NRC MONTHLY OPERATING REPORT

DOCKET NO. 50-206
DATE 5-4-79
COMPLETED BY J.P.Rodriguez TELEPHONE (714) 492-7700

OPERATING STATUS

SAN ONOFRE NUCLE	AR GENERATING STATI	ON-UNIT 1	
1. Unit Name: Applied 1 1070	to April 30, 1979		
2. Reporting Period:	347		•
3. Licensed Thermal Power (MWt):	456	-	
4. Nameplate Rating (Gross MWe):	436		
5. Design Electrical Rating (Net MWe):	456		•
6. Maximum Dependable Capacity (Gross MW	126		
7. Maximum Dependable Capacity (Net MWe)),	I D C F	
8. If Changes Occur in Capacity Ratings (Item	is Number 3 Through /) Sinc	e Last Report, Give B	teasons:
N.A.			
·		·	,
9. Power Level To Which Restricted, If Any (Net MWe) N.A.		
10. Reasons For Restrictions, If Any:	i.A.		•
To. Reasons For Restrictions, if Any.			
	······································		
	This Month	Yrto-Date	Cumulative
	•		
11. Hours In Reporting Period	719	2879	78612.29
12. Number Of Hours Reactor Was Critical	637	2797	77512.88
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	637	2797	74044.21
15. Unit Reserve Shutdown Hours	. 0	0	0
16. Gross Thermal Energy Generated (MWH)	848005	3745434.	94754096.
17. Gross Electrical Energy Generated (MWH)	285000	1263600.	32414834.
18. Net Electrical Energy Generated (MWH)	270945.	1204428	30696263。
19. Unit Service Factor	88.6	97.2	74.6
20. Unit Availability Factor	88.6	97.2	74.6
21. Unit Capacity Factor (Using MDC Net)	86.9	96.1	72.0
22. Unit Capacity Factor (Using DER Net)	86.9	96.1	72.0
23. Unit Forced Outage Rate	11.4	2.85	9.45
24. Shutdowns Scheduled Over Next 6 Months None	(Type, Date, and Duration o	f Each):	,
None		·	
25. If Shut Down At End Of Report Period, Es	timated Date of Startun	NA ·	
26. Units In Test Status (Prior to Commercial C	•	Forecast	Achieved
	IVA .		
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERAT	TION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-206		
UNIT	SONGS-1 5-4-79		
COMPLETED BY	J.P.Rodriguez		
TELEPHONE	(714) 492-7700		

APRIL		
AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
431	17	430
430	18	432
427	19	434
426	20	436
369	21	436
0	22	387
0		435
0		436
190		436
426		436
433		433
434		434
434	•	434
433		433
433		
432	31	
	AVERAGE DAILY POWER LEVEL (MWe-Net) 431 430 427 426 369 0 0 190 426 433 434 434 433 433	APRIL AVERAGE DAILY POWER LEVEL (MWe-Net) 431 430 18 427 19 426 20 369 21 0 22 0 23 0 24 190 25 426 433 27 434 28 434 29 433 433 30 433

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. **UNIT NAME** DATE **COMPLETED BY**

50-206 SONGS-1 5-4-79 J.P. Rodriguez

REPORT MONTH

TELEPHONE

No.	Date	Typel	Duration (Hours)	Reason	Method of Shutting Down Reactor3	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
7	4-4	S	1.3	В	4	N.A.	НВ	VALVEX	Turbine stop valve test.
8	4-5	F	82.0	А	1.	79-02	CH	INSTRU HTEXCH	Feedwater flow straightener dislodged from its permanent location causing erroneous feed flow indication. Flow straighteners were fitted into a welded ring restraint and re-enforced the welded end of each straightener assembly. Condenser tube leak.
9	4-22	S	7.17	В	4	N.A.	HF	NA	Heat treat of circulating water system.
	·						HJ	PIPEXX	Steam leak-repaired welded pipe nipple and isolation valve.

F: Forced S: Scheduled

Reason:

A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling

D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

3 Method:

!-Manual

2-Manual Scram.

3-Automatic Scram.

4-Other (Explain)

Exhibit F - Instructions

for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

Exhibit H- Same Source

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

DOCKET NO.	50-206
UNIT	SONGS-1
DATE	5-4-79
COMPLETED	BYJ.P. Rodriguez
TELEPHONE	(714) 492-7700

At the beginning of the month the unit was operating at 100% reactor power and 451 MWe.

On April 4 at 0500, unit load was reduced to 365 MWe to perform turbine stop valve test. The unit was returned to full load, 100% reactor power and 450 MWe, at 0620.

On April 4, continuous surveillance of steam-feedwater flow conditions was instituted due to a shift of the feedwater flow straightener in loop B. Feed flow measurement of loop C had been corrected due to a similar problem.

On April 5 at 1336, a major condenser tube leak was discovered. Unit load was reduced to 360 MWe to investigate and stop the tube leak. At 2247, the unit was taken off line and the reactor placed in hot shutdown condition to repair the condenser tube leak and the feedwater flow straighteners.

On April 9 at 0759, after completion of repairs, the unit was returned to power and full load was achieved at 1800 hrs.

On April 13, continuous manning at the auxiliary feedwater header was instituted in compliance with NRC directive.

On April 22 at 0800, unit load was reduced to 370 MWe to heat treat the circulating water intake tunnel. Load was further reduced to 150 MWe at 1117 for repairs of a steam leak in a turbine steam extraction line. Unit load was returned to 450 MWe at 1510.

On April 26, safety injection initiation logic was changed , from 2 of 3 low pressurizer pressure and 2 of 3 low pressurizer level to 2 of 3 low pressurizer pressure only, at the request of the NRC.

At the end of the month the unit was operating at 100% reactor power and 453~MWe.

DOCKET NO. 50-206

		UNIT	SONG	S 1			
		DATE5/4/1979					
		COMPL	ETED BY	J.P.R	odriguez		
		TELEP	HONE	(714)	492-7700		
1.	Scheduled date for next refueling shutdown.	÷					
	March 21, 1980						
2.	Scheduled date for restart following refueling. May 5, 1980						
3	Will refueling or resumption of operation there Technical Specification change or other license			a			
	No changes expected at this date.						
	What will these be?						
	N.A.						
					•		
4.	Scheduled date for submitting proposed licensing information.	ng act	ion and	support	ing		
	N.A.						
5.	Important licensing considerations associated we new or different fuel design or supplier, unrew performance analysis methods, significant change operating procedures. None	iewed	design	or	ew		
6 -	The number of fuel acception		t				
6.	The number of fuel assemblies.						
	a) In the core157						
	b) In the spent fuel storage pool. 58						
7.	Licensed spent fuel storage capacity. 216		·				
	Intended change in spent fuel storage capacity.	Nor	ne .				
8.	Projected date of last refueling that can be difuel storage pool assuming present capacity. January 11, 1983	scharg	ed to sp	pent			