

**Submittal of “Summary of Small Break Loss of Coolant Accident Analysis Changes and Results for AP1000 Plant” (Non-Proprietary)**

**September 2013**

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# Summary of Small Break Loss of Coolant Accident Analysis Changes and Results for AP1000 Plant

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## Executive Summary

- Core Reference Report (CRR) analysis revised to address Automatic Depressurization System (ADS) stage 4 (ADS-4) single failure assumption
- Minor core uncover and cladding heat-up observed
- Significant margin to 10 CFR 50.46 limits exist

## Overview

- ADS-4 failure location discussion/background
- Summarize Small Break LOCA (SBLOCA) changes from the Core Reference Report (CRR), Rev. 0
- Present summary of revised SBLOCA results contained in CRR Rev. 1
- Thermal Conductivity Degradation (TCD) assessment



## Background

- Analyzed SBLOCA single failure for **AP1000**<sup>®</sup> plant was a failure of an ADS-4 valve on the PRHR side.
  - Testing at the Oregon State University (OSU) APEX1000 test facility indicated that a single failure of an ADS-4 valve on the non-Passive Residual Heat Removal (PRHR) heat exchanger side could be more limiting.
- Analysis work has been completed to incorporate ADS-4 valve failure on non-PRHR side in SBLOCA analysis in CRR Rev. 1.
  - Consistent with the staff recommendation in Chapter 21 of **AP1000** plant Final Safety Evaluation Report (FSER) (NUREG-1793)



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## Background

- Major changes to SBLOCA analysis described in Revision 0 of the Core Reference Report (CRR; WCAP-17524-P)
  - Single failure assumption
    - ADS-4 on non-PRHR side
  - Enhanced containment backpressure for select breaks
    - Containment backpressure used currently for Double-ended Direct Vessel Injection (DEDVI) line break
    - Transient pressure history now being utilized for 2 inch Cold Leg Break and Inadvertent Automatic Depressurization System (INADS) simulations
  - Use of SBLOCTA (To be discussed later)
    - Minor core uncover predicted by NOTRUMP code



## ADS-4 Failure Location Effect

- Failure of ADS-4 valve on non-PRHR side results in liquid hold-up in hot leg
  - Liquid draindown from Pressurizer is entrained out the PRHR (i.e., Pressurizer) side ADS-4 discharge path
    - Does not return to vessel as easily
    - Results in loss of vessel inventory during the injection gap period
- Failure of ADS-4 valve on PRHR side
  - Liquid draindown from Pressurizer is drawn through the upper plenum before it can be discharged out ADS-4 path
    - Provides vessel makeup during injection gap period

# NOTRUMP AP1000 Plant Noding Diagram

a,c





## Containment Pressure Response Generation

- Mass and energy (MNE) releases from atmospheric containment pressure SBLOCA analysis generated with NOTRUMP
  - 2 inch Cold Leg Break
  - Inadvertent ADS
- MNE releases utilized in WGOTHIC minimum containment pressure model to generate containment pressure response for associated SBLOCA cases
- Additional iteration performed with 2 inch Cold Leg Break utilizing variable containment MNE releases to demonstrate effect





# AP1000 Plant WGOTHIC Minimum Pressure EM Background

- Used to calculate the minimum containment pressure for the DEDVI, SBLOCA break analysis discussed in Section 15.6.5.4B.3.1 of Revision 19 of the DCD
- Assumptions defined in Section 13.8 of WCAP-15846, Revision 1, “WGOTHIC Containment Minimum Pressure Calculation for Small-Break LOCA and Long-Term Cooling”
- Section 13.8 of WCAP-15846, Revision 1 states:

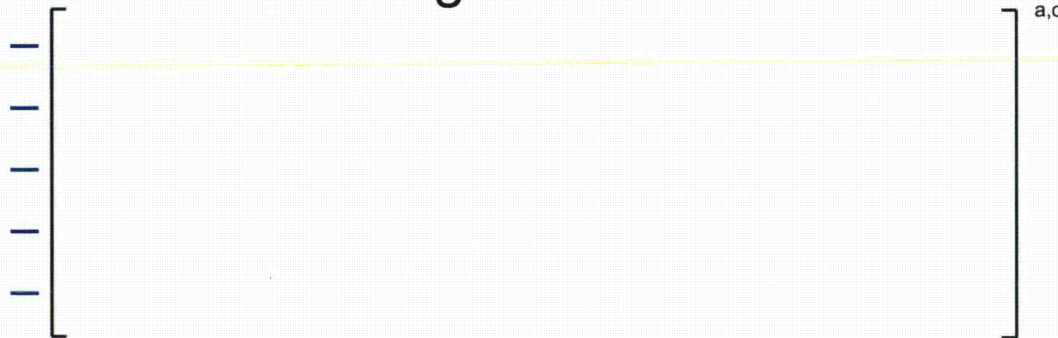
*“A conservative calculation of the containment pressure is needed to provide the containment boundary conditions for the **AP1000** small-break LOCA analysis and the long-term cooling analysis....For this reason, the WGOTHIC containment pressure calculation is biased to obtain the minimum containment pressure for a given event.”*





# AP1000 Plant WGOTHIC Minimum Pressure EM Modeling and Analysis for 2 inch and INADS

- Started with WGOTHIC **AP1000** plant DCD Revision 19 Double Ended Cold Leg Break LOCA peak containment pressure model
- Modeling/analysis performed in accordance with approved methodology and assumptions
  - Table 13-136 (Section 13.8) of WCAP-15846, Revision 1
- The following are some of the parameters conservatively biased in the WGOTHIC EM to calculate a minimum containment pressure according to Table 13-136:





