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Docket No. 50-206

Mr. R. Dietch, Vice President
Nuclear Engineering & Operations
Southern California Edison Company
2244 Walnut Grove Avenue
P. O. Box 800
Rosemead, California 91770

Dear Mr. Dietch:

During our meeting of February 24, 1982, with the Westinghouse Owners Group and the three Westinghouse NSSS Owners, who received our August 21, 1981 letter concerning pressurized thermal shock (PTS), it was requested that we provide a formal request for any additional information which would be desired regarding the PTS issue. The enclosure identifies the requested additional information. We request the information be submitted by April 30, 1982.

The reporting and/or recordkeeping requirements of this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

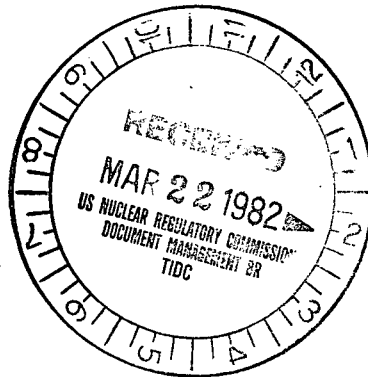
Sincerely,

ORIGINAL SIGNED BY

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosure:
Request for Additional
Information

cc w/enclosure:
See next page



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PDR

*See previous white for concurrences

OFFICE	ORB#4:DL	ORB#5:DL	C-ORB#4:DL	C-ORB#5:DL	AD:OR:DL	D:DST
SURNAME	GVising;cf	WPaulson*	JStolz*	*DCrutchfield	Novak	SHanauer*
DATE	3/11/82	3/5/82	3/5/82	-3/5/82	3/15/82	3/10/82

Docket No. 50-206

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Nuclear Engineering & Operations
Southern California Edison Company
2244 Walnut Grove Avenue
P. D. Box 800
Rosemead, California 91770

Dear Mr. Dietch:

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The reporting and/or recordkeeping requirements of this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

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SURNAME	GVissing;cf	WPaulson*	JStolz*	*DCrutchfield	TNovak	SHanauer
DATE	3/ /82	3/5/82	3/5/82	3/5/82	3/ /82	3/10/82

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Mr. R. Dietch, Vice President
Nuclear Engineering & Operations
Southern California Edison Company
2244 Walnut Grove Avenue
P. O. Box 800
Rosemead, California 91770

Dear Mr. Dietch:

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Sincerely,

Dennis M. Crutchfield, Chief
Operating Reactors Branch #5
Division of Licensing

Enclosure:
Request for Additional
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cc w/enclosure:
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OFFICE	ORB#4:DL GVissing/cb	ORB#5:DL WPaulson	C-ORB#4:DL JStutz	C-ORB#5:DL DCrutchfield	AD-OR:DL TNovak	D:DST SHanauer	
SURNAME							
DATE	3/5/82	3/5/82	3/5/82	3/5/82	3/ /82	3/ /82	

Mr. R. Dietch

cc

Charles R. Kocher, Assistant
General Counsel
James Beoletto, Esquire
Southern California Edison Company
Post Office Box 800
Rosemead, California 91770

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Orrick, Herrington & Sutcliffe
6600 Montgomery Street
San Francisco, California 94111

Harry B. Stoehr
San Diego Gas & Electric Company
P. O. Box 1831
San Diego, California 92112

Resident Inspector/San Onofre NPS
c/o U. S. NRC
P. O. Box 4329
San Clemente, California 92672

Mission Viejo Branch Library
24851 Chrisanta Drive
Mission Viejo, California 92676

Mayor
City of San Clemente
SSan Clemente, California 92672

Chairman
Board of Supervisors
County of San Diego
San Diego, California 92101

California Department of Health
ATTN: Chief, Environmental
Radiation Control Unit
Radiological Health Section
714 P Street, Room 498
Sacramento, California 95814

U. S. Environmental Protection Agency
Region IX Office
ATTN: Regional Radiation Representative
215 Fremont Street
San Francisco, California 94111

Robert H. Engelken, Regional Administrator
Nuclear Regulatory Commission, Region V
Office of Inspection and Enforcement
1450 Maria Lane
Walnut Creek, California 94596

REQUEST FOR ADDITIONAL INFORMATION
CONCERNING
PRESSURIZED THERMAL SHOCK

AND

REGARDING THE "150 DAY" RESPONSE TO NRC LETTER DATED AUGUST 21, 1981

FOR

SAN ONOFRE 1

DOCKET NO. 50-206

1. Provide the following information related to fluence determination:

- (A) Plant specific information which would allow determination of the pressure vessel fluence. Such information should contain as built core and pressure vessel dimensions, regional material composition and neutron source for a two-dimensional (R-0) and (R-Z) neutron transport solution, and
- (B) Plant specific values of the pressure vessel fluence and its estimated uncertainty.

2. Concerning Operator Action

In your evaluation, the actions described do not provide the operator with clear direction for dealing with conflicting concerns that need to be evaluated when considering the operation of HPI and the charging flow as it relates to vessel integrity and maintaining core cooling. Provide an evaluation of the need and effectiveness of procedure modifications to clearly identify the concerns in the emergency operating procedures themselves. This should be done in contrast of depending upon upgrading operator training alone.

3. Concerning Input Data and Assumptions

3.0 Provide a description of the models or data used for:

- (a) Heat sources (or sinks),
- (b) Decay heat,
- (c) ECC and feedwater temperatures (enthalpies) and flow rates,
- (d) Primary and secondary relief capacities,
- (e) Empirical correlation coefficients used for PTS evaluations,
- (f) Operator Actions,
- (g) Initial conditions.

- 3.2 Provide a list of all transients or accidents by class (for example: excessive feedwater, operating transients which result from multiple failures including control system failures and/or operator error, steam line break and small break LOCA) which could lead to inside vessel fluid temperatures of 300 F or lower. Provide any Failure Modes and Effects Analyses (FMEAs) of control systems currently available or reference any such analyses already submitted. Estimate the frequency of occurrence of these events and provide the basis for the estimates. Discuss the assumptions made regarding reactor operator actions.

For a given initiating event, potential multiple and consequences failures need to be considered to identify those transients which could lead to a PTS problem.

- 3.3 Identify all potential PTS events which have occurred at your facility. Include a designation of the operator actions and identify potential additional failures (including operator) which could have resulted in a more severe event.

4. Concerning a Review of Operating History.

Review your operating history at your plant and identify events which have resulted in exceeding the cooldown rate of 100°F/hr. as well as those events which could have exceeded the cooldown rate limit if not mitigated by plant controls or operator actions.