Docket No. 50-206 LS05-82- 02-056

> Mr. R. Dietch, Vice President Nuclear Engineering and Operations Southern California Edison Company 2244 Walnut Grove Avenue Post Office Box 800 Rosemead, California 91770

Dear Mr. Dietch:

SUBJECT: SAN ONOFRE 1 - SEP TOPIC XV-7, LOCKED REACTOR COOLANT

PUMP ANALYSIS - REQUEST FOR ADDITIONAL INFORMATION

We have reviewed the topic assessment on the above topic that you submitted on July 1, 1981, and we find that additional information is needed to complete our safety evaluation.

Please provide the requested information within 30 days of receipt of this letter.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,

Dennis M. Crutchfield, Chief Operating Reactors Branch No. 5 Division of Licensing

DS4 45E (08)

ADDI G. Stoley

E. Mc KENNA

cc w/enclosure: See next page

Information

Request for Additional

Enclosure:

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Cc Charles R. Kocher, Assistant General Counsel James Beoletto, Esquire Southern California Edison Company Post Office Box 800 Rosemead, California 91770

David R. Pigott Orrick, Herrington & Sutcliffe 600 Montgomery Street San Francisco, California 94111

Harry B. Stoehr
San Diego Gas & Electric Company
P. O. Box 1831
San Diego, California 92112

Resident Inspector/San Onofre NPS c/o U. S. NRC P. O. Box 4329 San Clemente, California 92672

Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

Mayor City of San Clemente San Clemente, California 92672

Chairman
Board of Supervisors
County of San Diego
San Diego, California 92101

California Department of Health ATTN: Chief, Environmental Radiation Control Unit Radiological Health Section 714 P Street, Room 498 Sacramento, California 95814

U. S. Environmental Protection Agency Region IX Office ATTN: Regional Radiation Representative 215 Freemont Street San Francisco, California 94111

## **ENCLOSURE**

## SAN ONOFRE SEP TOPIC XV-7

## REQUEST FOR ADDITIONAL INFORMATION

An analysis comparing the RCP rotor seizure accident with the RCP shaft break is requested to identify the limiting case with respect to maximum RCS pressure and limiting fuel performance. The analyses of these events presented in your topic assessment which utilized a comparison to the Surry Nuclear Plant's analyses are not sufficient to determine if the DNBR limits are met. Your analysis should assume a loss of offsite power as a result of turbine trip, the Technical Specification limit of steam generator tube leakage and a single active failure in safety systems. Appropriate delay time for loss of offsite power may be assumed if suitably justified. The evaluation should include tabulations of the sequence of events, disposition of normally operating systems, utilization of safety systems and transient curves including RCS pressure and DNBR and reactor coolant flow for the limiting accident. Confirm that the results of the analysis meet the acceptance criteria for these events per SRP 15.3.3. Assume any fuel rod which has a DNBR less than the minimum allowable value for San Onofre fails. State the amount of failed fuel in the results of the analysis.