# AP1000 Plant WGOTHIC Minimum Pressure EM Modeling Assumption for 2 inch and INADS

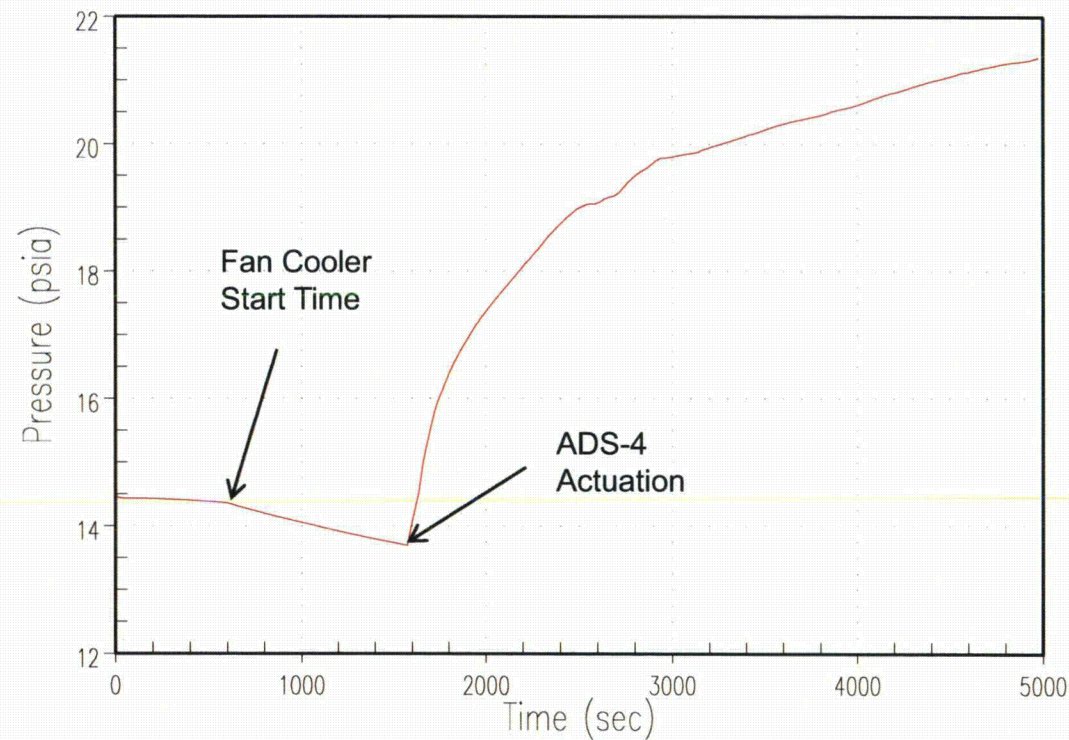
- Containment Purge Valve for Minimum Pressure Calculation
  - [ ]<sup>a,c</sup>
    - FSER Page 15-47: Isolated on a high containment pressure signal
- 2 inch and INADS SBLOCA will not pressurize the containment atmosphere high enough to isolate the purge system on a high containment pressure signal
- Sections 7.3.1.1 and 7.3.1.2.1 of the DCD states automatic containment isolation occurs on a safeguard actuation (S) signal which include: low pressurizer pressure, high-2 containment pressure, low cold leg temperature, and low lead-lag compensated steam line pressure
- Therefore, purge system is isolated on [ ]<sup>a,c</sup> (S) signal consistent with:
  - [ ]<sup>a,c</sup>
    - **AP1000** plant safeguard actuation signals/logic





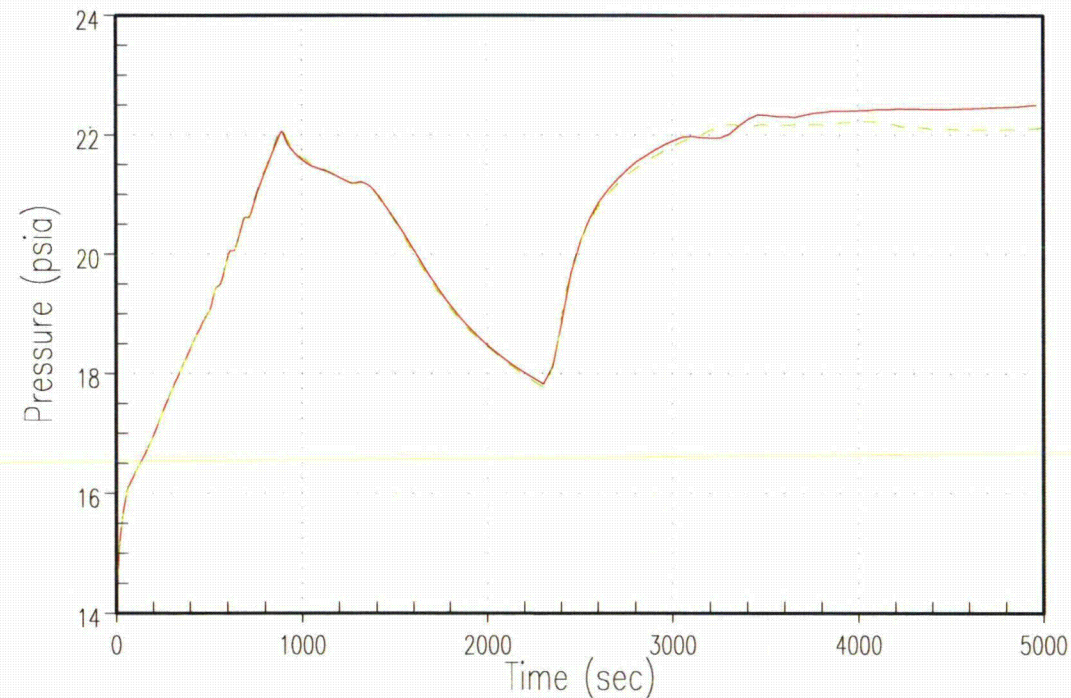
# Containment Pressure Response (Inadvertent ADS)

Inadvertent ADS Containment Pressure Response



# Containment Pressure Response (2 inch Cold Leg Break)

2-Inch Cold Leg Break Containment Pressure Response





## SBLOCTA Background

- SBLOCTA is the fuel rod heat-up code used for small break LOCA analyses completed with the NOTRUMP-EM
- Modified version of the LOCTA-IV Code
  - LOCTA-IV: WCAP-8301
  - Modified for use in the NOTRUMP-EM: WCAP-10054-P-A
- Currently utilized to support **AP1000** plant Probabilistic Risk Assessment (PRA) SBLOCA efforts

## SBLOCTA Description

- Calculates the fuel rod cladding temperature and oxidation transients for the hot rod and hot assembly average rod
  - 1-D radial heat conduction of a fuel rod
    - [ ]<sup>a,c</sup>
  - Zirc-water oxidation reaction based on the Baker-Just model
  - Accounts for fuel rod swelling, burst and blockage
- Requires boundary conditions from the NOTRUMP code

– [ ]<sup>a,c</sup>

– [ ]<sup>a,c</sup>

– [ ]<sup>a,c</sup>

– [ ]<sup>a,c</sup>

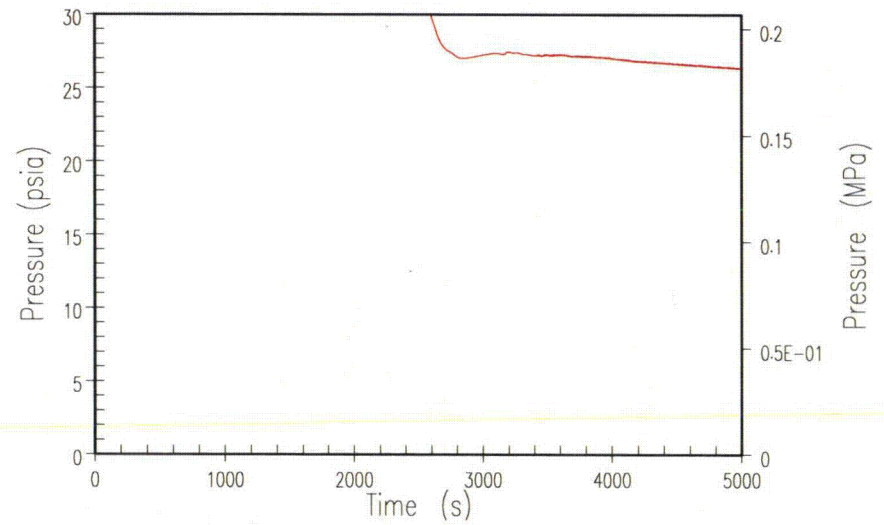
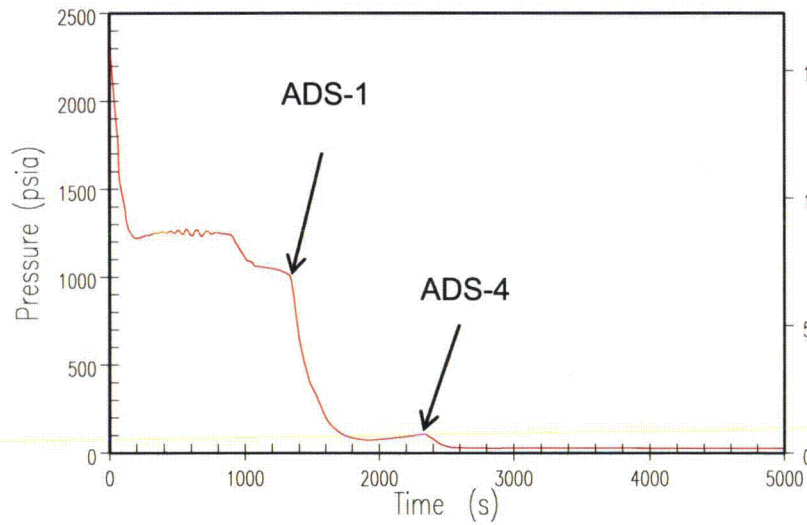
– [ ]<sup>a,c</sup>



