

**Figure 2.4-65 thru Figure 2.4-67 Are Not Used**

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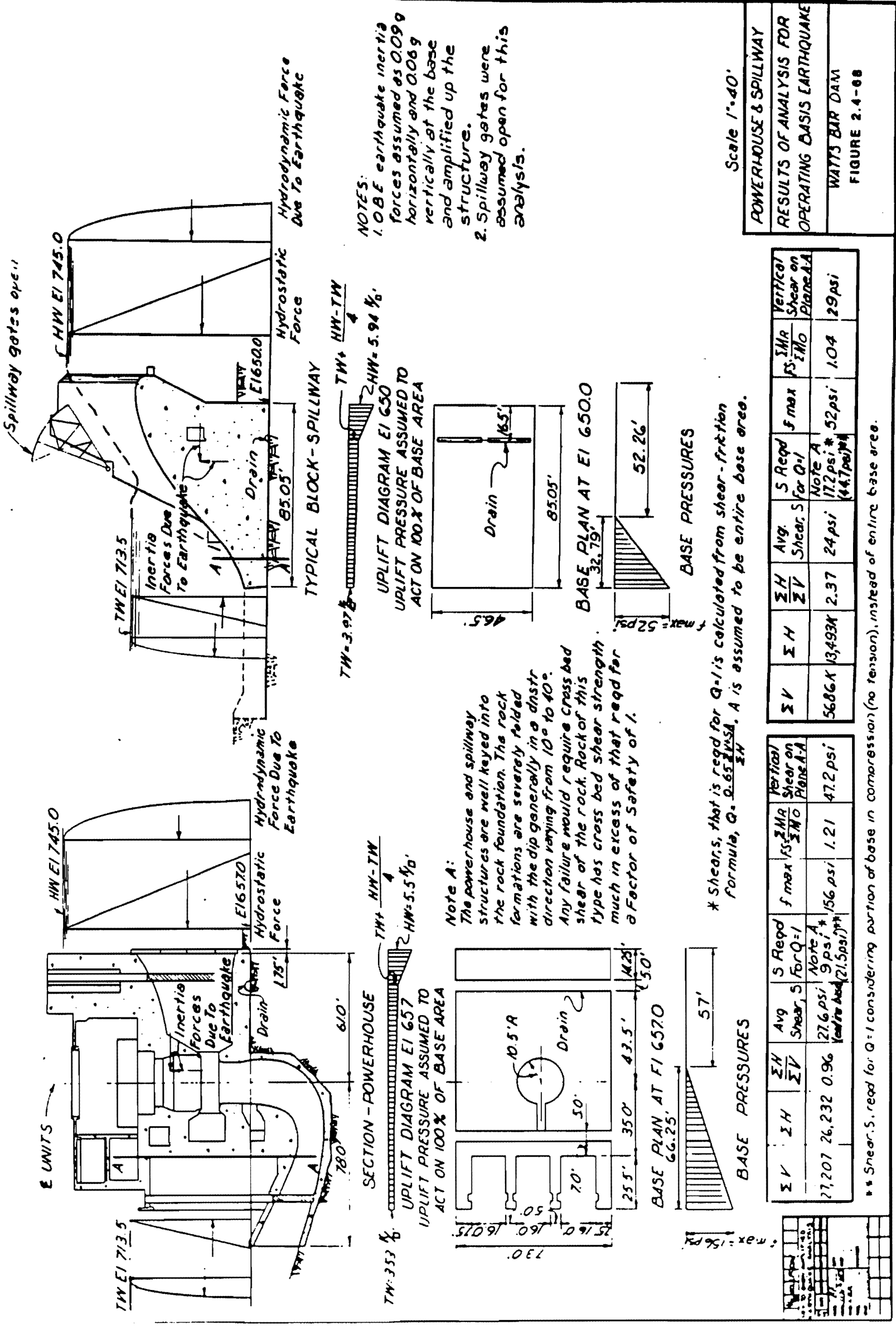


Figure 2.4-68 Powerhouse & Spillway Results of Analysis For Operating Basis Earthquake - Watts Bar Dam

Amendment 63

Figure 2.4-69 Deleted by Amendment 108

**Figure 2.4-70 Deleted by Amendment 83**

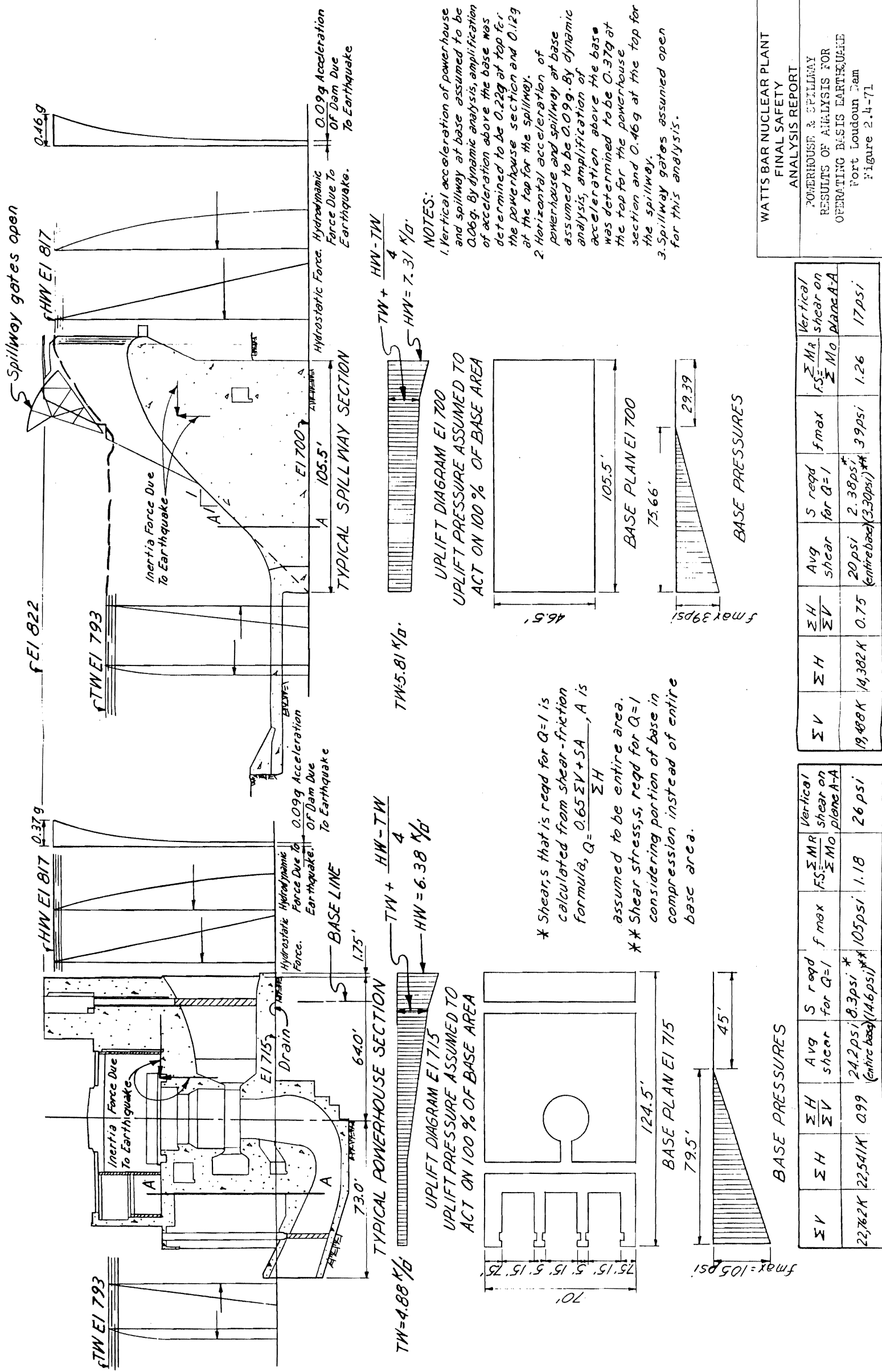
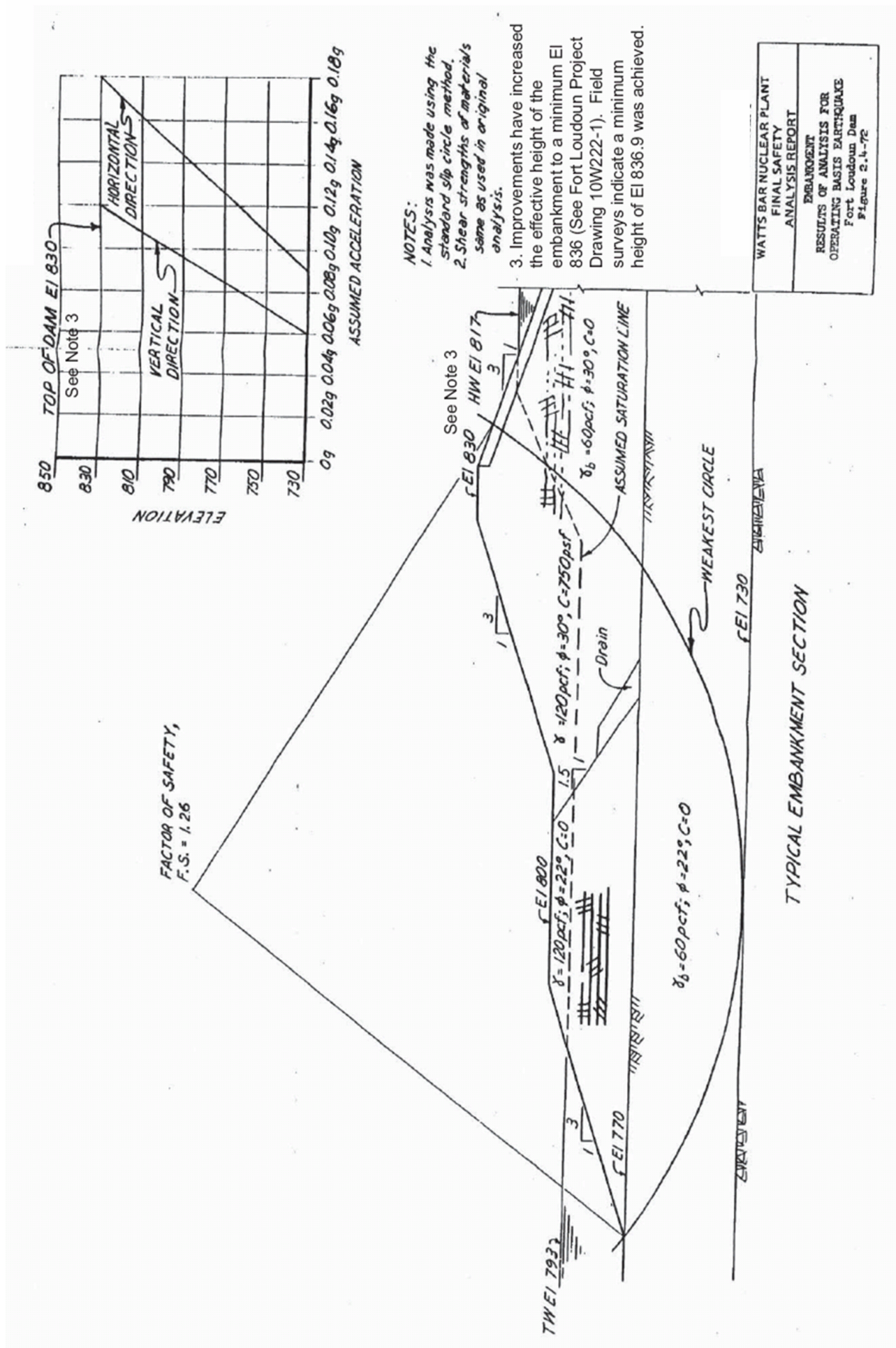


Figure 2.4-71 Powerhouse & Spillway Results of Analysis For Operating Basis Earthquake - Fort Loudoun Dam



**NOTES:**  
 1. Analysis was made using the standard slip circle method.  
 2. Shear strengths of materials same as used in original analysis.  
 3. Improvements have increased the effective height of the embankment to a minimum EI 836 (See Fort Loudoun Project Drawing 10W222-1). Field surveys indicate a minimum height of EI 836.9 was achieved.

