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Document Control Branch (Document Control Desk)						
SUBJECT: Provides correction to 891207 response to NRC Bulletin 89-002 re Anchor-Darling swing check valves.						
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F. R. NANDY MANAGER OF NUCLEAR LICENSING

April 19, 1990

TELEPHONE (714) 587-5400

U. S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, D. C. 20555

Gentlemen:

Subject: Docket Nos. 50-361 and 50-362 NRC Bulletin 89-02, "Anchor-Darling Swing Check Valves" San Onofre Nuclear Generating Station Units 2 and 3

Reference: F. R. Nandy (SCE) to Document Control Desk (NRC) letter dated December 7, 1989; Same subject

This provides a correction to Southern California Edison's (SCE) response to NRC Bulletin 89-02, "Stress Corrosion Cracking of High-Hardness Type 410 Stainless Steel Internal Preloaded Bolting in Anchor-Darling Model S350W Swing Check Valves or Valves of Similar Design" for San Onofre Units 2 and 3. During an internal review of SCE's original response (referenced above), it was determined that both the scope of this review and the conclusion drawn from it were not as comprehensive as originally reported.

SCE's original response implied that all safety-related check valves had been evaluated. In reality, only check valves included in the Inservice Testing (IST) program were evaluated. The IST program includes only the following check valves:

1. All active safety-related check valves.

These are valves for which the valve disc must open or close to perform its safety function.

2. All seat-leakage-tested passive safety-related check valves.

These are valves for which the valve disc is normally closed. The disc must seat in the closed position and meet specified maximum leakage criteria to perform its safety function.

Therefore, other passive safety-related check valves were not included in the original review.

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Document Control Desk

Because SCE's original response did not address the complete scope of the NRC Bulletin 89-02 requirements, SCE will prepare a supplemental report documenting evaluation of those valves that have not been addressed. This supplemental report will be forwarded to the NRC by August 1, 1990. Additionally, SCE Engineering will perform an independent verification of our response (original plus supplemental response) to ensure that all of the valves subject to the requirements of Bulletin 89-02 have been properly evaluated.

If you have any questions, please contact me.

Very truly yours,

cc: J. B. Martin, Regional Administrator, NRC Region V C. Caldwell, NRC Senior Resident Inspector, San Onofre Units 1, 2 and 3