## SBLOCA Analysis Results Summary

- Minor core uncover observed for INADS and 2 inch Cold Leg Breaks
  - INADS PCT = 654.7°F
  - 2 inch PCT = 663.5°F
- Larger break simulations not adversely impacted
  - DEDVI
  - 10 inch Cold Leg Break
  - DEDVI entrainment studies not re-performed
- Significant Margin Exists to 10 CFR 50.46 Limits

# 2 inch Cold Leg Break

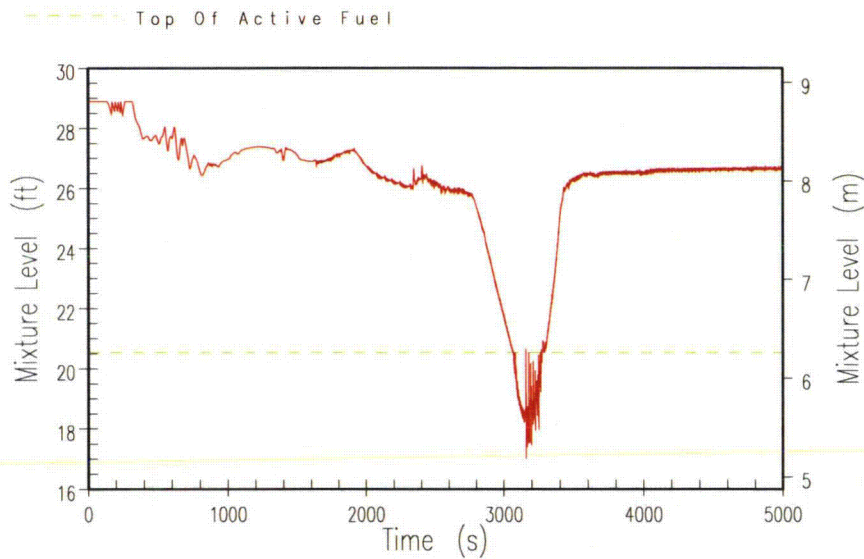


RCS Pressure

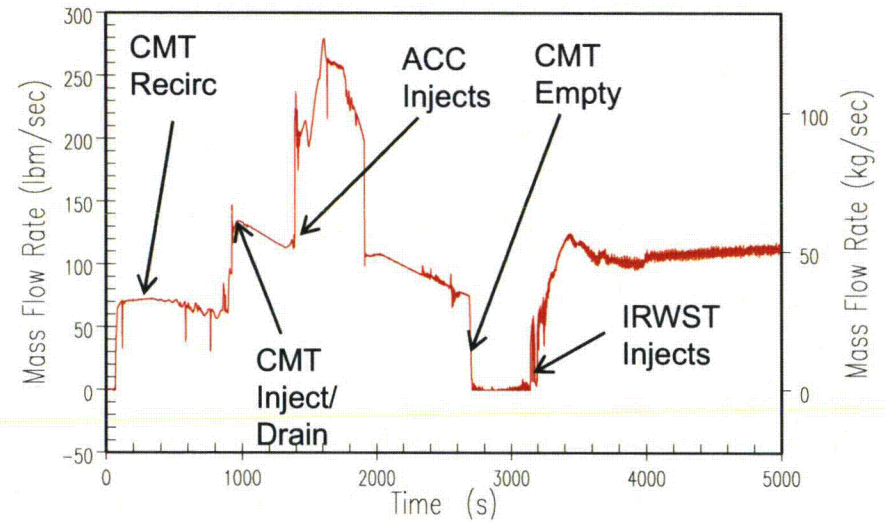