Figure 2.4-72 Embankment Results Of Analysis For Operating Basis Earthquake - Fort Loudoun Dam

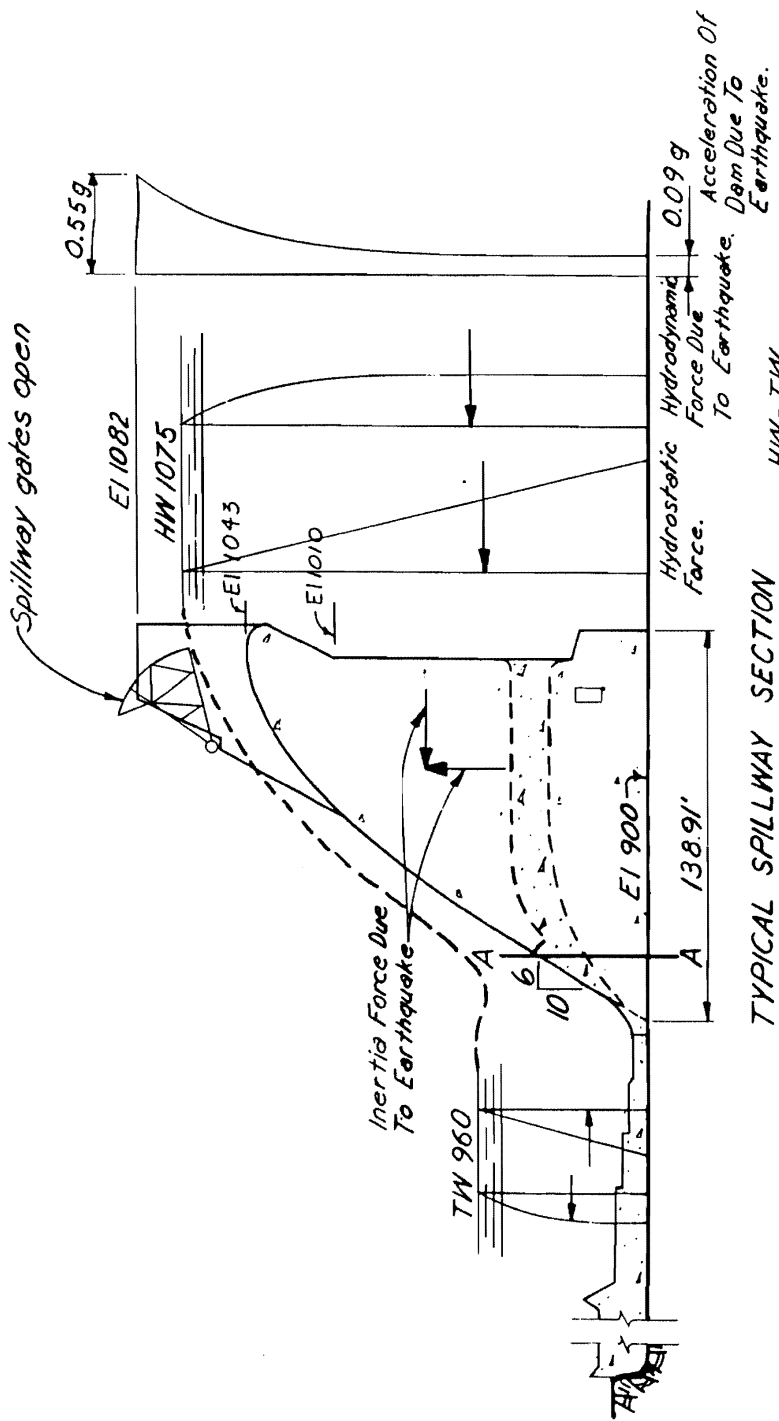
Figure 2.4-73 Deleted by Amendment 103



Figure 2.4-74 Deleted by Amendment 108

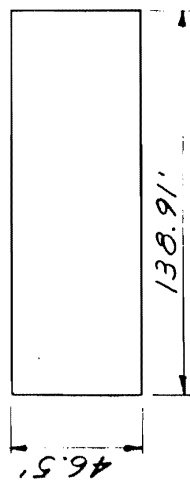
Figure 2.4-75 Deleted by Amendment 103

Figure 2.4-76 Analysis For OBE & 1/2 PMF Assumed Condition of Dam After Failure Norris Dam



$TW = 3.75 \text{ ft}$   
 $HW = 10.94 \text{ ft}$

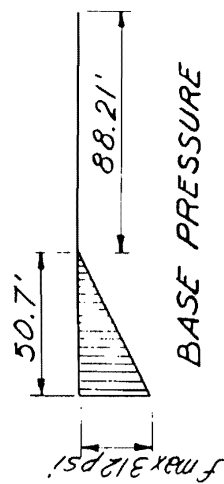
UPLIFT DIAGRAM EI 900  
 UPLIFT PRESSURE ASSUMED TO ACT ON 100% OF BASE AREA



BASE PLAN AT EI 900

\* Shears, that is reqd for  $Q=1$  is calculated from shear-friction formula,  $Q = \frac{0.65 \sum V + SA}{\sum H}$ ,  $A$  is assumed to be entire area.

\*\* Shear stress,  $s$ , reqd for  $Q=1$  considering portion of base in compression instead of entire base area.



BASE PRESSURE

$\sum V$	$\sum H$	$\frac{\sum H}{\sum V}$	Avg Shear, $S$	$S$ Req'd For $Q=1$	$f \text{ max}$	$\frac{\sum MR}{\sum Mo}$	Vertical Shear on Plane A-A
53,007K	57,276K	1.08	61 psi (entire base)	25 psi (67 psi)**	312 psi	1.13	173 psi

- NOTES:
- Vertical acceleration of the spillway at the base assumed to be 0.06 g. By dynamic analysis, amplification of acceleration above the base was determined to be 0.11 g at the top.
  - Horizontal acceleration of the spillway at the base assumed to be 0.09 g. By dynamic analysis, amplification of acceleration above the base was determined to be 0.55 g at the top.
  - Spillway gates assumed open for this analysis.

WATTS BAR NUCLEAR PLANT FINAL SAFETY ANALYSIS REPORT
PROPERTY OF NORTON CONSULTANTS INCORPORATED ENGINEERING BUILDING, 1100 Cherokee Dam Figure 2.4-77

Figure 2.4-77 Spillway & Nonoverflow Results of Analysis For Operating Basis Earthquake -Cherokee Dam

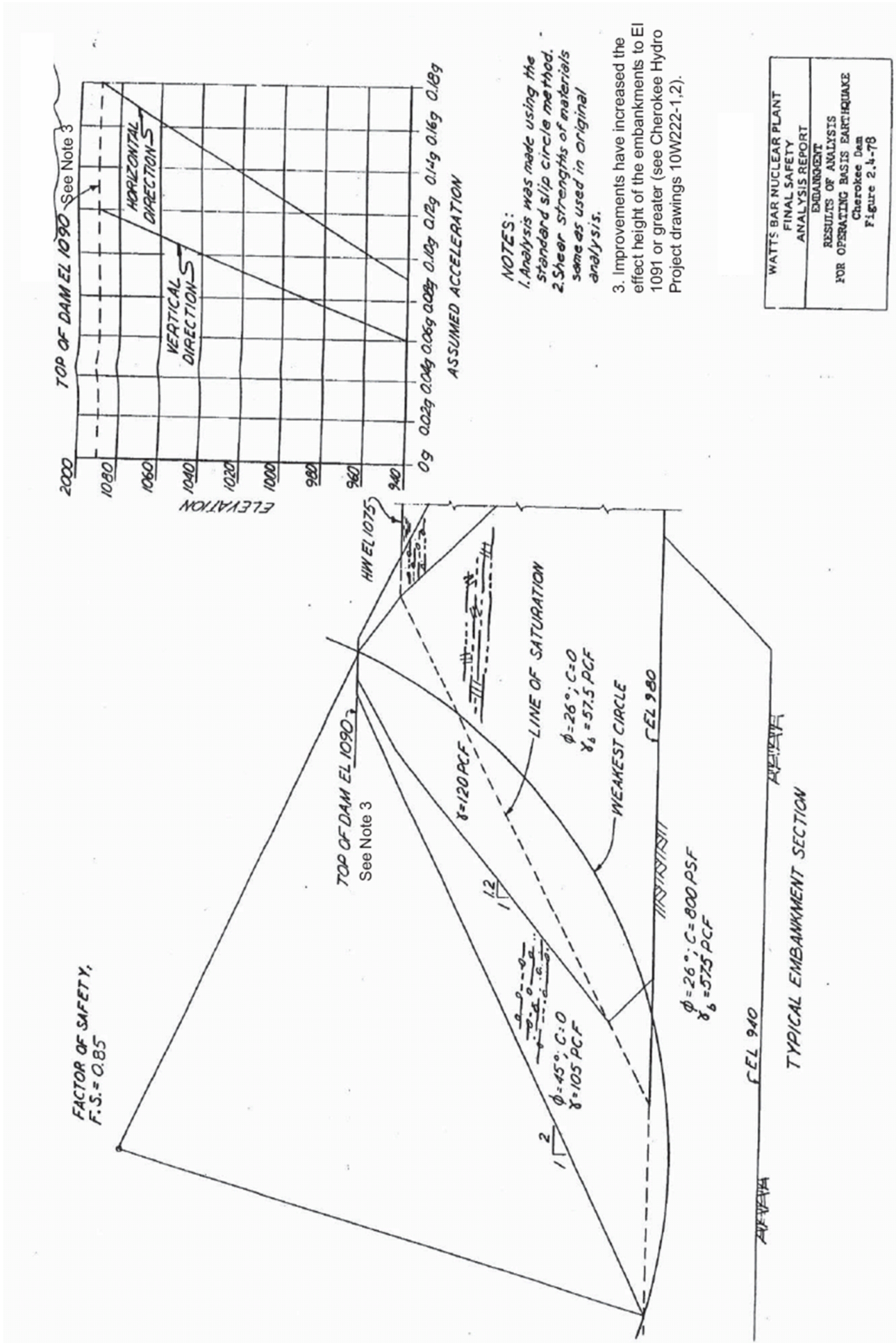
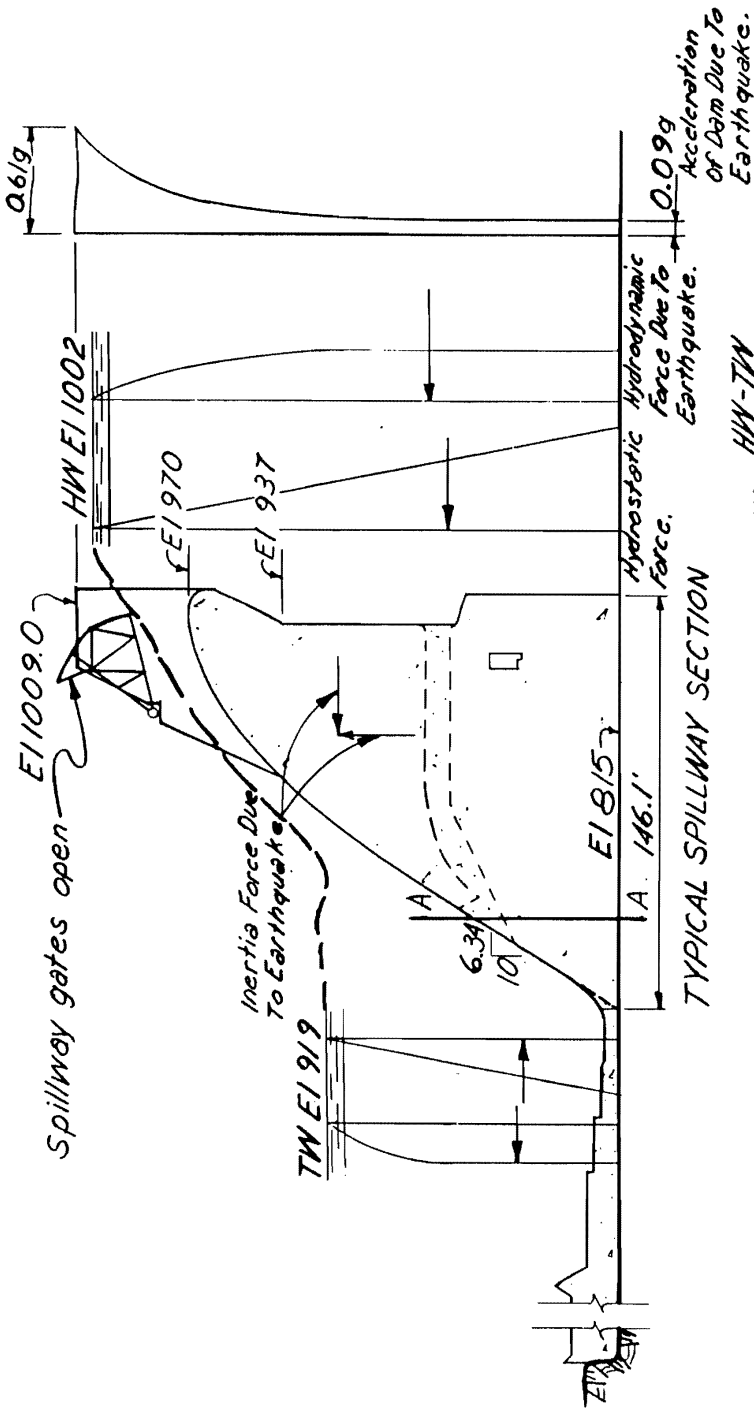


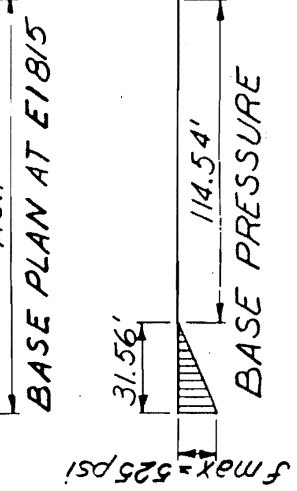
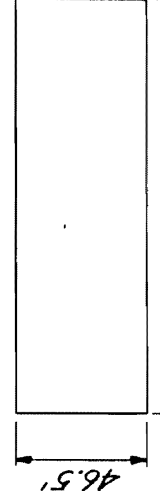
Figure 2.4-78 Embankment Results of Analysis For Operating Basis Earthquake - Cherokee Dam

Figure 2.4-79 Assumed Condition of Dam After Failure PBE And 1/2 Probable Max Flood - Cherokee Dam



$TW = 6.5 \frac{K}{ft}$   
 $HW - TW = \frac{4}{11.7} \frac{K}{ft}$   
 $HW = 11.7 \frac{K}{ft}$

