

Southern California Edison Company



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ROBERT DIETCH
VICE PRESIDENT

February 8, 1980

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Mr. H. R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Gentlemen:

Subject: Docket No. 50-206
Amendment No. 88
San Onofre Nuclear Generating Station
Unit 1

Enclosed are three executed and thirty-seven conformed copies of Amendment No. 88 in Docket No. 50-206. Amendment No. 88 consists of Proposed Change Nos. 86, 87, 88 and 90 to the Technical Specifications incorporated into Provisional Operating License No. DPR-13 as Appendices A and B.

Proposed Change No. 86 is a request to delete Technical Specification 3.1.2a(1)A, "Plankton Studies" from the Appendix B Environmental Technical Specifications.

Proposed Change No. 87 is a request to revise Section 4.4.E, "Auxiliary Feedwater System", of the Appendix A Technical Specifications. These revisions are in response to NRC Staff requests based on NRR Bulletins and Orders Task Force review in light of the accident at Three Mile Island, Unit 2. As discussed in our letter dated February 8, 1980 from K. P. Baskin to D. G. Eisenhut, Proposed Change No. 87 cannot be fully implemented (i.e. Sections 4.4.E.2 and 4.4.E.4) until such time as the Auxiliary Feedwater System is automated.

Proposed Change No. 88 is a request to revise Technical Specification 3.5.2, "Control Group Insertion Limits", of the Appendix A Technical Specifications. This revision will ensure that Technical Specification objectives are met with the loading of the reactor core for Cycle 8 operation and remove cycle dependent limits in this specification.

Cycle 8 of the San Onofre reactor core has been reanalyzed as a consequence of changes in the core parameters used for the analysis of Cycle 7, and the results of the reanalysis are presented in the report entitled "Reload Safety Evaluation, San Onofre Generating Station, Unit 1, Cycle 8, January, 1980,"

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Mr. H. R. Denton

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forty (40) copies of which accompany this letter. The reanalysis evaluated fuel performance limits, power capability and accident analysis. The report demonstrates that the reactor can be operated at rated power during Cycle 8 without compromise to the safety or an increase in the risk associated with such operation as presented in the FSA.

Proposed Change No. 90 is a request to (1) revise the definition of Containment Integrity as contained in Section 1.0 of the Appendix A Technical Specifications; (2) add requirements for calibration and testing of Auxiliary Feedwater Flow and Condensate Tank level instrumentation in Table 4.1.1 of Section 4.1 of the Appendix A Technical Specifications, and; (3) revise the requirements for the containment isolation system design in Section 5.2 of the Appendix A Technical Specifications. These revisions are required as a result of (1) implementation of the NRC Short Term Lessons Learned Requirements, and (2) the NRR Bulletins and Orders Task Force review in light of the accident at Three Mile Island, Unit 2.

The San Onofre Unit 1 Cycle 8 refueling outage is scheduled to commence approximately April 12, 1980; the outage is expected to last seven to ten weeks. Accordingly, it is respectfully requested that the NRC review and take appropriate action on the enclosed Proposed Change No. 88 to the Technical Specifications, and the supporting report, "Reload Safety Evaluation, San Onofre Nuclear Generating Station, Unit 1, Cycle 8, January, 1980," prior to resumption of power operation.

Also enclosed is Southern California Edison Company's Check No. T013461 dated February 5, 1980, in the amount of \$12,000.00 as required by 10CFR 170.22.

Very truly yours,

A Arenal for Robert Dietrich

Enclosures