# 2 inch Cold Leg Break



Core/Upper Plenum Mixture Level

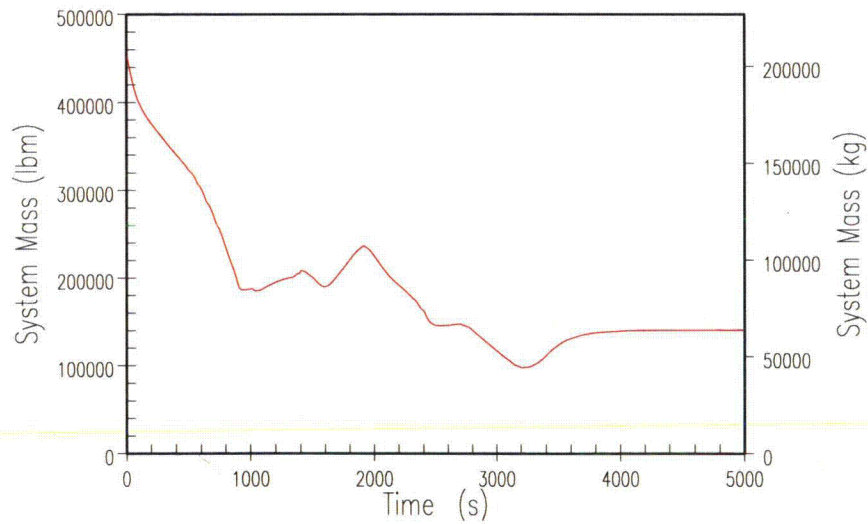


DVI-2 Injection Characteristics

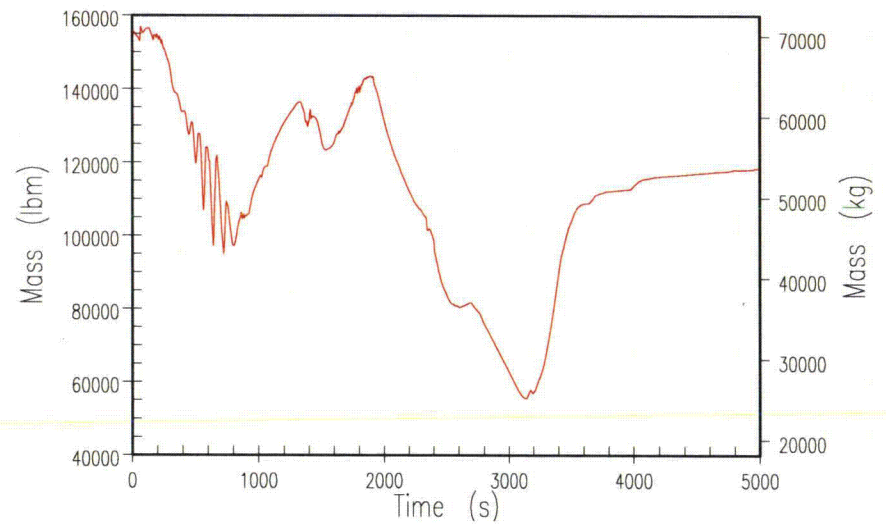




# 2 inch Cold Leg Break



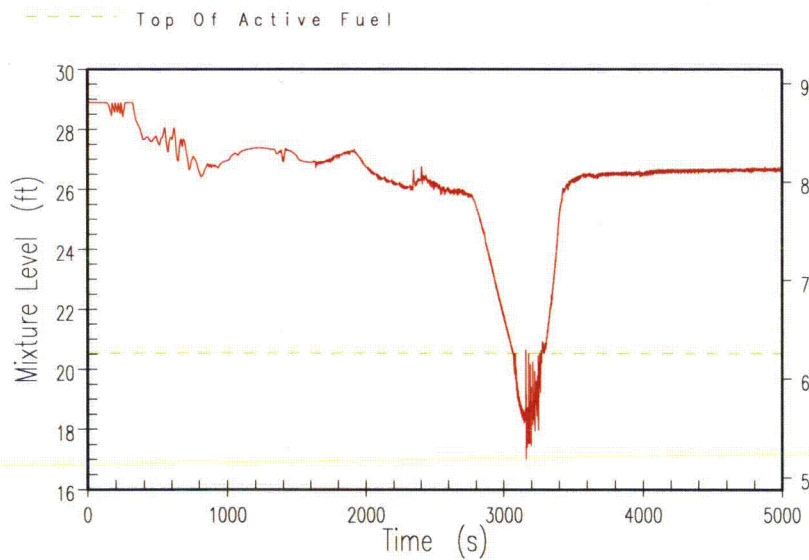
RCS Inventory



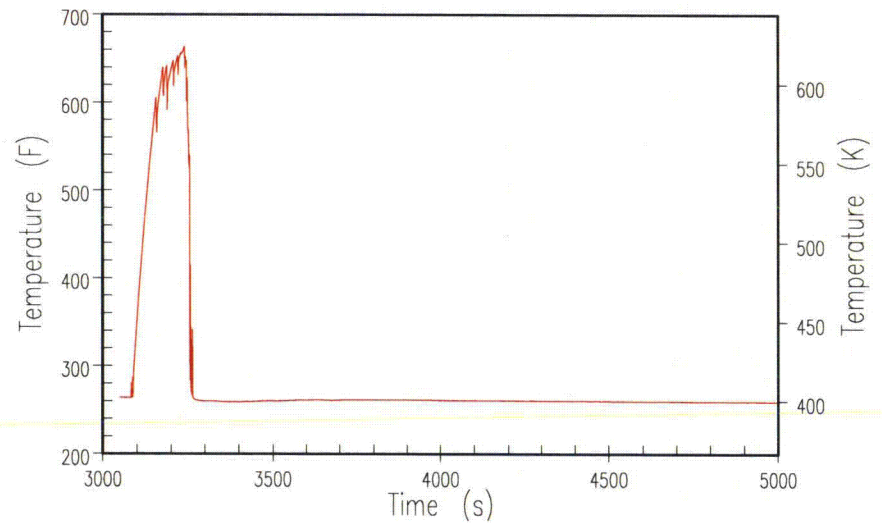
Vessel Mixture Mass



# 2 inch Cold Leg Break



Core/Upper Plenum Mixture Level

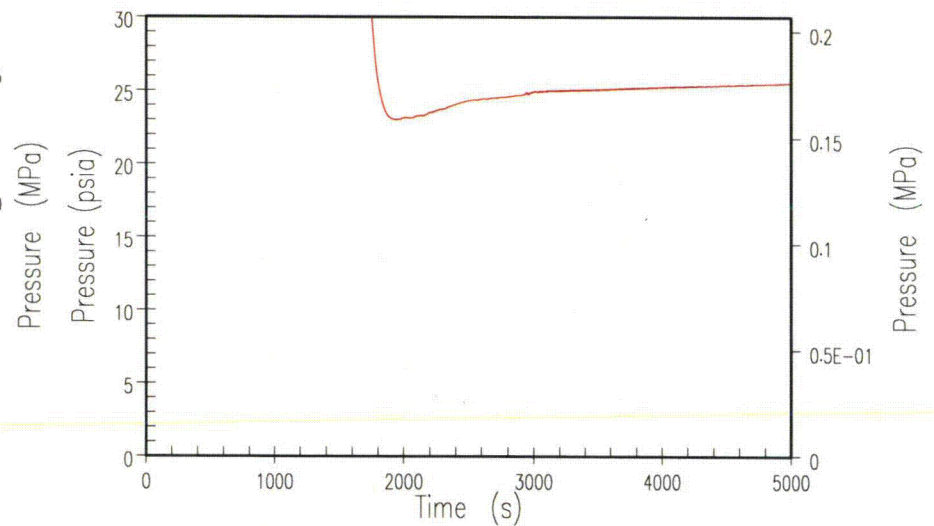
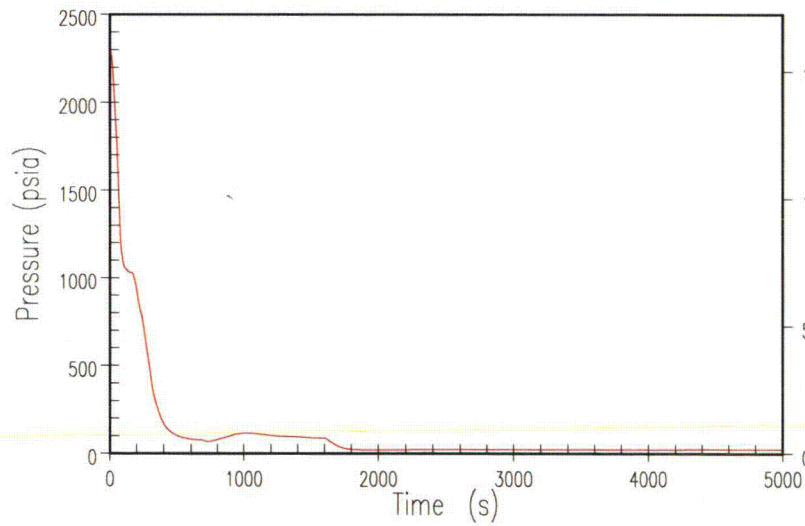