UPLIFT DIAGRAM E1815  
 UPLIFT PRESSURES ASSUMED  
 TO ACT ON 100% OF BASE AREA



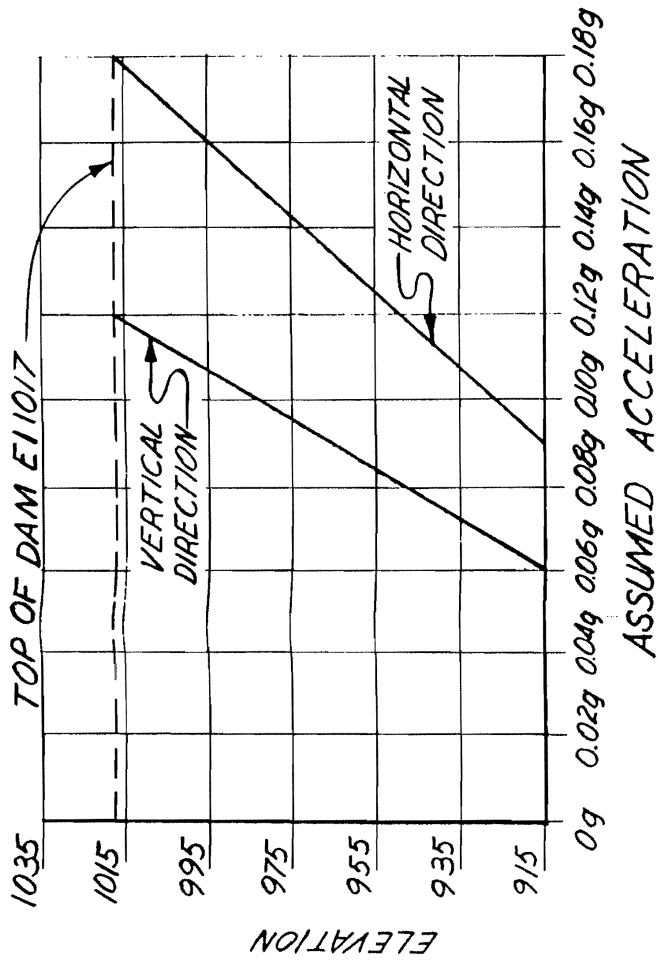
\* Shear,  $s$ , that is reqd for  $Q=1$  is calculated from shear-friction formula,  $Q = \frac{0.65 \Sigma V + SA}{\Sigma H}$ ,  $A$  is assumed to be entire area.  
 \*\* Shear stress,  $s$ , reqd for  $Q=1$  considering portion of base in compression instead of entire base area.

$\Sigma V$	$\Sigma H$	$\frac{\Sigma H}{\Sigma V}$	Avg Shear, $s$	$S$ Reqd For $Q=1$	$f$ max	$\frac{\Sigma MR}{FS \cdot \Sigma MO}$	Vertical Shear on Plane AA
55,483K	60,245K	1.09	61.6 psi (entire base)	25 psi*	525 psi**	1.06	156 psi

NOTES:  
 1. Vertical acceleration of the spillway at the base assumed to be 0.06 g. By dynamic analysis, amplification of acceleration above the base was determined to be 0.13 g at the top.  
 2. Horizontal acceleration of the spillway at the base assumed to be 0.09 g. By dynamic analysis amplification of acceleration above the base was determined to be 0.61 g at the top.  
 3. Spillway gates assumed open for this analysis.

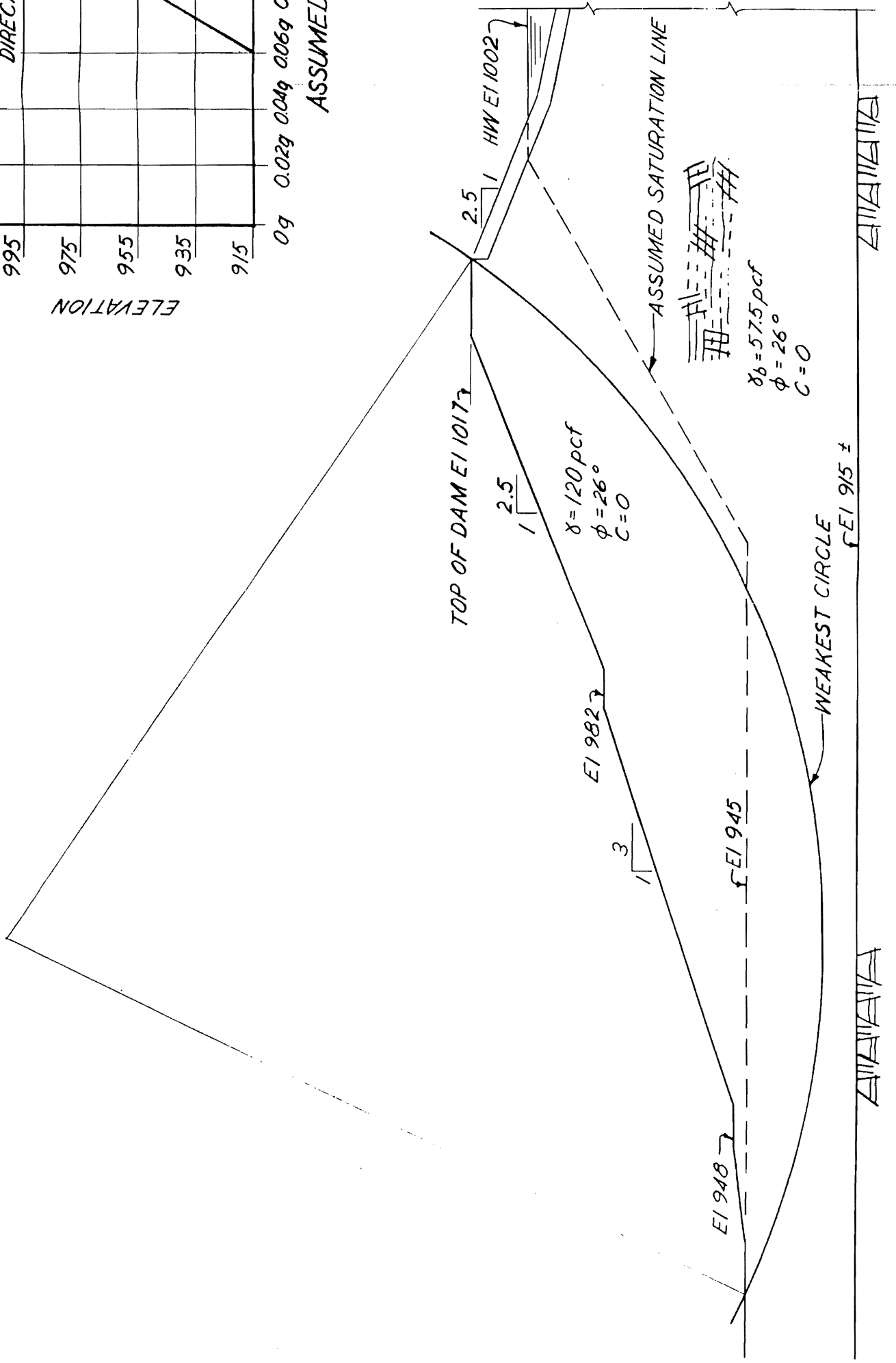
WATTS BAR NUCLEAR PLANT FINAL SAFETY ANALYSIS REPORT
SPILLWAY & NONOVERFLOW RESULTS OF ANALYSIS FOR OPERATING BASIS EARTHQUAKE Douglas Dam Figure 2.4-80

Figure 2.4-80 Spillway & Nonoverflow Results of Analysis For Operating Basis Earthquake - Douglas Dam



NOTES:  
 1. Analysis was made using the standard slip circle method.  
 2. Shear strengths of materials same as used in original analysis.

FACTOR OF SAFETY  
 F.S. = 1.0



SADDLE DAM No. 1

WATTS BAR NUCLEAR PLANT FINAL SAFETY ANALYSIS REPORT
SADDLE DAM NO. 1 RESULTS OF ANALYSIS FOR OPERATING BASIS EARTHQUAKE Douglas Dam Figure 2.4-81

Figure 2.4-81 Saddle Dam No. 1 Results of Analysis For Operating Basis Earthquake - Douglas Dam

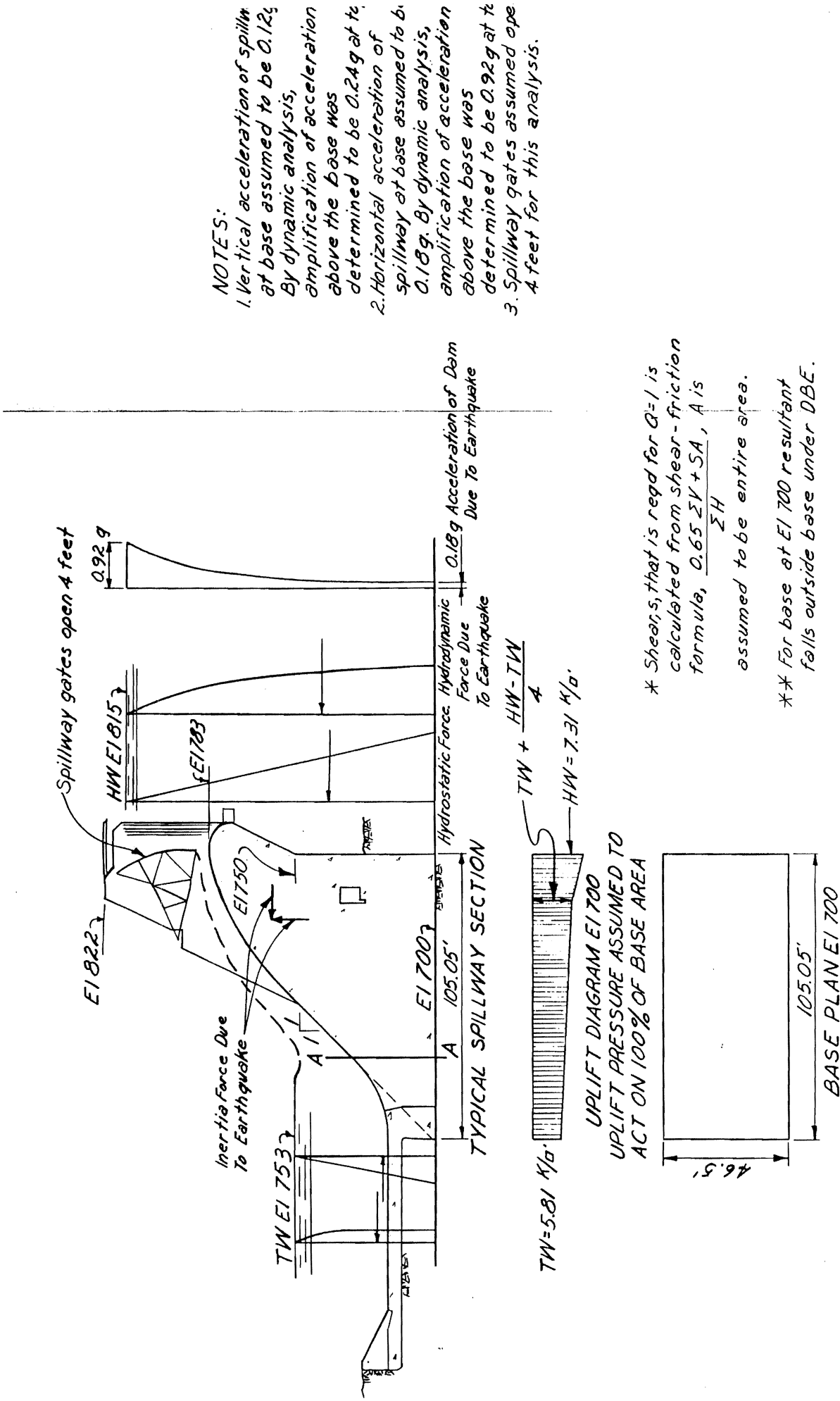


Figure 2.4-82 Douglas Dam Assumed Condition of Dam After Failure aBE And 1/2 Probable Maximum Flood - Douglas Project

Figure 2.4-83 Fontana Dam Assumed Condition of Dam after Failure OBE And 1/2 Probable Maximum Flood

Figure 2.4-84 Deleted by Amendment 63

**Figure 2.4-85 Deleted by Amendment 63**



- NOTES:
1. Vertical acceleration of spillway at base assumed to be 0.12g. By dynamic analysis, amplification of acceleration above the base was determined to be 0.24g at the spillway at base assumed to be 0.18g. By dynamic analysis, amplification of acceleration above the base was determined to be 0.92g at the spillway at base assumed to be 0.18g. By dynamic analysis, amplification of acceleration above the base was determined to be 0.92g at the spillway at base assumed to be 0.18g. By dynamic analysis, amplification of acceleration above the base was determined to be 0.92g at the spillway at base assumed to be 0.18g.
  2. Horizontal acceleration of spillway at base assumed to be 0.18g. By dynamic analysis, amplification of acceleration above the base was determined to be 0.92g at the spillway at base assumed to be 0.18g.
  3. Spillway gates assumed open 4 feet for this analysis.

\* Shear,  $s$ , that is reqd for  $Q=1$  is calculated from shear-friction formula,  $\frac{0.65 \sum V + SA}{\sum H}$ ,  $A$  is assumed to be entire area.

\*\* For base at EI 700 resultant falls outside base under DBE.

