

Southern California Edison Company



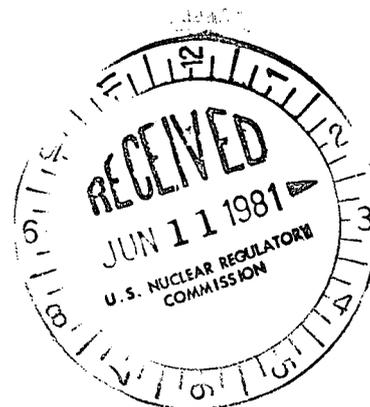
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June 8, 1981

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Director, Office of Nuclear Reactor Regulation
Attention: D. M. Crutchfield, Chief
Operating Reactors Branch No. 5
Division of Licensing
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555



Gentlemen:

Subject: Docket No. 50-206
Clarification of TMI Action Plan Requirements
San Onofre Nuclear Generating Station
Unit 1

Our April 13, 1981 letter provided information on TMI Action Plans including Containment Isolation Dependability (Item II.E.4.2(5)). The discussion regarding the containment isolation setpoint identified a discrepancy between the "Containment Post-Accident Pressure Reanalysis" and the operational limits specified in the Technical Specifications. We indicated that the reanalysis would be evaluated to determine the effect of the operational limits. This letter provides the results of the evaluation.

The "Containment Post-Accident Pressure Reanalysis" for San Onofre Unit 1 was transmitted to the NRC by letter dated January 19, 1977. That analysis indicated the post-accident temperature and pressure to be 2910F and 49.4 psig. The initial conditions for that analysis included an initial containment pressure of 14.7 psia or 0 psig. The Technical Specifications for San Onofre Unit 1 contain a limiting condition for operation for the containment sphere. Specification 3.6.C requires that the reactor shall not be made critical nor be allowed to remain critical if the sphere internal pressure is greater than 0.4 psig or is less than -2.0 psig. This allows operation of Unit 1 with an internal sphere pressure between 0.4 and -2.0 psig. Therefore, in the April 13, 1981 letter, we committed to evaluating the containment post-accident pressure utilizing an initial condition of 0.4 psig.

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The evaluation of the containment post-accident pressure has been performed to determine the effect of utilizing 0.4 psig as an initial condition. This evaluation included an analysis performed in accordance with the Bechtel Topical Report BN-TOP-3, Revision 4, Performance and Sizing of Dry Pressure Containments. The previous analysis utilized Revision 3A of the report. Revision 4 incorporates changes made in response to NRC comments. The results of the analysis provide a peak post-accident temperature and pressure of 2880F and 48.2 psig. Therefore, the previous peak post-accident pressure and temperature as documented in the January 19, 1977 analysis is still applicable and conservative. Therefore, no further action is deemed necessary at this time.

If you have any questions regarding this matter, please let me know.

Very truly yours,