Peak Cladding Temperature





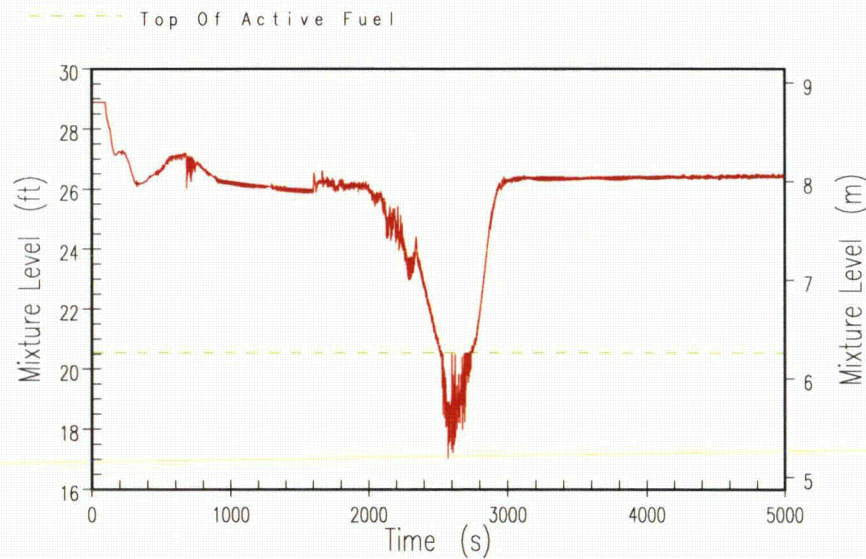
# INADS Results



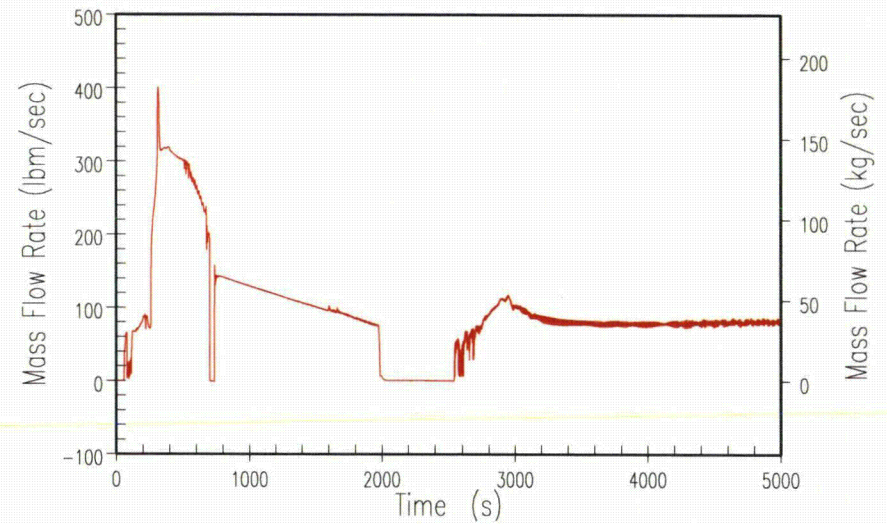
RCS Pressure



# INADS Results



Core/Upper Plenum Mixture Level

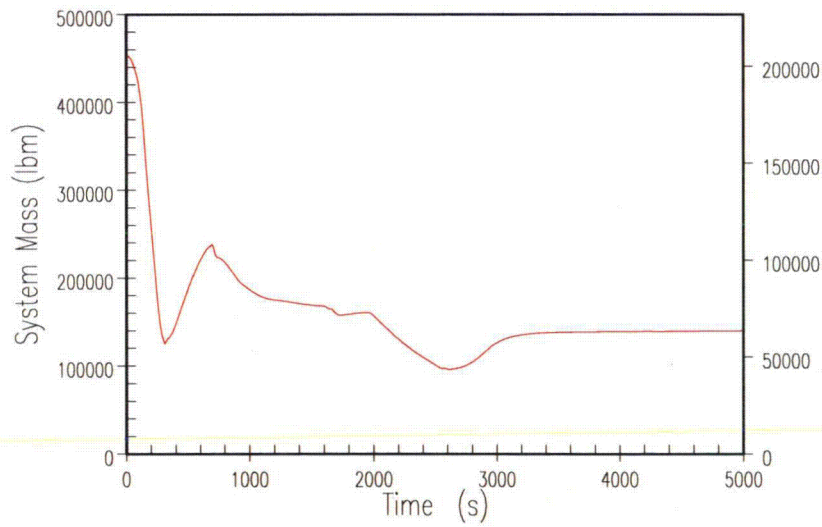


DVI-2 Injection Characteristics

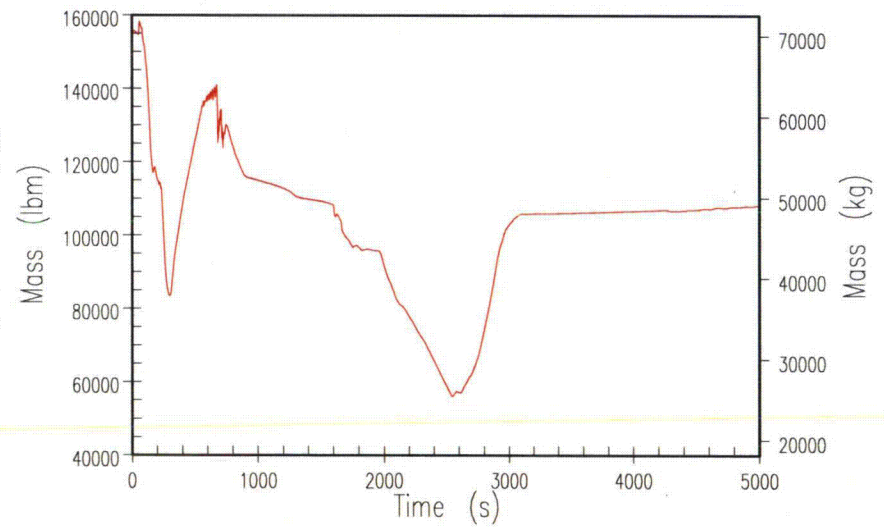




# INADS Results



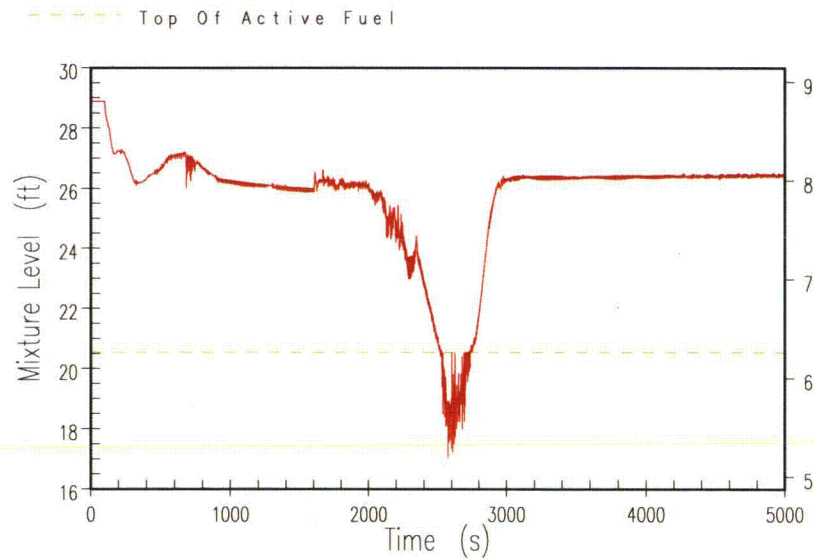