BASE PRESSURE \*\*

$\sum V$	$\sum H$	$\frac{\sum H}{\sum V}$	Avg shear	$S_{reqd}$ for $Q=1$	$f_{max}$	$\frac{FS \sum MR}{\sum MO}$
19,254K	29,534K	1.62	42 psi (entire base)	25 psi *	* *	0.9

WATTS BAR NUCLEAR PLANT FINAL SAFETY ANALYSIS REPORT SPILLWAY
RESULTS OF ANALYSIS FOR SSE EARTHQUAKE Fort Loudoun Dam Figure 2.4-86

Figure 2.4-86 Spillway Results of Analysis For SSE Earthquake Fort Loudoun Dam

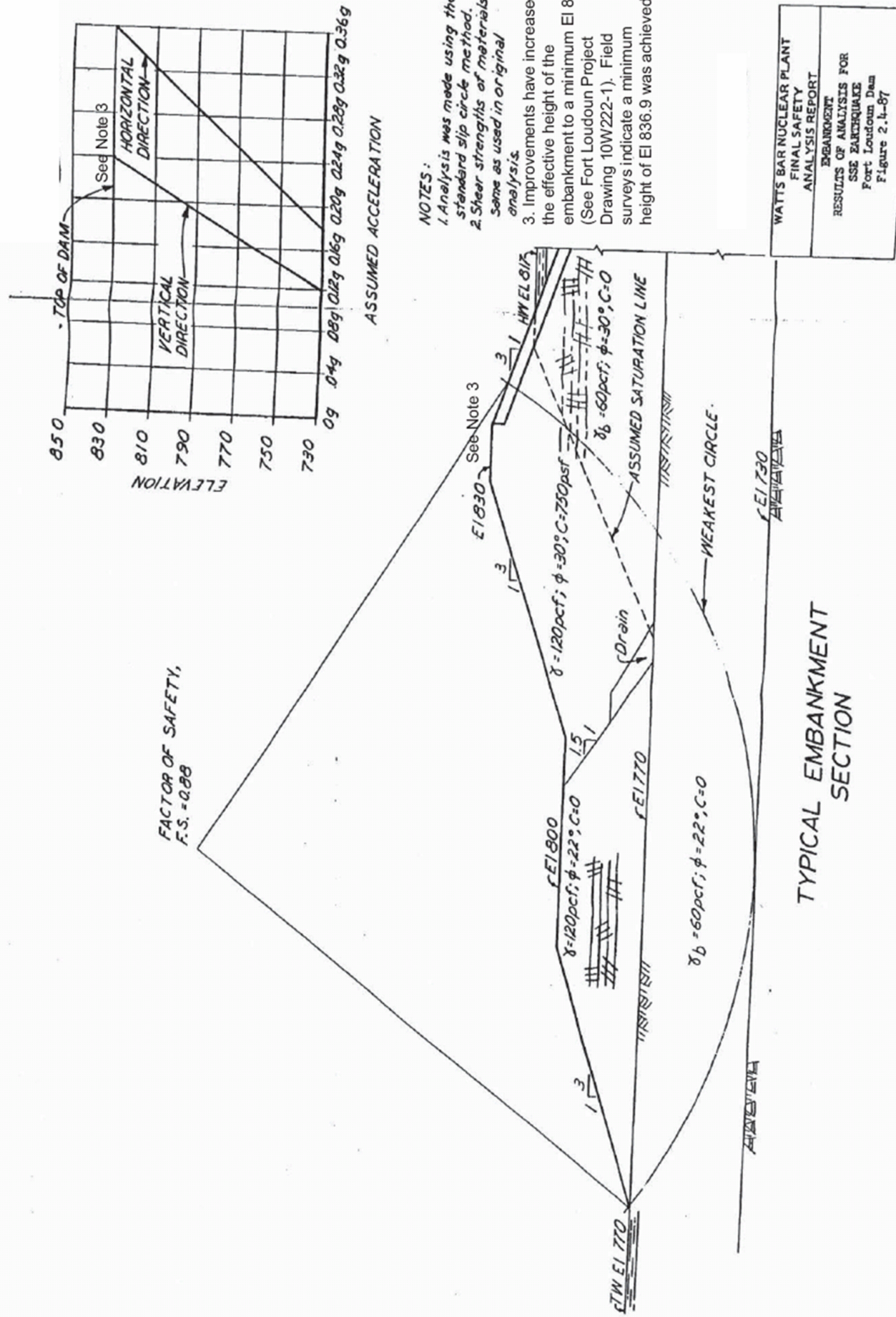


Figure 2.4-87 Embankment Results of Analysis For SSE Earthquake Fort Loudoun Dam

Figure 2.4-88 Fort Loudoun Dam Assumed Condition of Dam After Failure SSE Combined With a 25 Year Flood - Fort Loudoun Dam

Figure 2.4-89 Tellico Dam Assumed Condition of Dam After Failure SSE Combined With a 25 Year Flood Tellico Project



Figure 2.4-90 Norris Dam SSE + 25 Year Flood Judged Condition of Dam After Failure - Norris Dam

