a measure of hydrogen production of a degraded core resulting from core voiding and metal-water reaction.		
	Β.	Radionuclide	Not Available	The normal sampling system would be utilized during most of the FSAR Chapter 15 postulated spectrum of accidents because radiation levels would not preclude these samples being processed in the sample lab. In addition the dose rates at the high range in containment area radiation monitors have been correlated to reactor coolant		

correlated to reactor coolant activity levels as shown in FSAR Figures 432.42-1 and 432.42-2. Curves of dose rate versus time are provided for the following conditions:

RCS Average - 100% of core average activity from FSAR Table 11.1-3 released to containment.

RCS Maximum - 100% of core average activity from FSAR Table 11.1-2 released to containment.

	I	PASS FUNCTION	STATUS	INTERIM CAPABILI	ТҮ
				1% Failed Fuel - Assume total core activity is release and of that 1%, noble gases, 50% of the 1% of the other isotope	s that 1% of available for 100% of the halogens and s are released.
				10% Failed Fuel: 10 tin case.	nes the 1%
		· · · · ·		Gap Activity: Utilizes 1.25 assumptions with 1 Xenon and Krypton, 30% 85 and 10% of the Iodin	Reg. Guide 0% of the core of the Krypton es released.
				LOCA: Utilizes Reg. Gu assumptions with 100% of gases, 50% of the haloge the other isotopes relea	ide 1.4 f the noble ens and 1% of ased.
				Extrapolation between the will provide an indicat degraded core condition	nese curves ion of s.
	с.	Diluted samples	Not Availa	e Normal sampling is avai discussed above for Rad IB.	lable as ionuclides in
II.	RCS	Analysis			
	Α.	Ga s			
		1. Hydrogen	Not Availa	e Use Post LOCA Hydrogen r discussion in I.A. above	nonitor. See 2.
		2. Oxygen	Not Availa	e Oxygen measurements are requirement of NUREG-073 useful in analyzing the effects of chlorides on coolant boundary materia	not a 37 but are combined reactor als.
		3. Radionuclide	Not Availa	e Use the high range incor monitors. See the discu	itainment ussion of I.B.

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PASS FUNCTION

STATUS

4. Total Dissolved Not Available Gas

INTERIM CAPABILITY

The Seismic I, Quality Class II, IE, NUREG-0588 qualified subcooled margin monitor and the interim core exit thermocouple system for detection of ICC will provide temperature and pressure parameters necessary to determine the amount of dissolved gas which reactor coolant can retain.

- B. Liquid
 - 1. Boron, pH, Radio- Available nuclide

III. Containment Sump

Not Available

The analysis of containment sump liquids may be correlated with the RCS sample, corrected by the amount of spray and safety injection water. The amount of water injected and the RCS inventory are available from safety grade Refueling Water Storage Tank and Safety Injection Tank level indications. The activity of the RCS coolant is known from the PASS RCS liquid sample.