RCS Inventory



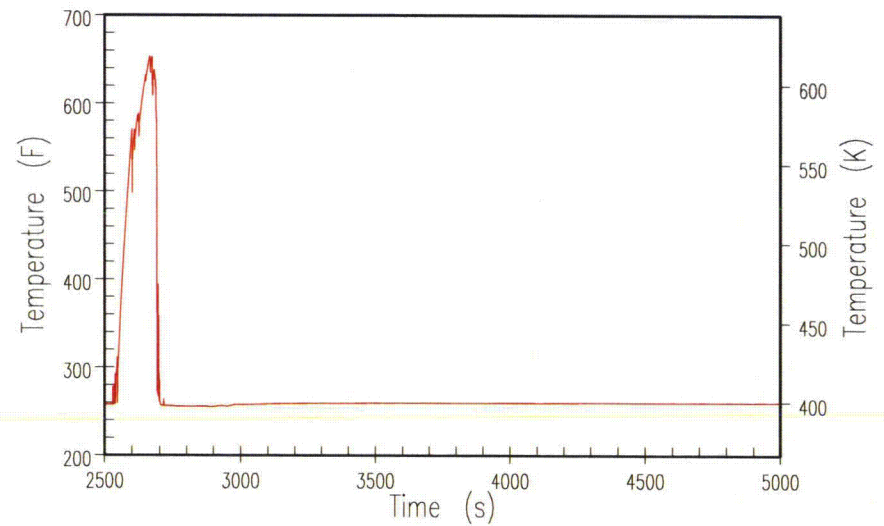
Vessel Mixture Mass



# INADS Results



Core/Upper Plenum Mixture Level

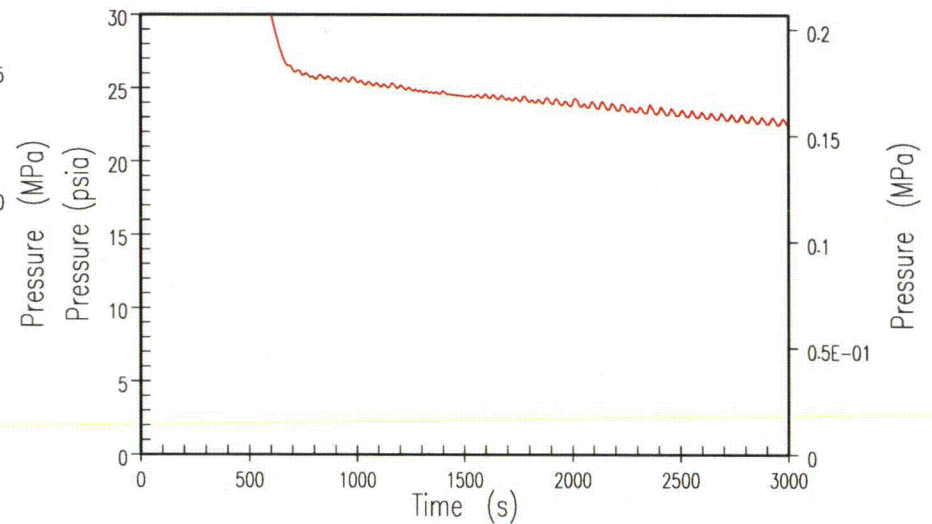
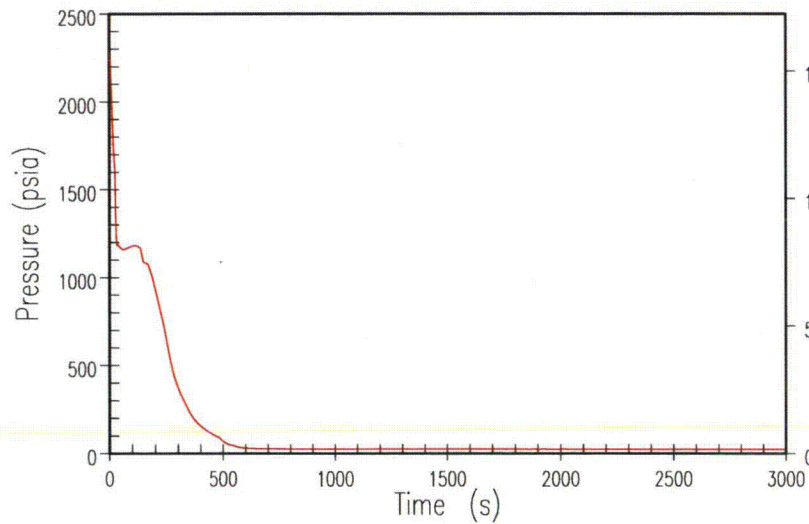


Peak Cladding Temperature





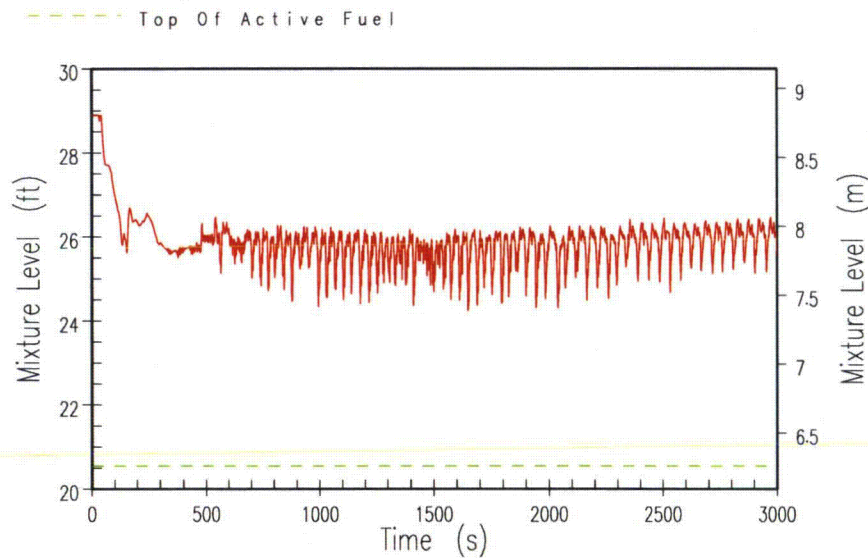
# DEDVI Line Break (20 psi containment pressure)