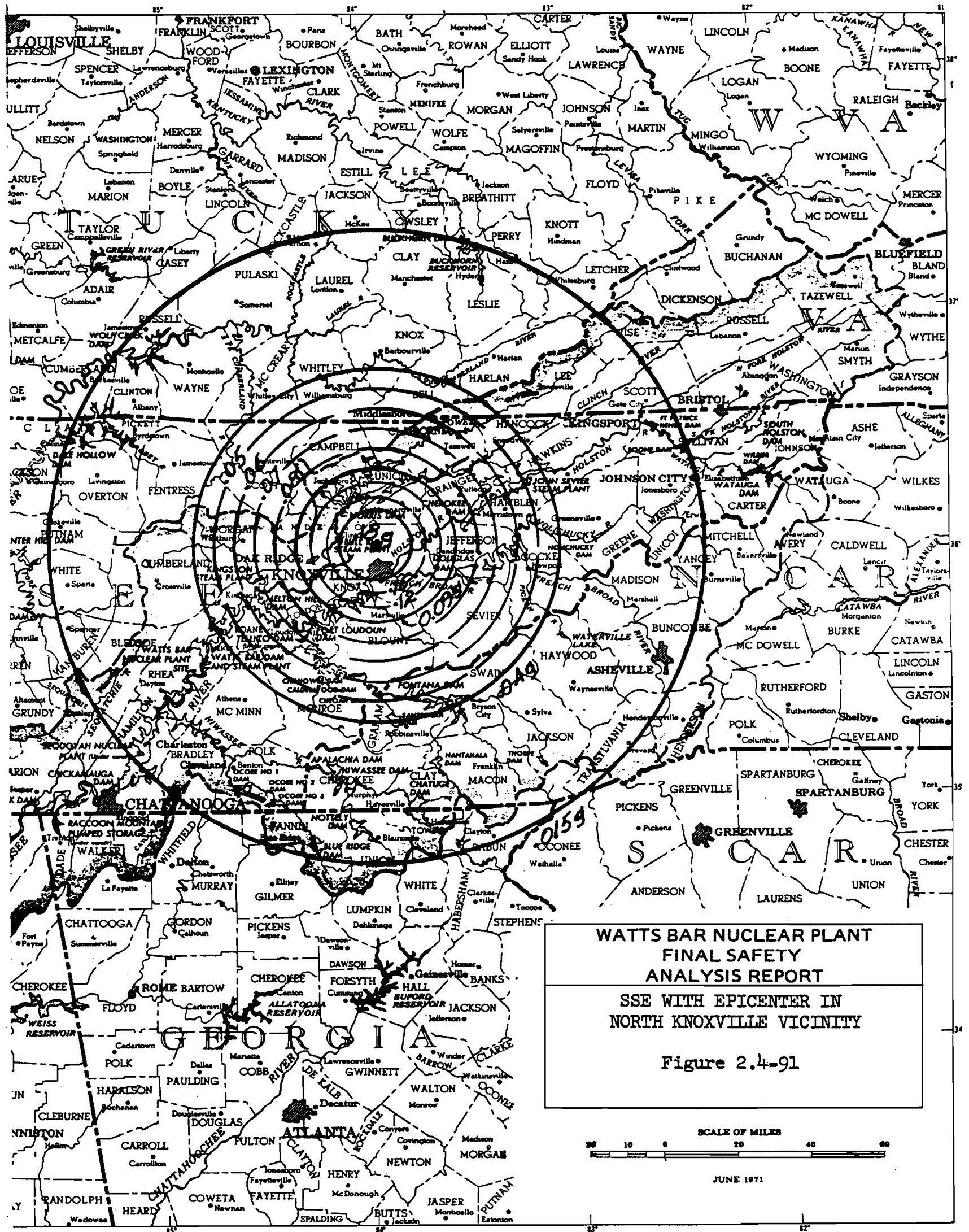


Figure 2.4-91 SSE With Epicenter In North Knoxville Vicinity

Figure 2.4-92 Deleted by Amendment 103

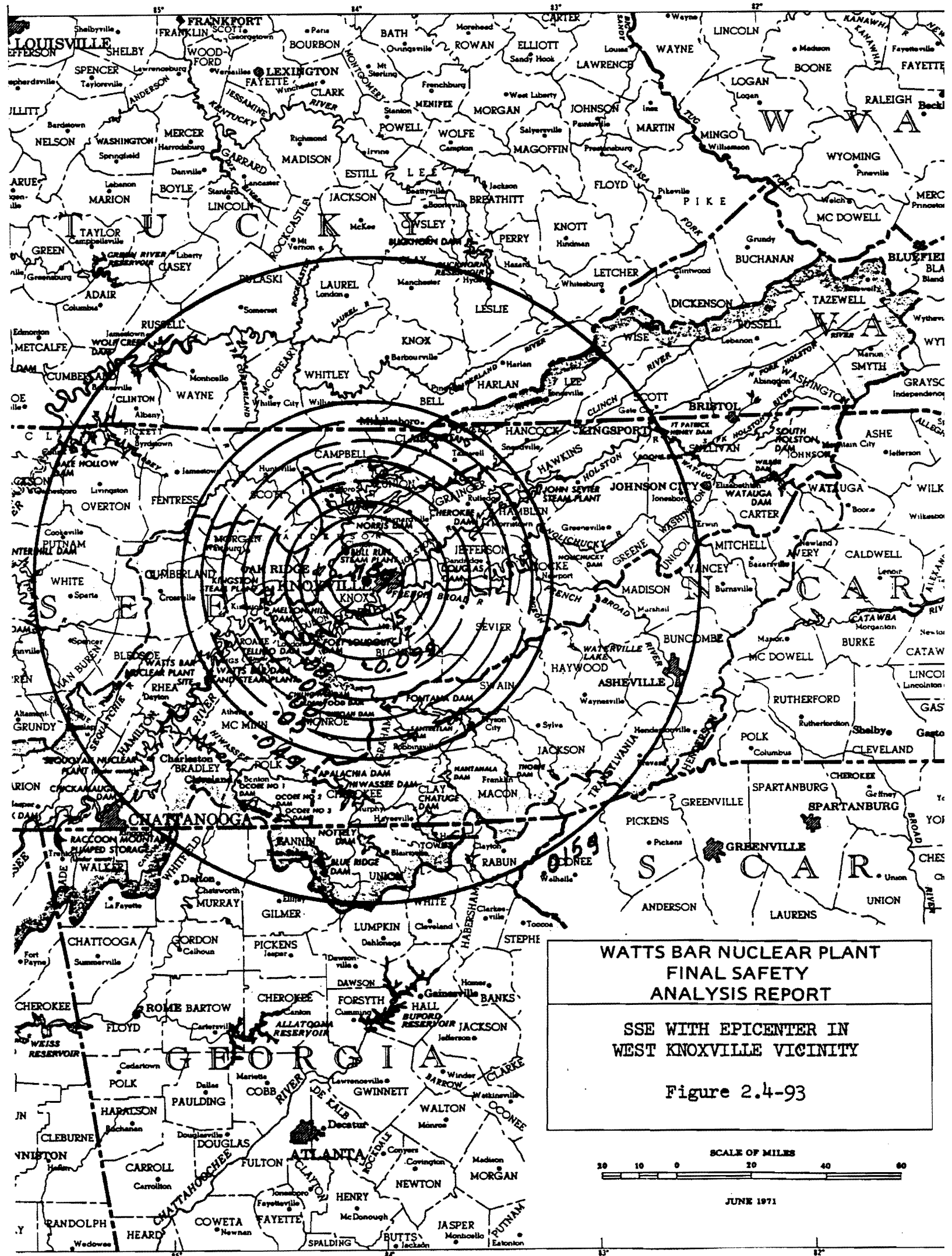


Figure 2.4-93 SSE With Epicenter In West Knoxville Vicinity

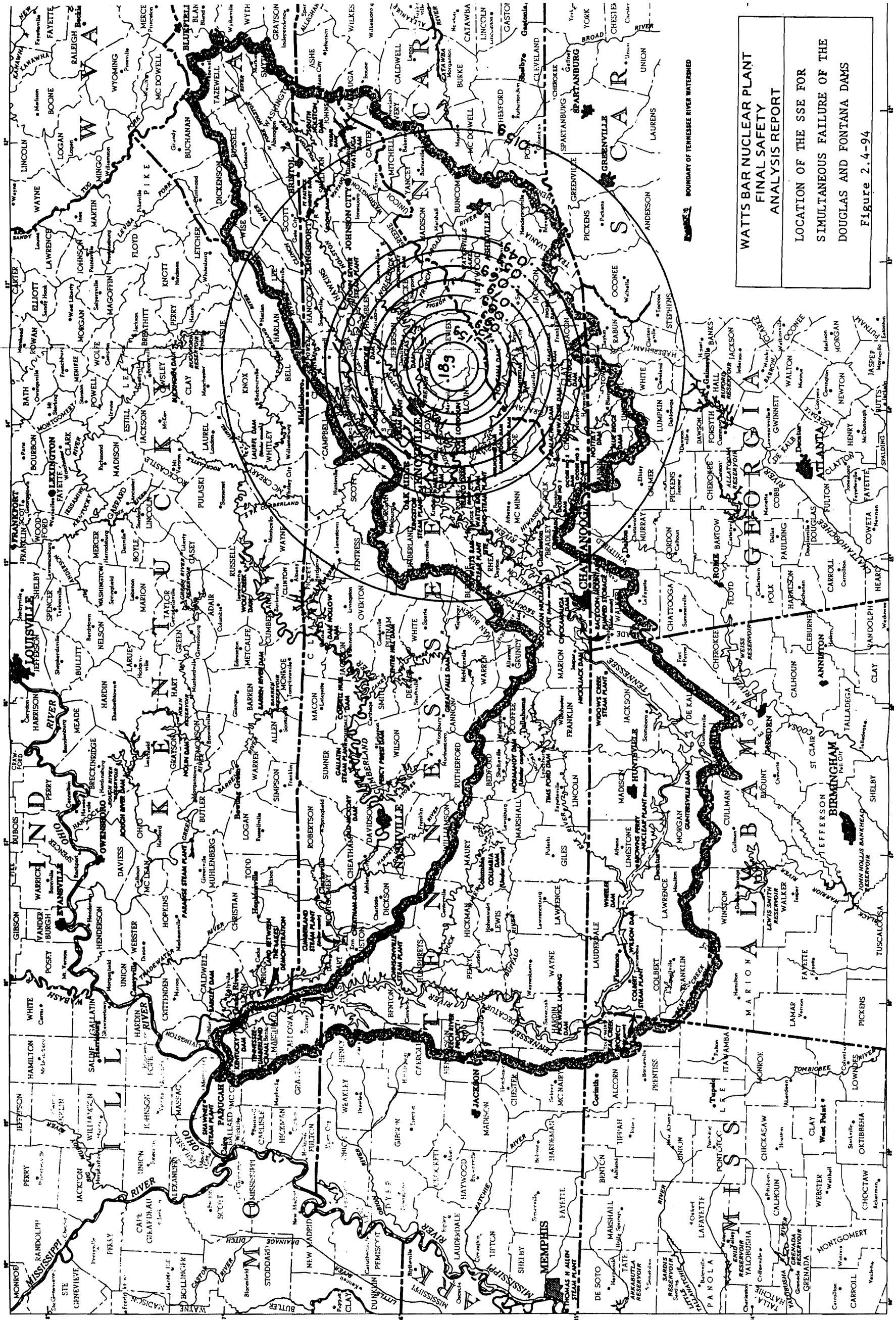


Figure 2.4-94 Location of SSE For Simultaneous Failure of The Douglas and Fontana Dams

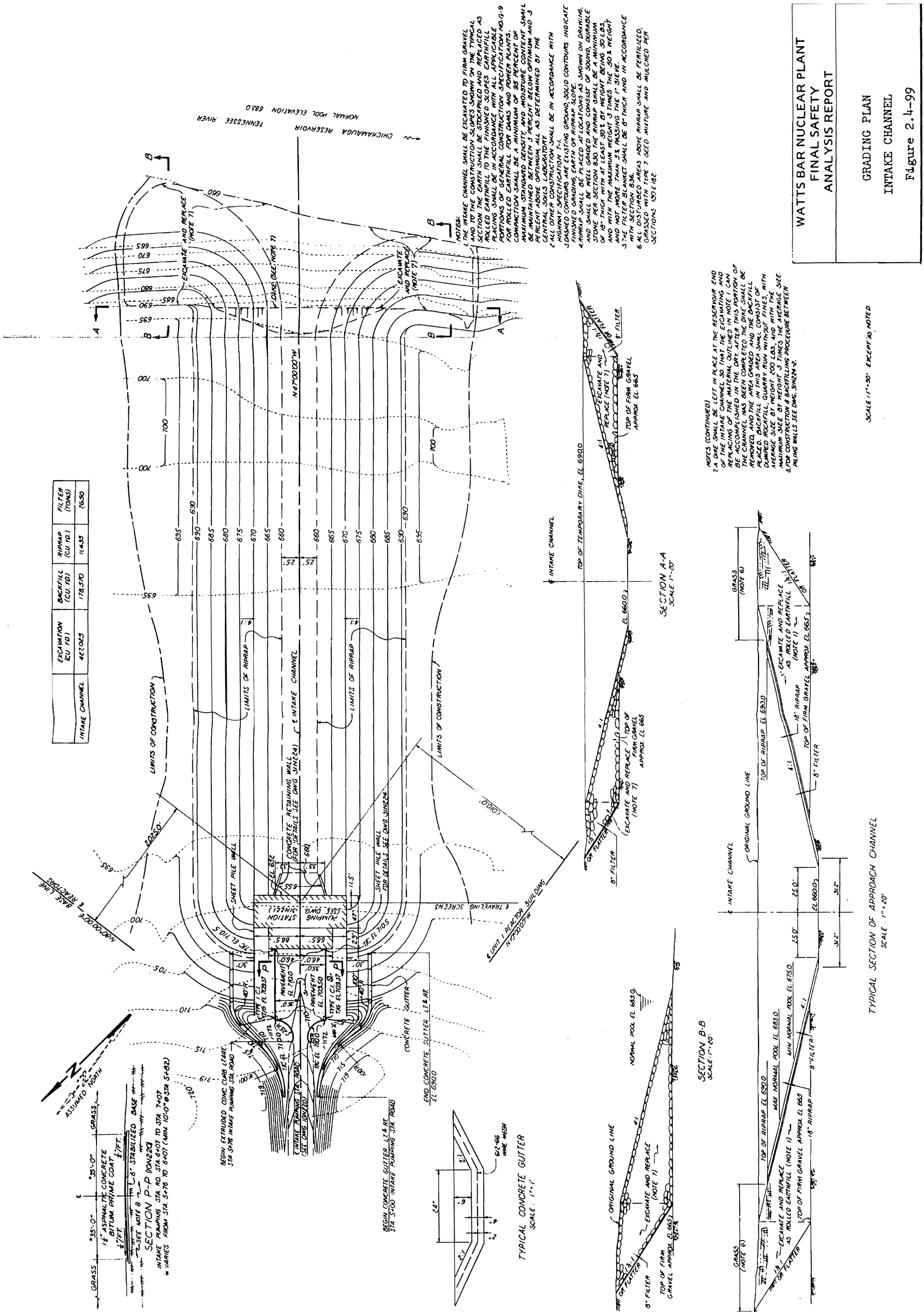
Figure 2.4-95 Deleted by Amendment 103

Figure 2.4-96 Deleted by Amendment 103

Figure 2.4-97 Deleted by Amendment 103



Figure 2.4-98 Not Used



WATTS BAR NUCLEAR PLANT  
FINAL SAFETY  
ANALYSIS REPORT

GRADING PLAN  
INTAKE CHANNEL  
Figure 2.4-99

NOTES (CONTINUED)

2. A DIRT SHALL BE LEFT IN PLACE AT THE RESERVOIR END OF THE CHANNEL SO THAT EXCAVATING AND REPLACING OF THE MATERIAL OUTLINED IN NOTE 1 CAN BE ACCOMPLISHED IN THE DRY. AFTER THIS OPERATION OF THE CHANNEL HAS BEEN COMPLETED THE DIRT SHALL BE REMOVED AND THE AREA GRADED AND THE BACKFILL PLACED. BACKFILL QUARRY RUM SHALL BE FINES WITH AVERAGE SIZE BY WEIGHT 200 LBS. AND WITH THE MAXIMUM SIZE BY WEIGHT 3 TIMES THE AVERAGE SIZE. FOR CONSTRUCTION & BACKFILLING PROCEDURE BETWEEN PILING WALLS SEE DWG. 31924-2.

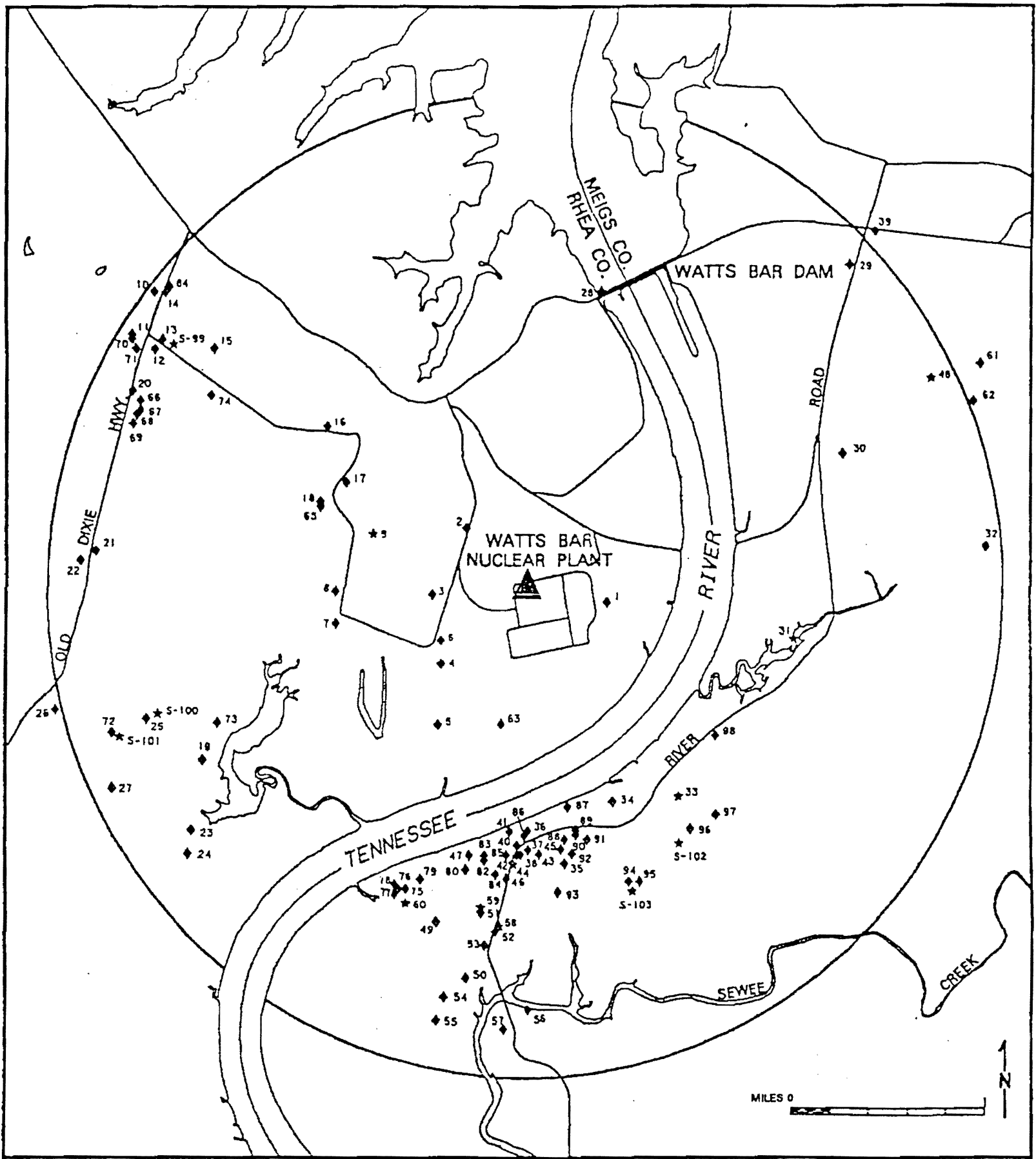
SCALE 1" = 50' EXCEPT AS NOTED

TYPICAL SECTION OF APPROACH CHANNEL  
SCALE 1" = 20'

Figure 2.4-99 Grading Plan Intake Channel

Figure 2.4-100 Deleted by Amendment 83

**Figure 2.4-101 Deleted by Amendment 33**



LEGEND

- ◆ WELL
- ★ SPRING
- ROADS
- 2 MILE RADIUS OF PLANT SITE

AMENDMENT 83

WATTS BAR NUCLEAR PLANT  
FINAL SAFETY  
ANALYSIS REPORT

WELL AND SPRING INVENTORY  
WITHIN 2 MILE RADIUS OF WATTS BAR  
NUCLEAR PLANT SITE  
FSAR FIG 2.4-102

SCANNED DOCUMENT  
THIS IS A SCANNED DOCUMENT MAINTAINED ON  
THE WBNP OPTICRAPHICS SCANNER DATABASE

Figure 2.4-102 Wells And Spring Inventory Within 2-Mile Radius of Watts Bar Nuclear Plant Site

