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June 27, 1990

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U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Subject: Docket No. 50-206
Status of Implementation of Generic Safety Issue Requirements
San Onofre Nuclear Generating Station
Unit 1

This letter provides the status of implementing Generic Safety Issue (GSI) requirements at San Onofre Nuclear Generating Station, Unit 1 (SONGS 1). The letter is in response to Generic Letter 90-04, "Request for Information on the Status of Licensee Implementation of Generic Safety Issues Resolved with Imposition of Requirements or Corrective Actions," dated April 25, 1990.

The GSI implementation status for SONGS 1 is summarized in the enclosure to this letter for all GSI's for which the NRC has issued final technical resolution. The enclosure is formatted to provide the information required in Enclosure 1 to the Generic Letter.

If you have any questions, please contact me.

Very truly yours,

cc: J. B. Martin, Regional Administrator, NRC Region V
C. Caldwell, NRC Senior Resident Inspector, San Onofre Units 1, 2 and 3

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FACILITY NAME: San Onofre Nuclear Generating Station, Unit 1
 DOCKET NO.: 50-206
 LICENSEE: Southern California Edison

STATUS OF SONGS 1 IMPLEMENTATION OF GENERIC SAFETY ISSUES
RESOLVED WITH IMPOSITION OF REQUIREMENTS OR CORRECTIVE ACTIONS
IN RESPONSE TO GENERIC LETTER 90-04

<u>GSI/(MPA No.)</u>	<u>TITLE</u>	<u>APPLICABILITY</u>	<u>STATUS*</u>	<u>COMMENTS</u>
40 (B065)	Safety Concerns Associated With Pipe Breaks In the BWR Scram System	All BWRs	NA	
41 (B058)	BWR Scram Discharge Volume Systems	All BWRs	NA	
43 (B107)	Reliability of Air Systems	All Plants	I 9/30/90 (Estimated)	SCE response to GL 88-14 projected for September 30, 1990.
51 (L913)	Improving the Reliability of Open-Cycle Service Water Systems	All Plants	I 1/15/91 (Estimated)	SCE responded to I.E. Bulletin 81-03 by letters dated June 4, 1981 and August 12, 1982. Follow-on SCE response to GL 89-13 that was issued on January 26, 1990 committed to notify NRC of completing GL Recommended Actions within 30 days of implementation (during Cycle 11).
67.3.3 (A017)	Improved Accident Monitoring	All Plants	I 2/93 (Estimated)	By letter dated May 2, 1990, SCE committed to complete evaluation of separation of cables/power supplies by June 30, 1991. By letter dated October 2, 1989, SCE committed to implement any necessary plant modifications by the end of the Cycle 12 refueling outage.
75 (B076)	Item 1.1 - Post-Trip Review (Program Description and Procedure)	All Plants	C	NRC SER issued on May 9, 1985.
75 (B085)	Item 1.2 - Post-Trip Review - Data and Information Capability	All Plants	I 7/13/90	Additional NRC questions received by letter dated May 16, 1990.
75 (B077)	Item 2.1 - Equipment Classification and Vendor Interface (Reactor Trip System Components)	All Plants	I	SCE response submitted November 28, 1983. By letter dated March 11, 1985, the NRC indicated they had reviewed the SCE response for completeness and adequacy. No outstanding SCE action.
75 (B086)	Item 2.2.1 - Equipment Classification for Safety-Related Components	All Plants	C	NRC SER issued on April 25, 1988.

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 NC - No Changes Necessary
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GSJ/(MPA No.)	TITLE	APPLICABILITY	STATUS*	COMMENTS
75 (L003)	Item 2.2.2 - Vendor Interface for Safety-Related Components	All Plants	I 9/23/90	SCE response to GL 90-03, "Relaxation of Staff Position on Generic Letter 83-28, Item 2.2 Part 2, 'Vendor Interface for Safety-Related Components,'" is scheduled to be issued by September 23, 1990.
75 (B078)	Items 3.1.1 and 3.1.2 - Post-Maintenance Testing (Reactor Trip System Components)	All Plants	C	NRC SER issued on November 4, 1985.
75 (B079)	Item 3.1.3 - Post-Maintenance Testing-Changes to Test Requirements (Reactor Trip System Components)	All Plants	C	NRC SER issued on August 19, 1985.
75 (B087)	Items 3.2.1 and 3.2.2 - Post-Maintenance Testing (All Other Safety-Related Components)	All Plants	I	Additional information provided to NRC by letter dated July 24, 1985. No outstanding SCE actions.
75 (B088)	Item 3.2.3 - Post-Maintenance Testing-Changes to Test Requirements (All Other Safety-Related Components)	All Plants	C	NRC SER issued on August 19, 1985.
75 (B080)	Item 4.1 - Reactor Trip System Reliability (Vendor-Related Modifications)	All Plants	C	NRC SER issued on July 1, 1985.
75 (B081)	Items 4.2.1 and 4.2.2 - Reactor Trip System Reliability-Maintenance and Testing (Preventative Maintenance and Surveillance Program for Reactor Trip Breakers)	All PWRs	C	NRC SER issued on September 16, 1988.
75 (B082)	Item 4.3 - Reactor Trip System Reliability - Design Modifications (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	NRC SER issued on November 28, 1984.
75 (B090)	Item 4.3 - Reactor Trip System Reliability - Tech Spec Changes (Automatic Actuation of Shunt Trip Attachment for Westinghouse and B&W Plants)	All W and B&W Plants	C	NRC SER issued on November 28, 1984.