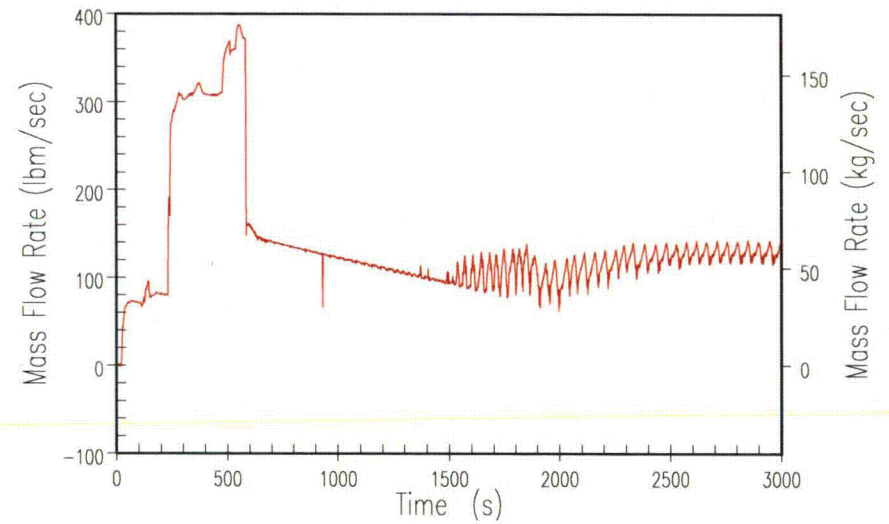
RCS Pressure



# DEDVI Line Break (20 psi containment pressure)



Core/Upper Plenum Mixture Level

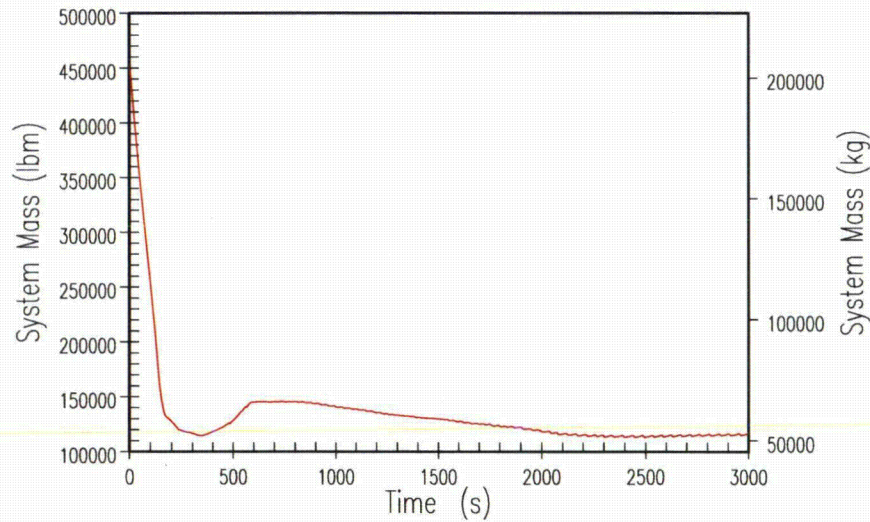


DVI-2 Injection Characteristics

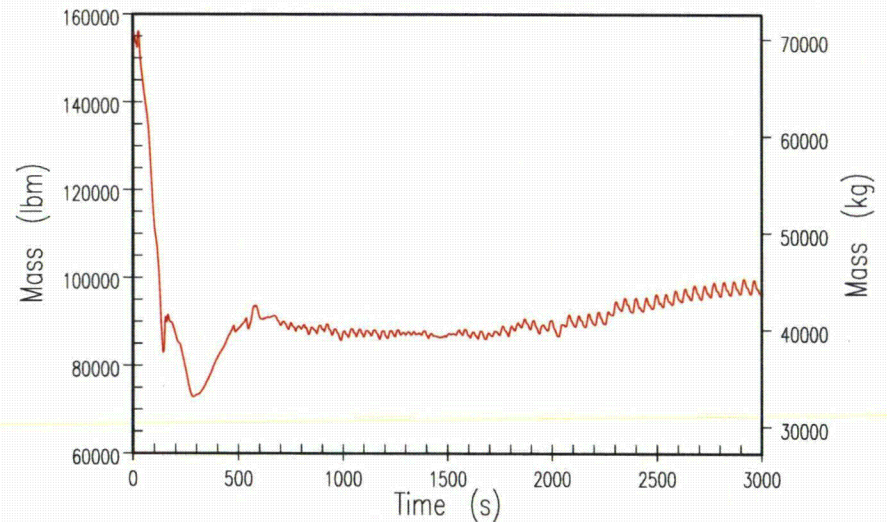




# DEDVI Line Break (20 psi containment pressure)



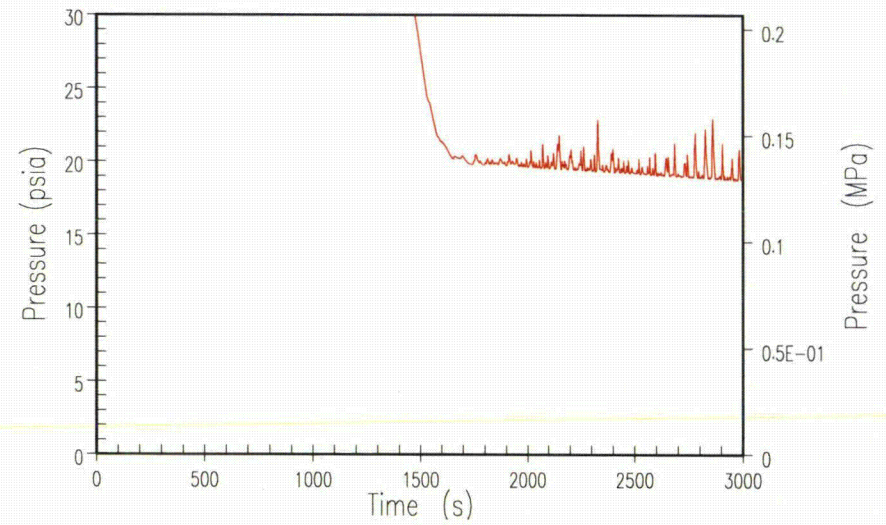
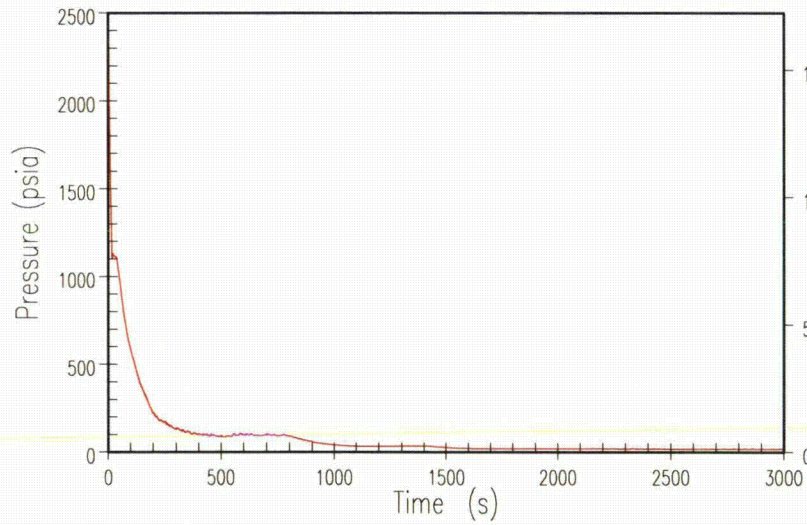
RCS Inventory