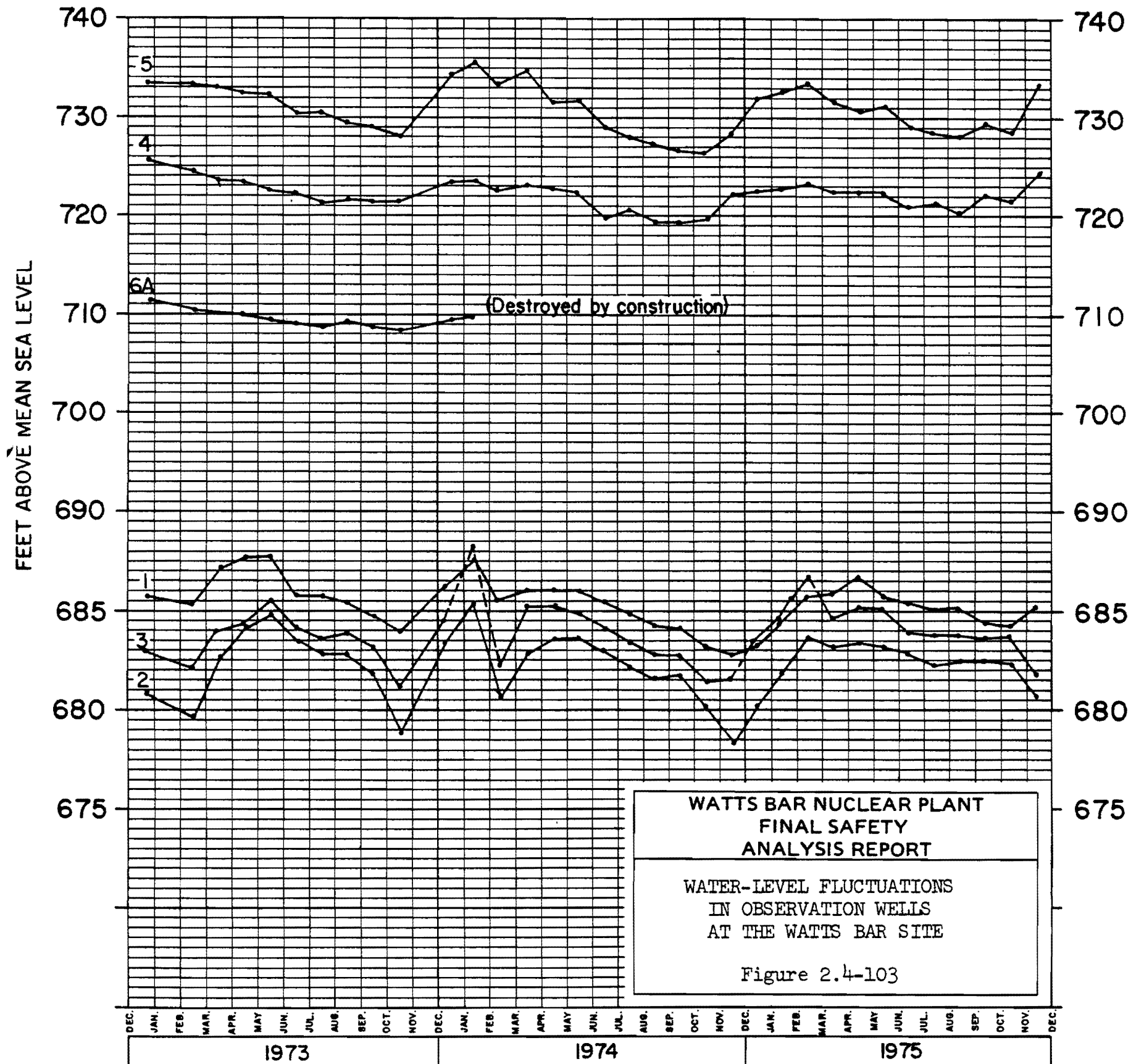
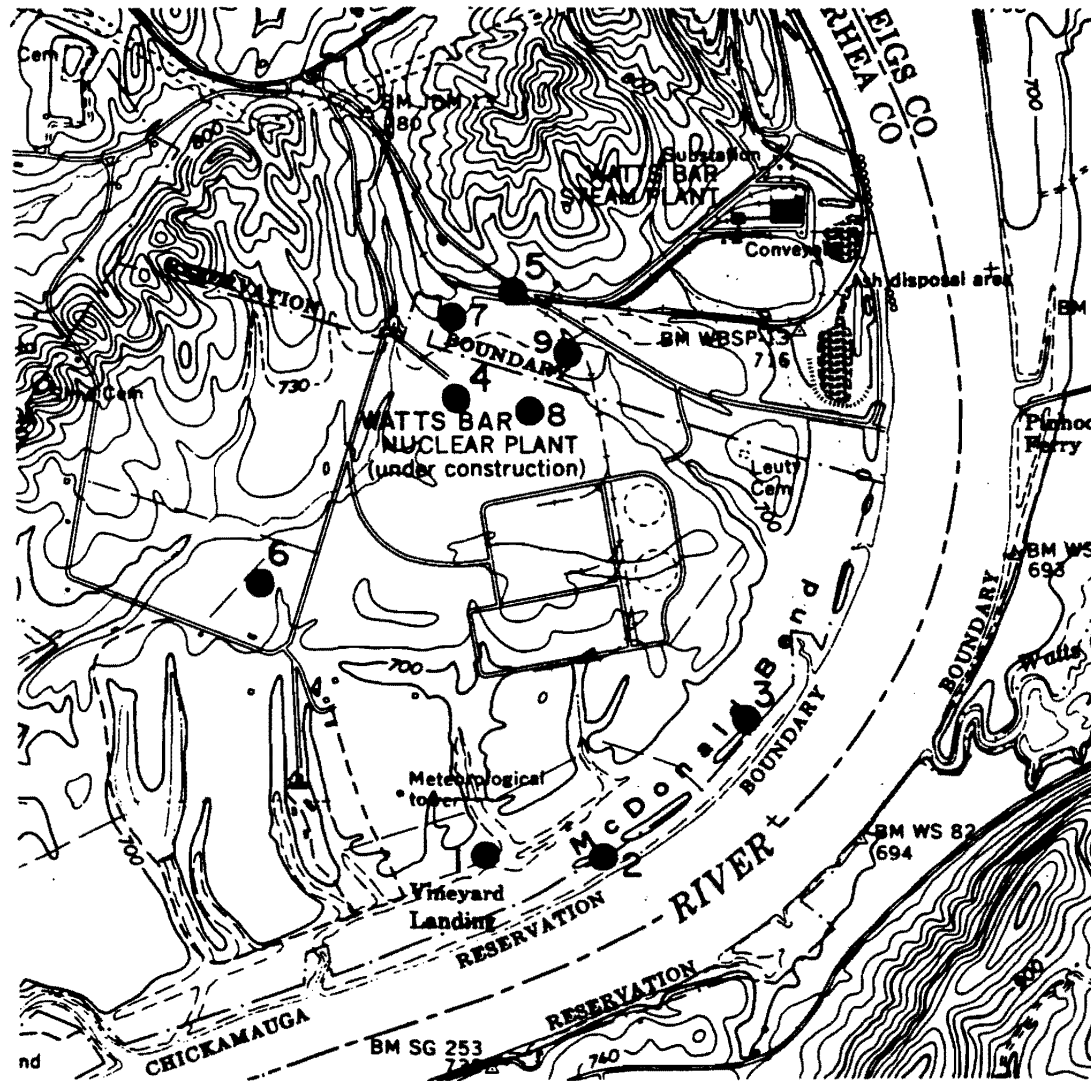


Figure 2.4-103 Water-Level Fluctuations In Observation Wells at The Watts Bar Site



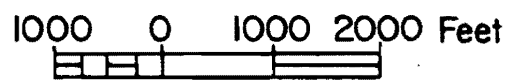
**NOTE:**

Topographic base from U.S.G.S - T.V.A. 7.5 minute quadrangle, Decatur, Tenn., 118-SE, Contour interval 20 feet.

**LEGEND:**

●<sup>2</sup> - Ground-water observation well showing number.

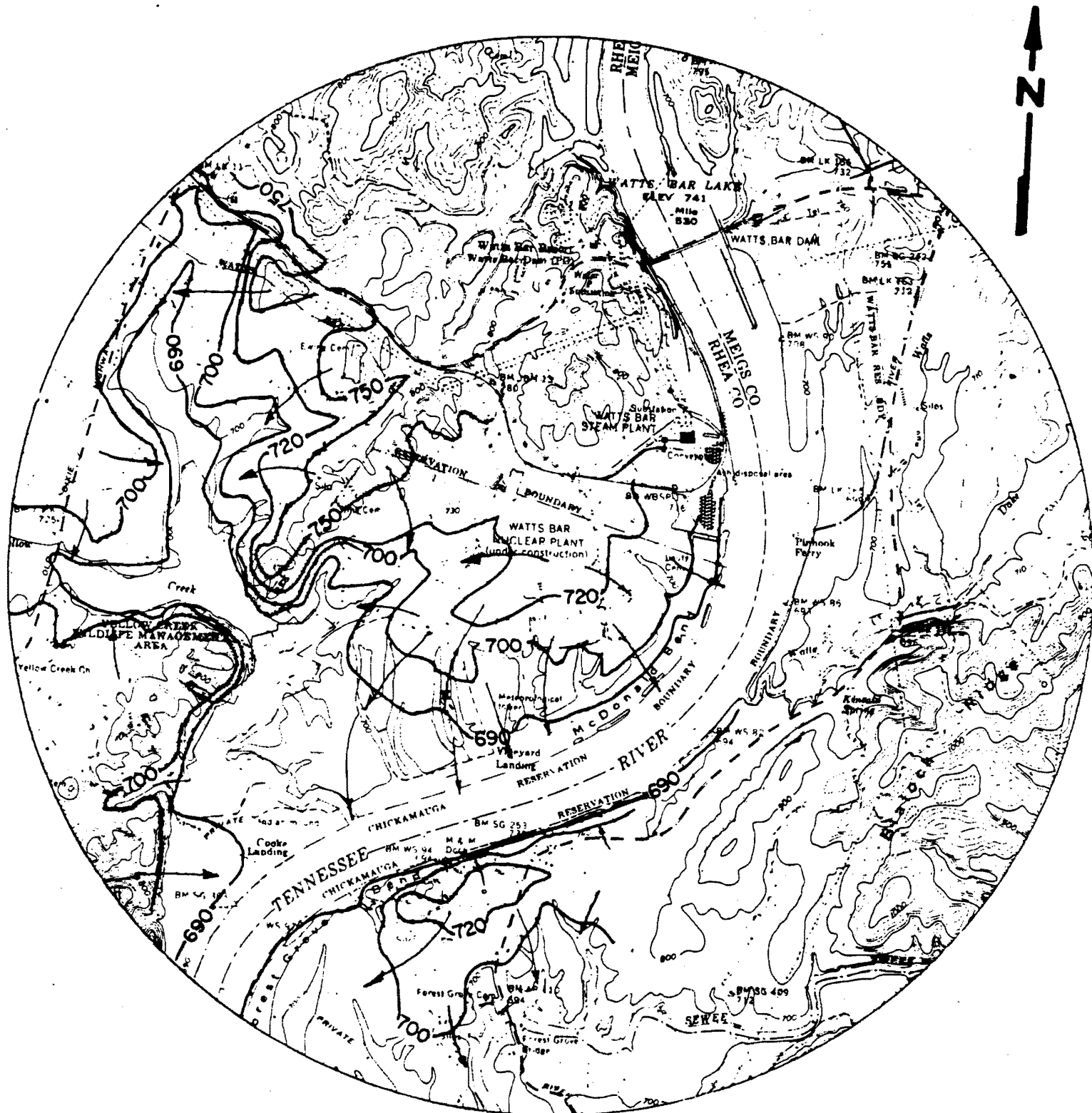
**SCALE:**



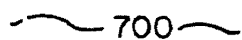

Revised by Amendment 50

<p><b>WATTS BAR NUCLEAR PLANT FINAL SAFETY ANALYSIS REPORT</b></p>
<p><b>LOCATIONS OF GROUND - WATER OBSERVATION WELLS FIGURE 2.4-104</b></p>

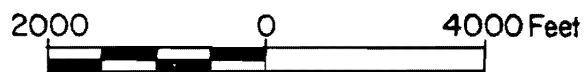
Figure 2.4-104 Locations of Ground - Water Observation Wells



EXPLANATION:

-  700 — Water table contour, in feet above mean sea level.
-  — General direction of ground-water movement.

SCALE:



Revised by Amendment 50

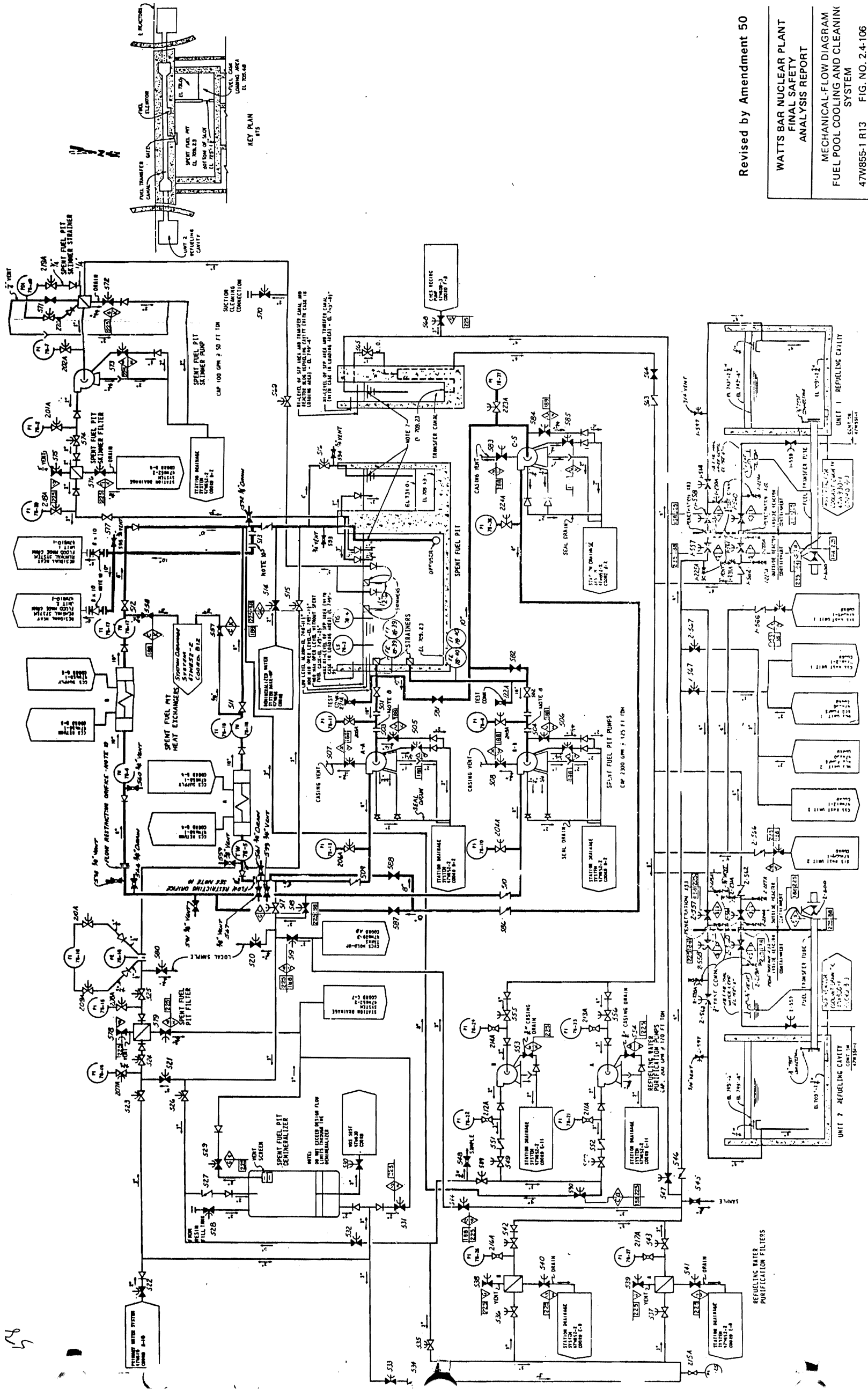
WATTS BAR NUCLEAR PLANT  
FINAL SAFETY  
ANALYSIS REPORT

