

August 30, 1989

Docket No.: 50-206

Mr. Harold B. Ray
Vice President
Southern California Edison Company
Irvine Operations Center
23 Parker
Irvine, California 92718

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Dear Mr. Ray:

SUBJECT: TMI ACTION PLAN ITEM II.K.3.5 (MPA G-01), REACTOR COOLANT PUMP
TRIP ISSUE (TAC NO. 60514)

Re: San Onofre Nuclear Generating Station, Unit No. 1

We have completed our review of TMI Action Plan Item II.K.3.5 regarding reactor coolant pump (RCP) trip requirements for San Onofre Nuclear Generating Station, Unit 1. As discussed in the enclosed safety evaluation, we find that the RCP trip issue is satisfactorily resolved based on the revised Westinghouse Owners Group methodology that was used.

Sincerely,

original signed by Charles Trammell

Charles M. Trammell, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Safety Evaluation, RCP Trip

cc: See next page

[S01 RCP TRIP]

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

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Sincerely,

A handwritten signature in cursive script that reads "Charles M. Trammell".

Charles M. Trammell, Senior Project Manager
Project Directorate V
Division of Reactor Projects - III,
IV, V and Special Projects
Office of Nuclear Reactor Regulation

Enclosure:
Safety Evaluation, RCP Trip

cc: See next page

Mr. Harold B. Ray
Southern California Edison Company

San Onofre Nuclear Generating
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ENCLOSURE

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SONGS-1

For the reactor coolant pump (RCP) trip issue defined in TMI Action Plan Item II.K.3.5 (Multiplant Action G-01), the staff generically approved the Westinghouse Owners Group (WOG) methodology in the Generic Letter 85-12 with the condition that plant-specific information needs to be addressed. For SONGS Unit 1, WOG methodology does not directly apply because of unique plant features such as fuel rods with stainless steel cladding. Therefore, the licensee chose to use a revised WOG methodology to address two safety concerns - (1) Delayed RCP trip during a small break LOCA, and (2) Potential pressurizer power operated relief valve (PORV) failures.

The analysis results showed that (1) Adequate core cooling is provided for SONGS Unit 1 for any SBLOCA with delayed RCP trip, and (2) PORV will be used only as a secondary steam generator tube rupture (SGTR) recovery technique due to a revision to the approved Emergency Operating Procedure (EOP), thereby eliminating the PORV potential failures. We therefore conclude that the licensee has adequately addressed the staff concerns and satisfactorily resolved the RCP trip issue for SONGS Unit 1 based on the acceptable revised WOG methodology.

- References:
- (1) Letter, M.O. Medford, SCE, to J.A. Zwolinski, NRC, TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps," dated October 10, 1985.
 - (2) Letter, M.O. Medford, SCE, to G.E. Lear, NRC, TMI Action Item II.K.3.5, "Automatic Trip of Reactor Coolant Pumps," dated January 21, 1986.