Vessel Mixture Mass



# 10 inch Cold Leg Break

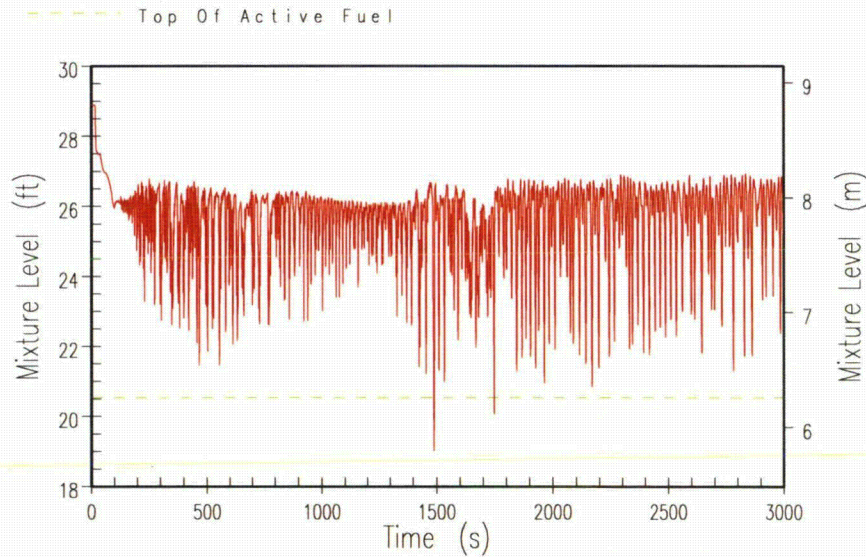


RCS Pressure

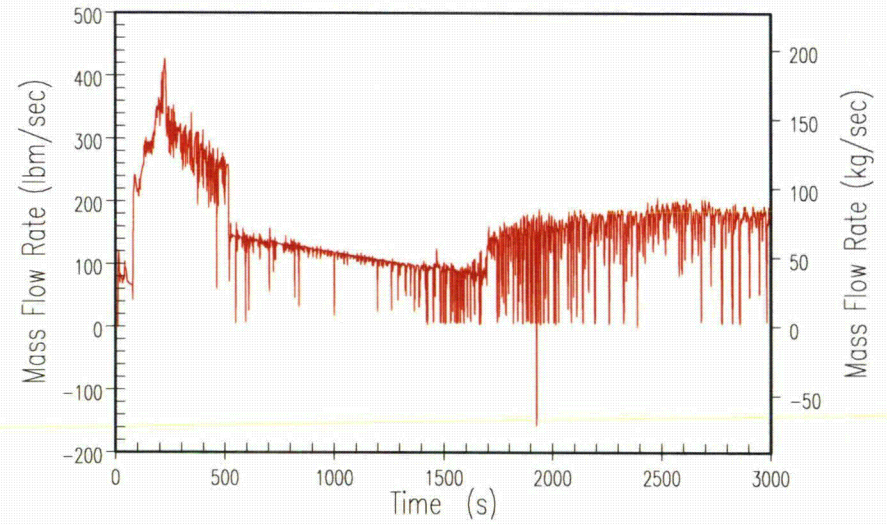




# 10 inch Cold Leg Break



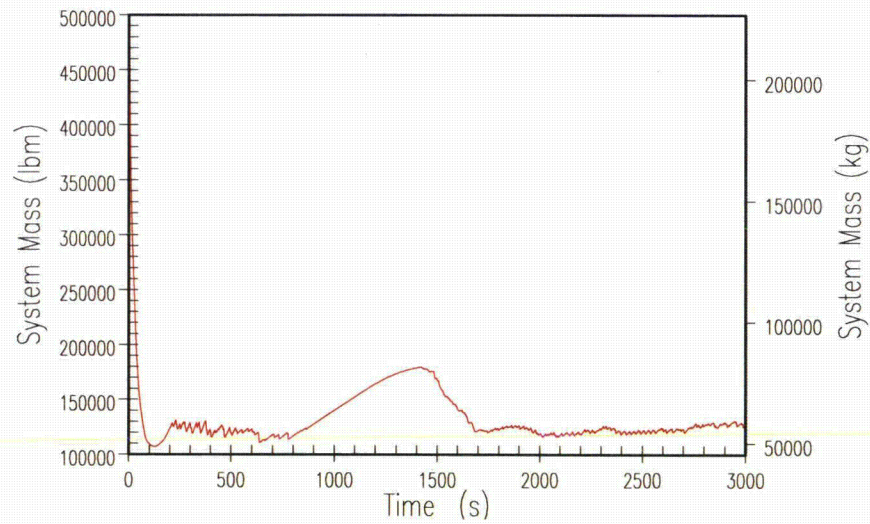
Core/Upper Plenum Mixture Level



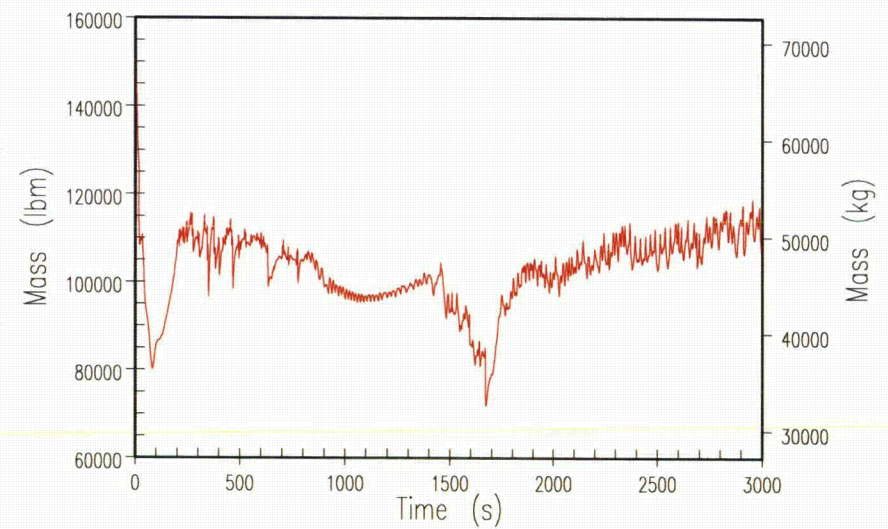
DVI-2 Injection Characteristics



# 10 inch Cold Leg Break



RCS Inventory



Vessel Mixture Mass





# Thermal Conductivity Degradation (TCD) Assessment

- The effects of TCD on SBLOCA for **AP1000** plant were previously assessed in LTR-NRC-12-56, LTR-NRC-12-86 and LTR-NRC-13-18
  - Core stored energy increases due to TCD do not affect SBLOCA due to the nature of the transients
  - Rod internal pressure effects on burst and blockage, if significant core uncover and cladding heat-up predicted, accounted for as part of burn-up studies
  - CRR Rev. 0 transients did not result in core uncover and therefore not affected by TCD
- Due to revision of the SBLOCA CRR results, the impact of TCD has been re-assessed
  - SBLOCA results are negligibly impacted when considering the effects of TCD
    - Initial core stored energy increase removed prior to uncover
    - PCTs are low therefore fuel rod burst and blockage not a concern for **AP1000** SBLOCA



## Conclusion

- Single-Failure on non-PRHR side ADS-4 discharge path more limiting
  - Affected Core Reference Report sections updated to reflect the limiting failure location
- Limited duration uncover observed for small breaks
- Significant Margin to 10 CFR 50.46 PCT Limits