GENERALIZED WATER-TABLE  
CONTOUR MAP

Figure 2.4-105

Figure 2.4-105 Generalized Water-Table Contour Map January 1972





Revised by Amendment 50

WATTS BAR NUCLEAR PLANT  
FINAL SAFETY  
ANALYSIS REPORT  
MECHANICAL-FLOW DIAGRAM  
FUEL POOL COOLING AND CLEANING  
SYSTEM  
47WB55-1 R13 FIG. NO. 2.4-106

Figure 2.4-106 Mechanical - Flow Diagram Fuel Pool Cooling and Cleaning System

DESIGN PRESSURE & TEMPERATURE DATA		
Line Design No.	Pressure (PSI)	Design Temperature (°F)
1	100	450
2	100	400
3	100	100

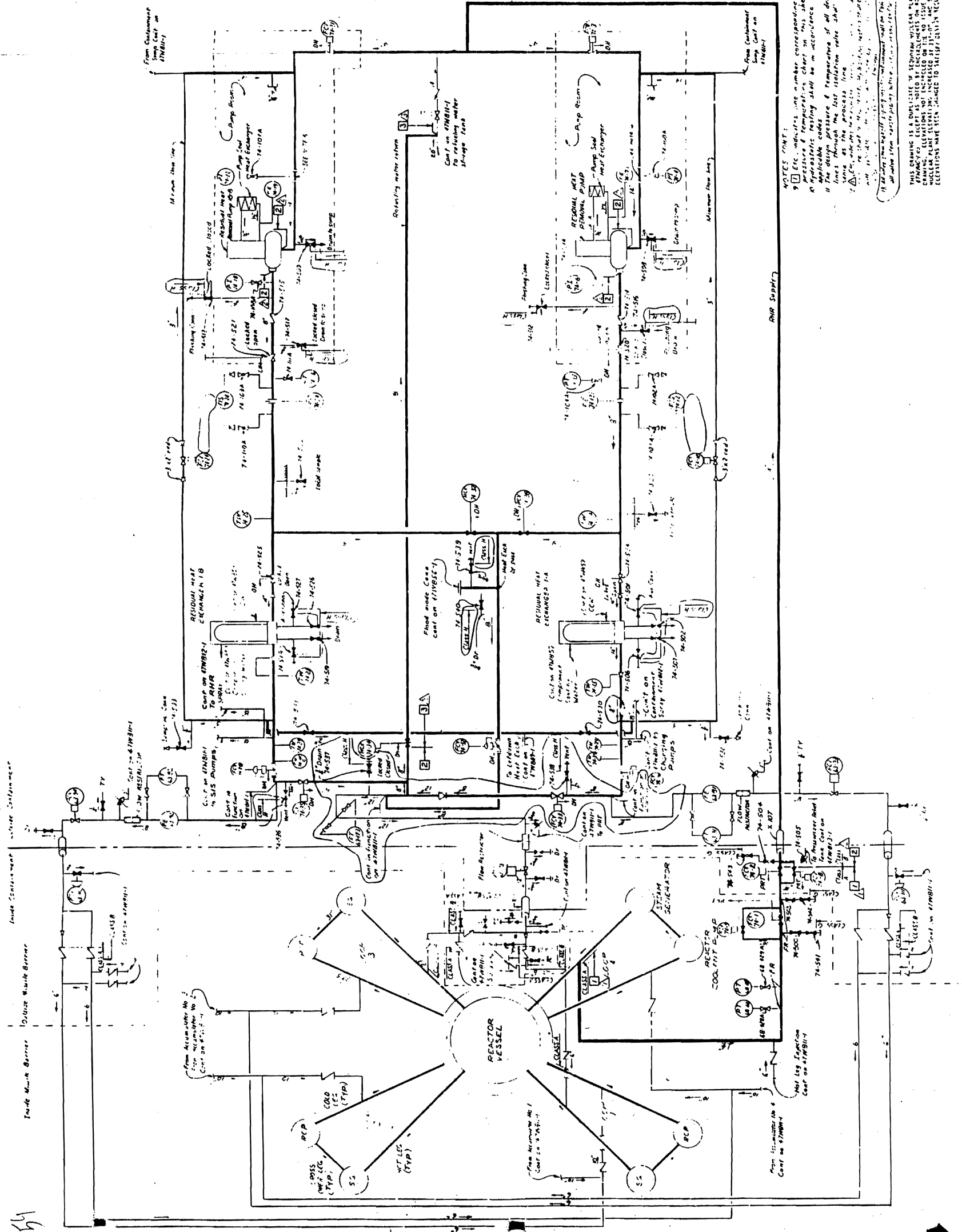
THIS DRAWING IS A DUAL COPY OF THE ORIGINAL DRAWING. THE ORIGINAL DRAWING IS THE ONLY COPY TO BE USED FOR CONSTRUCTION. THIS DRAWING IS NOT TO BE USED FOR CONSTRUCTION.

- NOTES:
1. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
  2. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
  3. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
  4. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
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  6. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
  7. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
  8. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
  9. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.

REFERENCE DRAWING:  
 1. WATTS BAR NUCLEAR PLANT SAFETY ANALYSIS REPORT  
 2. WATTS BAR NUCLEAR PLANT SAFETY ANALYSIS REPORT  
 3. WATTS BAR NUCLEAR PLANT SAFETY ANALYSIS REPORT  
 4. WATTS BAR NUCLEAR PLANT SAFETY ANALYSIS REPORT  
 5. WATTS BAR NUCLEAR PLANT SAFETY ANALYSIS REPORT  
 6. WATTS BAR NUCLEAR PLANT SAFETY ANALYSIS REPORT  
 7. WATTS BAR NUCLEAR PLANT SAFETY ANALYSIS REPORT  
 8. WATTS BAR NUCLEAR PLANT SAFETY ANALYSIS REPORT  
 9. WATTS BAR NUCLEAR PLANT SAFETY ANALYSIS REPORT

Revised by Amendment

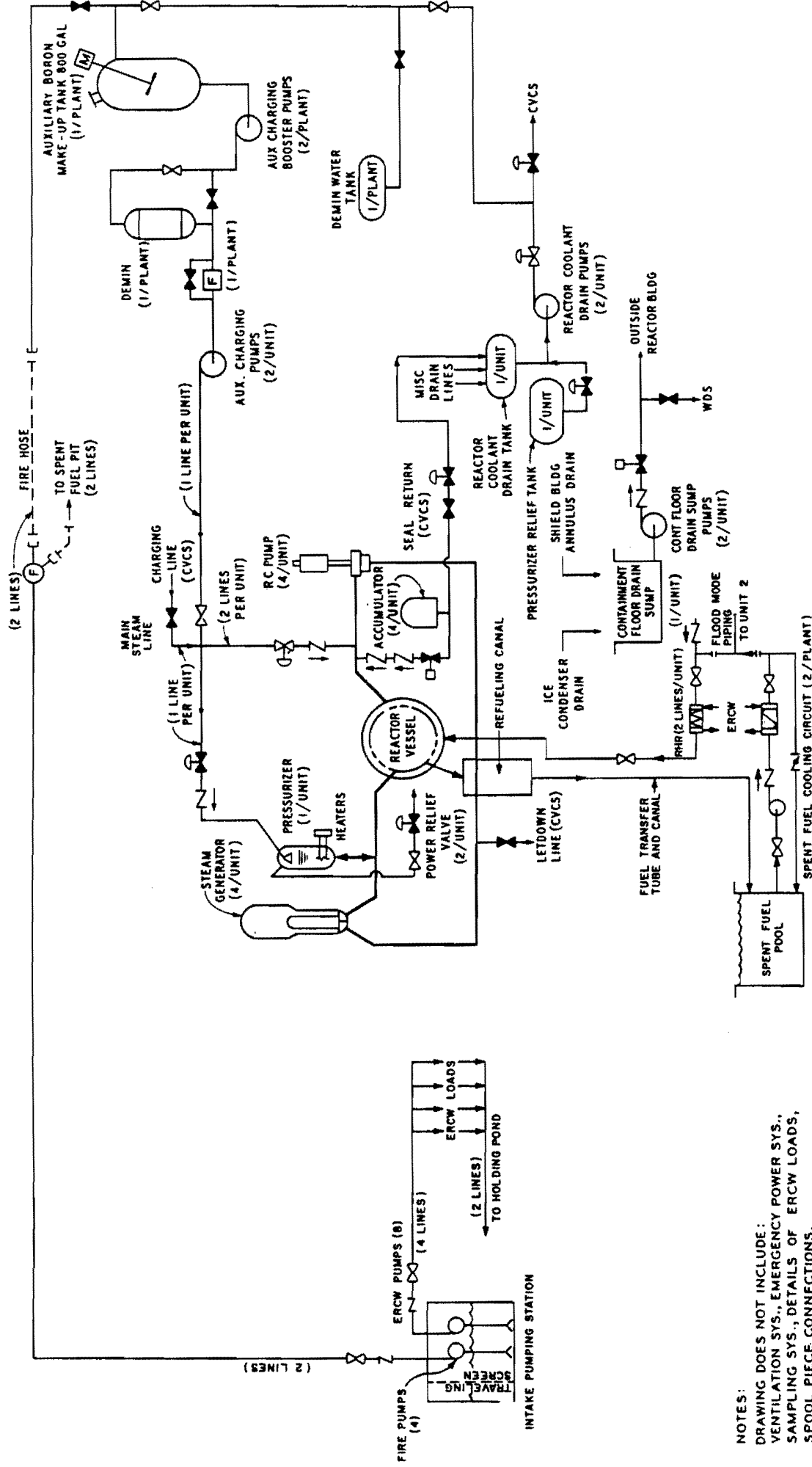
WATTS BAR NUCLEAR PLANT SAFETY ANALYSIS REPORT  
 POWERHOUSE UNITS 1 & 2  
 RESIDUAL HEAT REMOVAL SYSTEM  
 TVA DWG NO. 47W610-1 R16  
 FIGURE 2.4-107



NOTES CONT:

1. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
2. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
3. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
4. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
5. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
6. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
7. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
8. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.
9. ALL SYSTEMS ARE TO BE DESIGNED TO WITHSTAND THE DESIGN PRESSURE AND TEMPERATURE.

