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Docket No. 50-206

Mr. R. Dietch
Vice President
Nuclear Engineering and Operations
Southern California Edison Company
2244 Walnut Grove Avenue
Post Office Box 800
Rosemead, California 91770

MAR 18 1980

Dear Mr. Dietch:

On March 10 and 11, 1980, we visited San Onofre Unit 1 to review the implementation of Category A lessons learned requirements. Based on this review we have determined that the additional information/action identified in the enclosure is necessary. Please provide your written response to each of these items by March 18, 1980.

Sincerely,

Dennis L. Ziemann, Chief
Operating Reactors Branch #2
Division of Operating Reactors

Enclosure:
Request for Additional
Information

cc w/enclosure:
See next page



8004090142

OFFICE	DOR:ORB #2	DOR:ORB #2	LLTF	DOR:ORB #2		
SURNAME	JJShea:ah	ABurger	CNelson	DLZiemann		
DATE	3/11/80	3/13/80	3/13/80	3/13/80		

March 13, 1980

cc w/enclosure:

Charles R. Kocher, Assistant
General Counsel
Southern California Edison Company
Post Office Box 800
Rosemead, California 91770

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Chickering & Gregory
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San Diego Gas & Electric Company
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Resident Inspector
c/o U. S. NRC
P. O. Box 3550
San Onofre, California 92672

Mission Viejo Branch Library
24851 Chrisanta Drive
Mission Viejo, California 92676

Mayor
City of San Clemente
San Clemente, California 92672

Chairman
Board of Supervisors
County of San Diego
San Diego, California 92101

California Department of Health
ATTN: Chief, Environmental
Radiation Control Unit
Radiological Health Section
714 P Street, Room 498
Sacramento, California 95814

Director, Technical Assessment
Division

Office of Radiation Programs
(AW-459)

U. S. Environmental Protection
Agency

Crystal Mall #2
Arlington, Virginia 20460

U. S. Environmental Protection
Agency

Region IX Office

ATTN: EIS COORDINATOR

215 Freemont Street

San Francisco, California 94111

SAN ONOFRE 1

LESSONS LEARNED - CATEGORY A

- 2.1.1
 - Provide statement of qualification for modifications made to backup nitrogen pneumatic supply to PORVs and block valves.
- 2.1.3.a
 - Provide statement of qualification for valve position indication modifications.
- 2.1.3.b
 - Clarify your position regarding the need for additional instrumentation for inadequate core cooling (e.g., level meter).
 - If your position is that additional instrumentation should be installed, provide the schedule for submitting the design of such instrumentation.
 - Provide statement of qualification for subcooling meter and inputs.
 - Propose a method (with schedule) to provide redundant loop temperature and system pressure inputs.
 - Propose a method (with schedule) to provide a range of subcooling margin display covering inputs of 300-700°F and 15 - 2500 psia.
 - Propose a method (with schedule) for automatic selection of the most limiting temperature and pressure inputs.
- 2.1.4
 - Describe the valve control circuit modifications you will make to provide individual valve reopening capability (include schedule).
- 2.1.5
 - Describe those systems installed that can be used for post accident H₂ control.
 - Show how these systems can accommodate a single failure without jeopardizing H₂ control or containment integrity.
 - Provide schedule for implementing procedures to use these systems for H₂ control.
- 2.1.6.a
 - Provide a schedule for implementation of plant modifications which have been found necessary as a result of the systems integrity review (including North Anna event review). If modifications are to be deferred pending completion of SEP, provide detailed justification to support this delay for each modification.

- 2.1.6.b
 - Provide a commitment to implement plant modifications resulting from the shielding and environmental qualification review by January 1, 1981. If modifications are to be deferred pending completion of SEP, provide detailed justification to support this delay for each modification.
- 2.1.7.b
 - Provide, in writing, the accuracy of your AFW flow instruments.
 - Provide schedule for safety grade qualification of the AFW flow instruments.
 - The adequacy of circuit breaker isolation of "control grade" AFW flow instruments from vital bases must be determined.
- 2.1.8.a
 - Incorporate criteria into existing emergency procedures to provide for onsite analysis of reactor coolant and containment air samples. The reactor coolant sample must be analyzed for radioisotopes, dissolved hydrogen and boron concentration. The containment atmosphere should be analyzed for radioisotopes and hydrogen concentration.
 - The dose criteria in GDC 19 should be met, i.e., 5 Rem whole body and equivalent to any part of the body (30 Rem-thyroid, 75 Rem-extremities).
 - Provide schedule for submitting the design details of proposed sampling station and radiological analysis facility. If these modifications are to be deferred pending SEP completion, provide detailed justification to support this position.
- 2.1.8.b
 - Incorporate procedures for using the Technical Associates remote readout monitor for determination of noble gas release rates from the plant stack (as per your January 17, 1980 letter). These procedures should be implemented within 30 days of the date of this letter.
- 2.1.8.c
 - Incorporate into the plant procedures that one of the two existing GeLi systems will be dedicated to analysis of air samples until the new cart mounted sampler is available and appropriate procedures for its use are in effect.
- NRR
 - Provide design details of RCS vents.
 - Commit to install by January 1, 1981 or propose a schedule with justification.

- 2.2.1.b
 - Describe your current program to satisfy the STA operation assessment function of NUREG-0578 (and October 30, 1979 Denton letter).
 - Describe how the STA is kept appraised of the work performed by the group doing the operations assessment function.
- 2.2.2.b
 - Modify your procedures to require that an individual be stationed in the control room (when TSC is activated) to supply data to the TSC. This function should not be assigned to the STA.
 - Provide your long term plan to upgrade the TSC to monitor plant data.