.

## REGULATORY INFORMATION DISTRIBUTION SYSTEM (RIDS)

ACCESSIO FACIL:5 AUTH.N BASKIN, RECIP. MIRAGLI	N NBR: 8209090 0=361 Sain Onoi AME. AU K'.P. Sow NAME RE A&F:	119 fre Ni THOR I thern CIPIE: Llicens	DOC. Jodean AFFILUI Callif NT AFF Sing B	DATE: Stati ATION ornia ILHATI ranch	82/09/08 5ny Unit Eldison Co DN: 3-	NOTIA 2, Sou Die	RIZED:	ND Cali	forn-	DOCKEIT. #. 05000361,
SUBJECT	: Submits add Proposed Ch	) saife ange (	tv an NPF⊨10	alysis =34,Am	clarific end 11.re	catioņ∉ accid	for: r lent: a	eview malvs	of isvin	
	response to	NRCI	reques	t.,.			•			
DISTRIB TITLE:	UȚION CODEE 3 Llicensina Sub	001S mittal	CDPILE I PSA	S RECE R/FSAR	EVED:LITR Amdts &		d Cor	SIZE	:@	e
NOTES: J	Hanchett: 10v	POR	) DiC U m e	nts. E	D: Chandi	er 1.co				05000361.
N	RR Scaletti 1									
	RECIPIENT		COPIE	S	RECIP	PIENTI		COPI	ES	
	ID CODEVNAM	El	LITTR	ENCL	ID. COU	EINAME	1	LITTR	ENCLI	
	NRR/DL/ADLI		1	0	NRR: LB3	5-8C		1.	0	
	NRR: LIB/3 - LIAI		1.	0	ROOD, H.	• •	01.	1	1	
INTERNAL	FLDZHD82		1	0	TELEALE	<b>T</b> (		1	1	
	IEZDEP. EPOS	3.5.	1	1	TEZDEP	- (EPI Bi	36	7	⊥ Z	
	NRR/DEVAEAB		1	Ō	NRR/DE	CEB	1.1	1	1	
	NRR/DEVEQB	13	3.	3	NRR/DEV	(GB)	28	2	2	
	NRR/DEVHGEB	30	Ž.	ž	NRR/DE	MEB	18	1.	1	
	NRR/DEVMTEB	1.7.	1.	1	NRR/DE	QAB.	21	1	1	
	NRRIDEVSAB	2.4	1	1.	NRR/DEV	SEB	25	<b>1</b> .	1	
	NRR/DHES/HEEL	340	1.	1	NRR/DHF	SILOB	32	1	1	
	NRR/DHFS/DLB	34	1	1	NRR/DHF	SIPTRE	20	1.	1	
	NRR/DSI/AEB	26	1	1	NRR/DSI	/ASB	27	1	1	
	NRR/DSI/CPO	10	1 -	1	NRR/DSI	/CS3	09	1	1	
	NRR/DSI/EITSB	12	1.	1	NRR/DSI	/ICS8	16	1	1	
	NRR/DSI/PSB	1.9	1.	1, .	NRR/DS1	/RAB	55	1.	1	
	NRRADSTARSB	23-	1	1	NRR/DST	VLGB	33	1	1	
$\leq$	REG FILE	0.4	1	1.	RGNS			2	2	
	RM/DDAMI/MIB		1	0						
EXTERNALI:	ACRS	41.	1.0	10	BNLICAMO	TS ONL	<b>Y</b> )	1	1	
	DMB/DSS (AMD)	TS) -	1 -	1	FEMA-RE	PIDIV	39	1	1	
	LPDR	03-	1.	1.	NRCI PDR	<b>}</b> .	02)	1	1	
	NSIC	0.5	1	1	NTIS			1	1	
NOTES:			3	3						

-2.00

TOTAL NUMBER OF COPIES REQUIRED: LITTE 62 ENCLI 56

## Southern California Edison Company

P. O. BOX 800 2244 WALNUT GROVE AVENUE ROSEMEAD. CALIFORNIA 91770

K. P. BASKIN MANAGER OF NUCLEAR ENGINEERING, SAFETY, AND LICENSING

September 8, 1982

Director, Office of Nuclear Reactor Regulation Attention: Mr. Frank Miraglia, Branch Chief Licensing Branch No. 3 U.S. Nuclear Regulatory Commission Washington, D.C. 20555

Gentlemen:

Subject: Docket No. 50-361 San Onofre Nuclear Generating Station Unit 2

By letter dated September 7, 1982 the Nuclear Regulatory Commission (NRC) was requested to expedite its review of Proposed Change NPF-10-34 which was formally submitted as part of Amendment Application No. 11. The purpose of this letter is to provide additional safety analysis clarification in response to a verbal request of NRC Staff.

## Safety Analysis:

8209090119 820908 PDR ADDCK 05000361 PDR ADDCK 05000361 PDR

These changes reflect a change from preliminary data to final data. The calculations have been verified and documented by Combustion Engineering(CE) under their approved QA program.

The FSAR Chapter 15 accident analyses establish the events which take credit for the low reactor coolant flow trip, and these analyses also establish the associated minimum acceptable values for the variables governing the trip setpoints. Because these events are "decreasing flow events", the setpoints sensitivity is to time(that is, value of flow rate at time of trip) rather than initial flow rate. (Note that minimum steady-state flow is specified in Tech Spec Section 3.2.5). The setpoint values shown in the proposed Table 2.2-1, item 11 in psid are the verified and documented values corresponding to the Chapter 15 analyses. The analyses for the sheared shaft accident(FSAR Section 15.3.3.2) remain conservative for the setpoint values shown in the proposed Table 2.2-1.

Suc

TELEPHONE (213) 572-1401

Bool

## Mr. Frank Miraglia

The values presently given in Table 2.2-1 item 11 are based on a very conservative representation of the trip setpoints in percent of a reference value. Since flow testing had not been completed at the time these values were specified, it was not known how conservative they would be. In fact, because the measured  $\Delta p$  input signal has an oscillation of about 2.5 psid peak to peak, the present setpoints result in spurious channel trips. The flow signals from the steam generator  $\Delta p$  instruments were measured as part of the post-core hot functional testing and found acceptable. Oscillations of the steam generator  $\Delta p$  signals as observed on SONGS Unit 2 are typical of those observed for these instrument channels on other CE plants.

Consequently, to insure operability of the reactor coolant flow-low trip function it is necessary to use the actual values of the trip setpoints in psid to provide sufficient margin to preclude spurious reactor trips while maintaining the minimum required values for these setpoints from the accident analysis.

If you have any questions or if I can be of any assistance to you, please contact me.

Very truly yours,

K.P. Baskin / P.Thlun

cc: Mr. H. Rood, Project Manager License Branch No. 3