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SUBJECT: Forwards Proposed Change NPF-10-34 to License NPF-10 previously submitted as part of 820903 Amend Application 11.Change alters Tech Spec Table 2.2-1 & Pages 2-3 & 2-4, deleting ref to full differential pressure.

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September 7, 1982

Director, Office of Nuclear Reactor Regulation
Attention: Mr. Frank Miraglia, Branch Chief
Licensing Branch No. 3
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Gentlemen:

Subject: Docket No. 50-361
San Onofre Nuclear Generating Station
Unit 2

Amendment Application No. 11 to Facility Operating License NPF-10 for San Onofre Nuclear Generating Station, Unit 2, was formally submitted to the NRC on September 3, 1982. Amendment Application No. 11 consisted of eight (8) proposed changes to Technical Specifications incorporated in Facility Operating License NPF-10 as Appendix A.

Enclosed you will find proposed change NPF-10-34, which was formally submitted as part of Amendment Application No. 11. NPF-10-34 is a change to Technical Specification 2.2.1, Table 2.2-1. It is requested that you please consider this change to be of emergency status and direct your attention to it as soon as possible. Proposed change to Technical Specification 2.2.1, Table 2.2-1, pages 2-3 and 2-4, deletes reference to percent of full differential pressure and inserts the actual instrument trip setpoints in psid.

If you have any questions or if I can be of any assistance to you concerning the enclosed information, please contact me.

Very truly yours,

K P Baskin

Enclosure

cc: Mr. H. Rood, Project Manager
License Branch No. 3

B001

DESCRIPTION OF PROPOSED CHANGE NPF-10-34 AND SAFETY ANALYSIS AMENDMENT APPLICATION NO. 11 TO OPERATING LICENSE NPF-10

This is a request to revise Table 2.2-1 of Specification 2.2.1.

This Proposed Change, (1) deletes reference to percent of full differential pressure and inserts the actual instrument calibration values, and (2) reflects a recalculation of the instrument setpoints.

Existing Specification:

Page	Functional Unit	Trip Setpoint	Allowable Value
2-3	11. Reactor Coolant Flow-Low		
	a) DN Rate	$\leq$ 0.3%/sec (6)(8)	$\leq$ 0.315%/sec (6)(8)
	b) Floor	$\geq$ 60% (6)(8)	$\geq$ 55% (6)(8)
	c) Step	$\leq$ 10% (6)(8)	$\leq$ 13% (6)(8)

Page 2-4

(6) DN RATE, % of reference value, is the maximum decrease rate of the trip setpoint.

FLOOR, % of reference value, is the minimum value of the trip setpoint.

STEP, % of reference value, is the amount by which the trip setpoint is below the input signal unless limited by DN Rate or Floor. The reference value is that of the input signal at operating flow and coolant temperature.

Proposed Specifications:

11. Reactor Coolant Flow-Low

Functional Unit	Trip Setpoint	Allowable Value
a. DN Rate	$\leq$ 0.22 psid/sec (6)(8)	$\leq$ 0.231 psid/sec (6)(8)
b. Floor	$\geq$ 13.2 psid (6)(8)	$\geq$ 12.1 psid (6)(8)
c. Step	$\leq$ 6.82 psid (6)(8)	$\leq$ 7.231 psid (6)(8)

PDR Dupe #
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To be consistent with this change, Table Notation (6) for Table 2.2.1 and is to read as follows:

(6) DN RATE, is the maximum decrease rate of the trip setpoint.

FLOOR, is the minimum value of the trip setpoint.

STEP, is the amount by which the trip setpoint is below the input signal unless limited by DN Rate or Floor.

Reason for Proposed Change:

This proposed change:

(1) Allows easier setpoint verification as the instruments are set to the values actually shown in the Technical Specifications.

(2) Modifies the setpoint values as the result of calculations performed during the startup program.

Safety Analysis:

These changes reflect a change from preliminary data to final data. The calculations have been verified in accordance with the procedures of the qualified and approved Q.A. Program.

Accordingly, it is concluded that: (1) Proposed Change NPF-10-34 does not present significant hazard considerations not described or implicit in the Final Safety Analysis; (2) there is reasonable assurance that the health and safety of the public will not be endangered by the proposed change; and (3) this action will not result in a condition which significantly alters the impact of the station on the environment as described in the NRC Final Environmental Statement.

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