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75 (B091)	Item 4.4 - Reactor Trip System Reliability (Improvements in Maintenance and Test procedures for B&W Plants)	All B&W Plants	NA	
75 (B092)	Item 4.5.1 - Reactor Trip System Reliability-Diverse Trip Features (System Functional Testing)	All Plants	C	Deferred to evaluation of Item 4.5.2 per NRC letter dated November 4, 1985.
75 (B093)	Items 4.5.2 and 4.5.3 - Reactor Trip System Reliability - Test Alternatives and Intervals (System Functional Testing)	All Plants	C (Item 4.5.2)	Justification for not performing on-line testing of Reactor Trip Breakers provided by SCE letters dated June 3, 1987 and November 13, 1987. NRC concurred that on-line testing is not required in SER issued on September 16, 1988.
			C (Item 4.5.3)	NRC SER issued on June 16, 1989.
86 (B084)	Long Range Plan for Dealing with Stress Corrosion Cracking in BWR Piping	All BWRs	NA	
93 (B098)	Steam Binding of Auxiliary Feedwater Pumps	All PWRs	C	SCE responded to I.E. Bulletin 85-01 by letter dated January 29, 1985 and to GL 88-03 on May 3, 1988. NRC SER issued on May 11, 1988.
99 (L817)	RCS/RHR Suction Line Valve Interlock on PWRs	All PWRs	I 12/2/90 (Estimated)	SCE responded to GL 87-12 by letters dated September 24, 1987; December 23, 1987; and February 12, 1988. SCE responded to GL 88-17 by letters dated January 5, 1989; February 1, 1989; and March 27, 1990. The NRC's responses concerning GL 88-17 are contained in letters dated July 12, 1989, April 26, 1990 and June 5, 1990. Implementation of plant enhancements to be completed by the end of the Cycle 11 refueling outage.
124	Auxiliary Feedwater System Reliability	ANO-1&2, Rancho Seco, Prairie Island 1&2, Crystal River-3, Ft. Calhoun	NA	

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A-13 (B017)	Snubber Operability Assurance - Hydraulic Snubbers	All Plants	I 9/30/90 (Estimated)	NRC SER issued on October 15, 1984. Exemption of large snubbers (capacity >120,000 pounds) from testing needs to be removed from Technical Specifications.
A-13 (B022)	Snubber Operability Assurance - Mechanical Snubbers	All Plants	C	NRC SER issued on October 15, 1984.
A-16 (D012)	Steam Effects on BWR Core Spray Distribution	Oyster Creek and NMP-1	NA	
A-35 (B023)	Adequacy of Offsite Power Systems	All Plants	I 2/93 (Estimated)	By SCE letter dated October 2, 1989, implementation of modifications that address the potential for degraded grid voltage and the adequacy of SONGS 1 voltage distribution are scheduled for the Cycle 12 refueling outage. NRC concurred with this schedule by Order dated January 2, 1990.
B-10	Behavior of BWR Mark III Containments	All BWR Mark III Plants	NA	
B-36	Develop Design, Testing, and Maintenance Criteria for Atmosphere Cleanup System Air Filtration and Adsorption Units for Engineered Safety Features Systems and for Normal Ventilation Systems	All Plants with OL Applications After 4/1/80	NA	
B-63 (B045)	Isolation of Low Pressure Systems Connected to the Reactor Coolant System Pressure Boundary	All Plants	C	Resolved as SEP Topic V-11.A in "Integrated Plant Safety Assessment, Systematic Evaluation Program, San Onofre Nuclear Generating Station, Unit 1," NUREG-0829, December, 1986.

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