Southern California Edison Company

P. O. BOX 800 2244 WALNUT GROVE AVENUE ROSEMEAD. CALIFORNIA 91770

M.O. MEDFORD MANAGER, NUCLEAR LICENSING

September 4, 1984

Director, Office of Nuclear Reactor Regulation Attention: D. G. Eisenhut, Director Division of Licensing U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Gentlemen:

Subject: Docket Nos. 50-206, 50-361 and 50-362 Revisions to Proposed Changes San Onofre Nuclear Generating Station Units 1, 2 and 3

By letters dated July 9, 1984 and May 1, 1984 we provided Proposed Change Nos. 125 and 130 to the San Onofre Unit 1 Technical Specifications. Our letter of March 7, 1984 provided Proposed Changes NPF-10-83 and NPF-15-83 to the San Onofre Units 2 and 3 Technical Specifications. Recent changes to the offsite organization and a review of procedures necessary to implement a proposed San Onofre Unit 1 Technical Specification revision, have resulted in the need to further modify these proposed changes currently under your review.

A revision to Proposed Change No. 130 to the San Onofre Unit 1 Technical Specifications and Proposed Changes NPF-10-83 and NPF-15-83 to the San Onofre Units 2 and 3 Technical Specifications is required to reflect changes to the offsite portion of the organization responsible for San Onofre Nuclear Generating Station. The offsite organization in the proposed changes, shown in Enclosure 1, should be revised as shown in Enclosure 2. The Senior Vice President now has the Vice President and Site Manager, Nuclear Generation Site and the Vice President, Nuclear Engineering, Safety and Licensing also reporting to him. Proposed Change No. 130 to the San Onofre Unit 1 Technical Specifications should also be revised to show the San Onofre Unit 1 Project Manager now reporting to the Manager, Nuclear Generation Services in the Nuclear Generation Site Department.

A revision to Proposed Change No. 125 to the San Onofre Unit 1 Technical Specifications is required to provide an appropriate radiation instrumentation setpoint. The high range containment radiation monitors (R-1255, R-1257), as indicated in proposed Table 3.5.10-1 (Enclosure 3) have a proposed alarm setpoint of 1 R/hr. The alarm setpoint should be revised to be 10 R/hr (Enclosure 4). This revision is required because 1 R/hr is the lowest end of the measurement capability of these monitors and the alarm setpoint cannot be reset after an alarm. The revision to 10 R/hr is acceptable because the purpose of these monitors is to monitor potential post-accident containment radiation which may exceed the range of the normal containment radiation monitors. $\frac{8409060240}{840904}$

PDR ADOCK OSOOOZO6

PDR

No Check Rec'd

TELEPHONE (213) 572-1749 Mr. D. G. Eisenhut

The revisions requested in this letter are administrative in nature and are made to update proposed changes currently under your review. Therefore, a special Amendment Application for the requested revisions is not required.

If you have any questions regarding the above discussed information, please let me know.

Very truly yours,

M.O. Medford

Enclosures

- cc: J. B. Martin, Administrator, NRC Region V
 - E. McKenna, NRR Project Manager
 - H. Rood, NRC Project Manager
 - A. E. Chaffee, USNRC Resident Inspector

ENCLOSURE 1