



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

REGULATORY DOCKET FILE COPY

September 22, 1980

Docket No. 50-206

LICENSEE: Southern California Edison Company
FACILITY: San Onofre Unit No. 1
SUBJECT: SEPTEMBER 3, 1980 MEETING TO DISCUSS THE STATUS OF STEAM
GENERATOR REPAIRS AT SAN ONOFRE UNIT NO. 1

NRC and Southern California Edison (SCE) Company representatives met in Bethesda, Maryland, on September 3, 1980, to discuss the status of steam generator repairs at San Onofre Unit No. 1. The meeting attendees are listed in Attachment 1.

The major items of discussion are described below.

1. SCE delivered with cover letter dated August 29, 1980, 20 copies of a report titled "Steam Generator Repair Report, for Southern California Edison, San Onofre Unit 1". SCE requested that any comments or questions resulting from review of the report be transmitted to them as quickly as possible.
2. SCE expects to start decontaminating the steam generator bowl around Saturday September 6, 1980 and sleeving about 10 days later.
3. The final restart report is still scheduled for October 7, 1980 and will include any proposed Technical Specification changes required plus the modified secondary water chemistry program.
4. The sleeve design has changed by using a hard rolled seal on the bottom joint instead of weld or braze. This change is expected to shorten the repair period by 4 to 6 weeks and to reduce radiation exposure to workers. The top joint will remain a brazed joint.
5. Test results conducted so far show that brazing can be successful within the sludge pile region. As a result of determining that brazing can be performed within the sludge pile another sleeve size will be used in the repair program. The sleeve lengths to be used in the repair program are now 36", 30" and 27". By using the 27 inch sleeve in the peripheral region most of the affected tubes can now be sleeved. The number of tubes expected to be plugged has been reduced from 200-300 down to less than 50.

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6. SCE described the sampling method to be used to determine the quality of the sleeving process. Level 1 inspection requires testing 35 tubes in a lot of 100. Level 2 inspection requires testing 15 tubes in a lot of 100. Level 3 inspection requires inspecting 5 tubes in a lot of 100. If one lot passes the level 1 inspection, level 2 is used for the second lot, and if the second lot passes inspection level 3 is used for the third lot. The NRC staff is reviewing the proposed sampling method for acceptability.
7. SCE described a leak test inspection that they propose to do initially 6 months after startup and then each refueling thereafter.
8. SCE briefly described the status of some of the tests conducted thus far. One tube had been brazed in situ and two more will be brazed the week of September 1, 1980. Strength tests conducted to determine if and by how much brazing weakens the tube were completed. Their results show that brazing does not weaken the joint, i.e., tube plus sleeve plus braze retains the same tensile strength as a new tube.
9. Since sleeving is considered to be a long term fix SCE is considering extending pot boiler tests beyond startup. The extent of the continued tests has not yet been determined.
10. The NRC staff discussed the anticipated schedule of review. Around September 10, 1980 the staff expects to issue a letter discussing our preliminary assessment of the repair program, the schedule for continued review, and any open items that need to be addressed before startup. Around October 1, 1980 the staff would issue a letter transmitting an evaluation of the sleeving operation and decontamination. On October 7, 1980 SCE plans to submit a letter request for restart to power with any proposed license changes. Around November 1, 1980 the NRC would transmit our evaluation of the request to return to power including a Safety Evaluation of integrity of steam generators including in-service inspection, ALARA and radiological evaluation, and the modified chemistry program. This transmittal will also include our evaluation of the environmental impact.

Stanley J. Nowicki, Project Manager
 Operating Reactors Branch #5
 Division of Licensing

Attachment:
 List of Attendees

OFFICE	DI: <i>SJM</i>	DL: <i>ORR</i> #5				
SURNAME	SNowicki:cc	DCrutchfield				
DATE	9/1/80	9/22/80				

MEETING SUMMARY DISTRIBUTION

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cc

Charles R. Kocher, Assistant
General Counsel
Southern California Edison Company
Post Office Box 800
Rosemead, California 91770

David R. Pigott
Samuel B. Casey
Chickering & Gregory
Three Embarcadero Center
Twenty-Third Floor
San Francisco, California 94111

Jack E. Thomas
Harry B. Stoehr
San Diego Gas & Electric Company
P. O. Box 1831
San Diego, California 92112

Resident Inspector
c/o U. S. NRC
P. O. Box AA
Oceanside, California 92054

Mission Viejo Branch Library
24851 Chrisanta Drive
Mission Viejo, California 92676

Mayor
City of San Clemente
San Clemente, California 92672

Chairman
Board of Supervisors
County of San Diego
San Diego, California 92101

California Department of Health
ATTN: Chief, Environmental
Radiation Control Unit
Radiological Health Section
714 P Street, Room 498
Sacramento, California 95814

Director, Technical Assessment
Division
Office of Radiation Programs
(AW-459)
U. S. Environmental Protection
Agency
Crystal Mall #2
Arlington, Virginia 20460

U. S. Environmental Protection
Agency
Region IX Office
ATTN: EIS COORDINATOR
215 Freemont Street
San Francisco, California 94111

ATTENDANCE

1. S. J. Nowicki	NRC
2. D. Crutchfield	NRC
3. B. Curtis	SCE
4. P. DeRosa	<u>W</u> NSD
5. C. Hirst	<u>W</u> NTD
6. A. W. Klein	<u>W</u> SOD
7. D. Rooyen	BNL
8. B. Turovlin	NRC
9. R. Krieger	SCE
10. W. Collins	IE/NRC
11. J. Wing	NRC
12. B. Allen	SCE
13. F. Wit	NRC
14. L. Frank	NRC
15. G. Frieling	WEPCo
16. J. Taylor	<u>W</u> NSD
17. J. S. Creswell	AEOD
18. L. F. Miller	NRC
19. E. J. Brown	NRC
20. D. T. Huang	NRC
21. E. L. Murphy	NRC
22. H. F. Conrad	NRC
23. R. Emch	NRC
24. B. D. Liaw	NRC