Figure 2.4-107 Powerhouse Units 1 & 2 Flow Diagram - Residual Heat Removal System

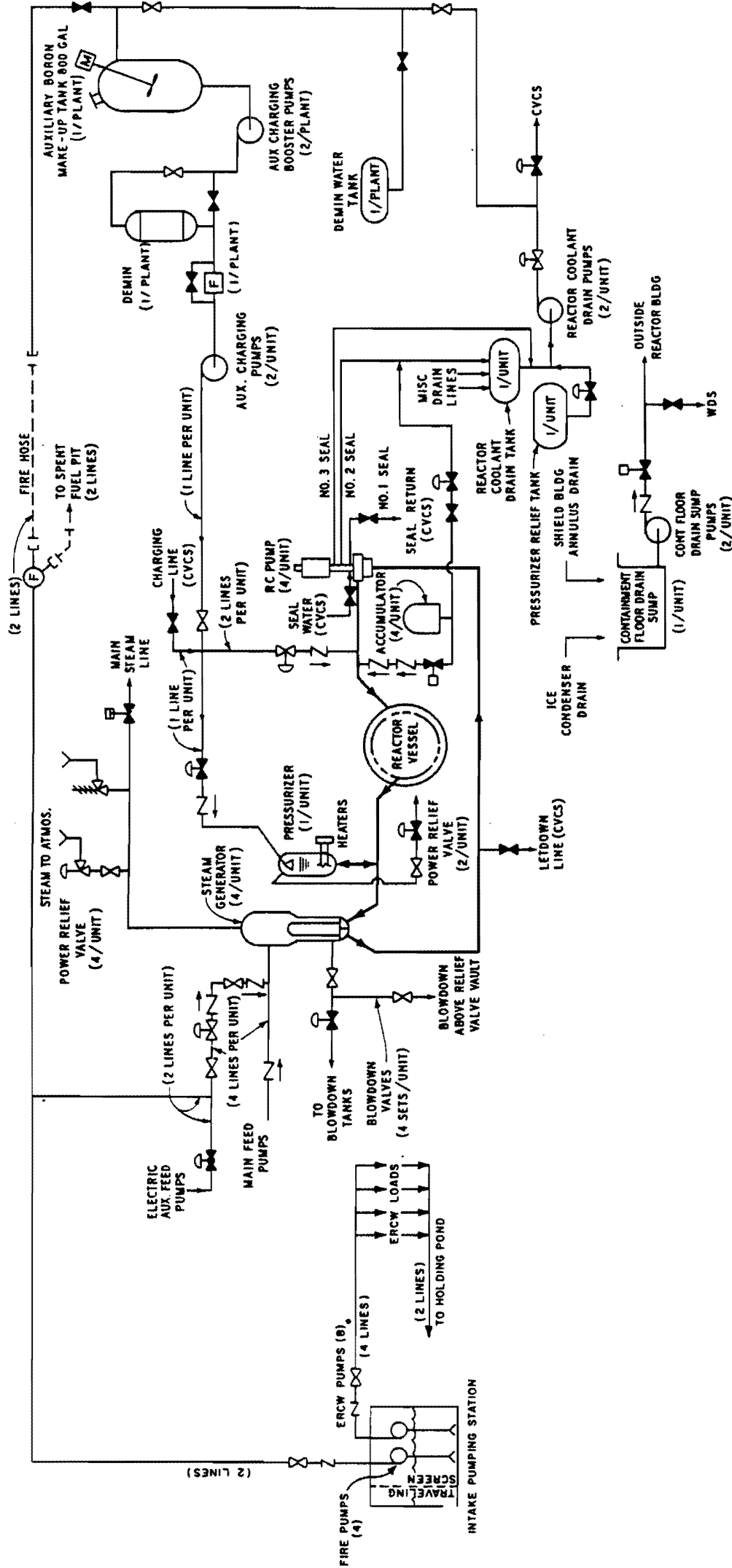


NOTES:  
 DRAWING DOES NOT INCLUDE:  
 VENTILATION SYS., EMERGENCY POWER SYS.,  
 SAMPLING SYS., DETAILS OF ERCW LOADS,  
 SPOOL PIECE CONNECTIONS.

WATTS BAR NUCLEAR PLANT  
 FINAL SAFETY  
 ANALYSIS REPORT

SCHEMATIC FLOW DIAGRAM  
 FLOOD PROTECTION PROVISIONS  
 OPEN REACTOR COOLING  
 (unit 1 shown, unit 2 similar)  
 Figure 2.4-108

Figure 2.4-108 Schematic Flow Diagram Flood Protection Provisions Open Reactor Cooling (Unit 1 Shown, Unit 2 Similar)

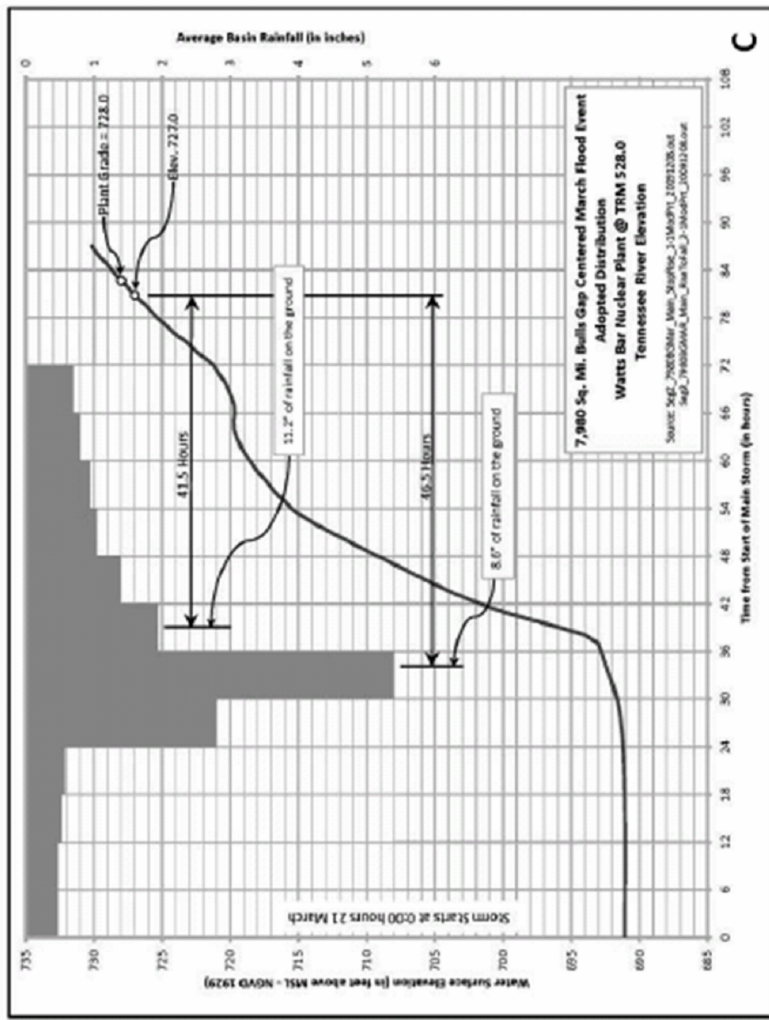


NOTES:  
 DRAWING DOES NOT INCLUDE:  
 VENTILATION SYS., EMERGENCY POWER SYS.,  
 SAMPLING SYS., DETAILS OF ERCW LOADS,  
 SPOOL PIECE CONNECTIONS.

WATTS BAR NUCLEAR PLANT  
 FINAL SAFETY  
 ANALYSIS REPORT

SCHEMATIC FLOW DIAGRAM  
 FLOOD PROTECTION PROVISIONS  
 NATURAL CONVECTION COOLING  
 (unit 1 shown, unit 2 similar)  
 Figure 2.4-109

Figure 2.4-109 Schematic Flow Diagram Flood Protection Provisions Natural Convection Cooling (Unit 1 Shown, Unit 2 Similar)



NOTE: Times shown allow 4 hours for communications and forecast computations.

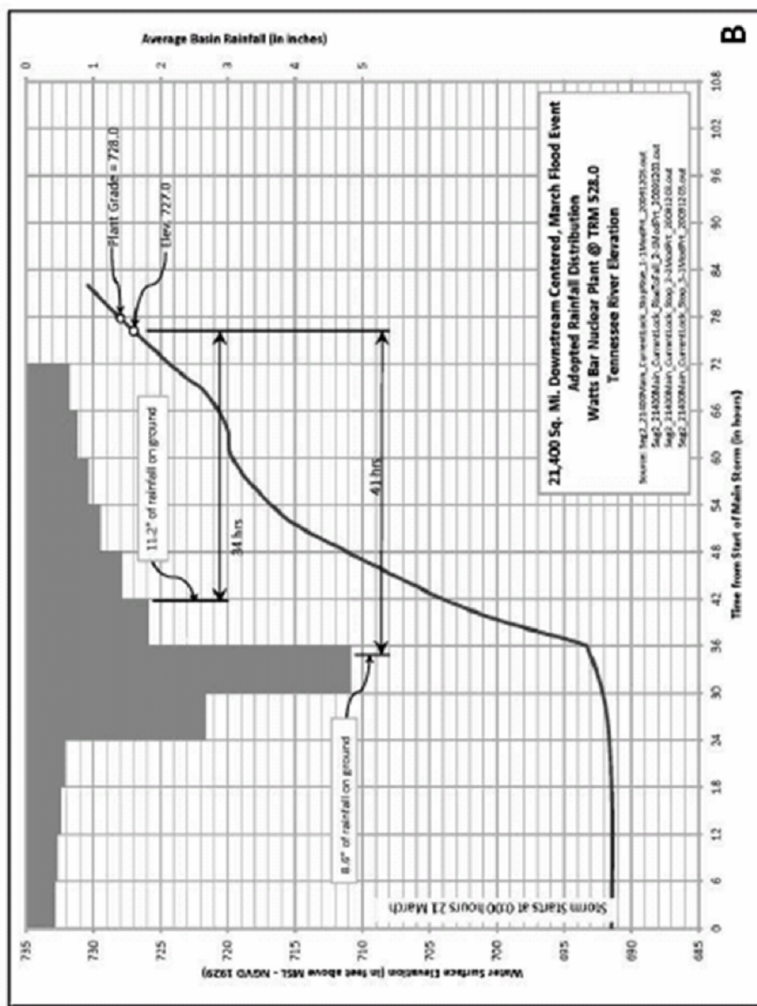
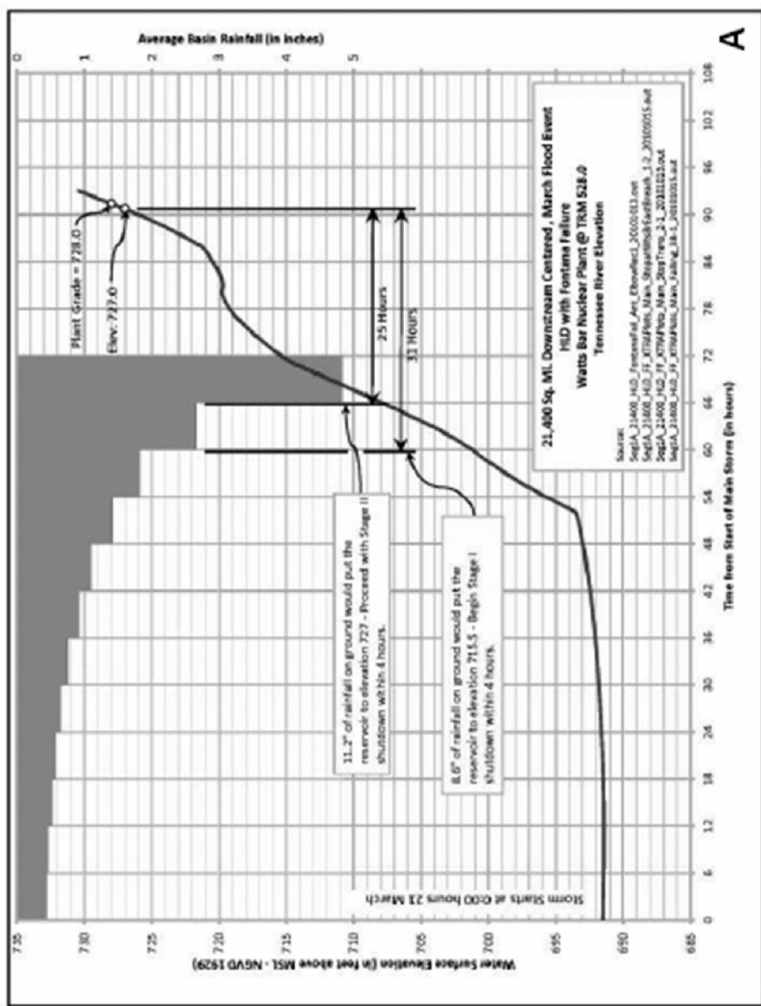
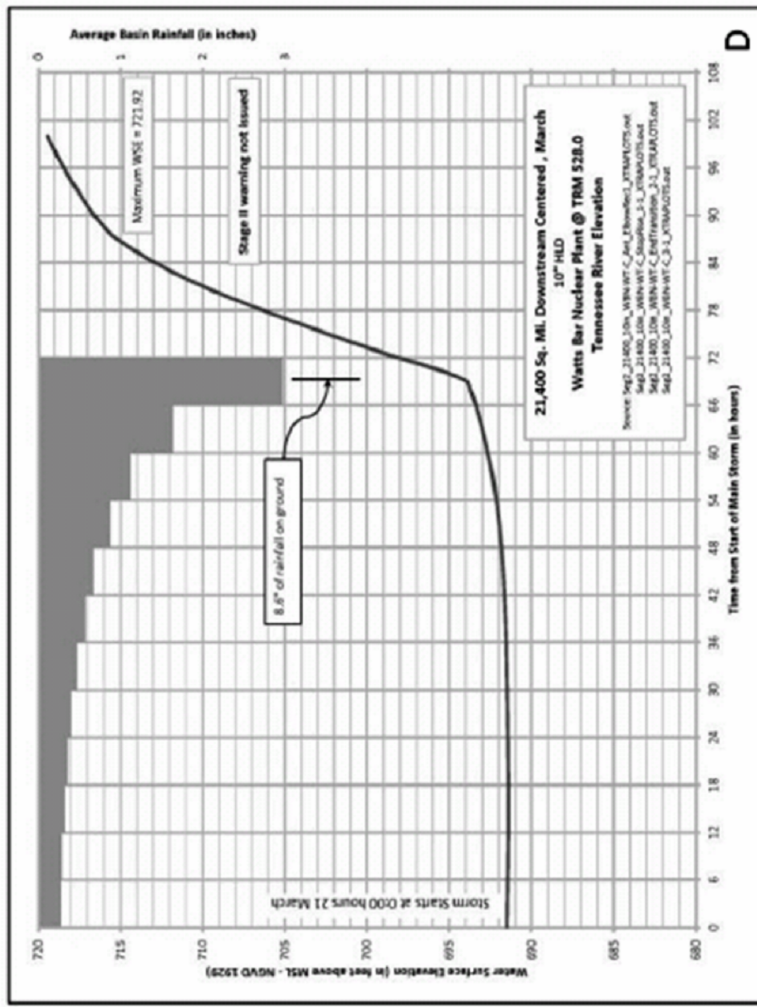
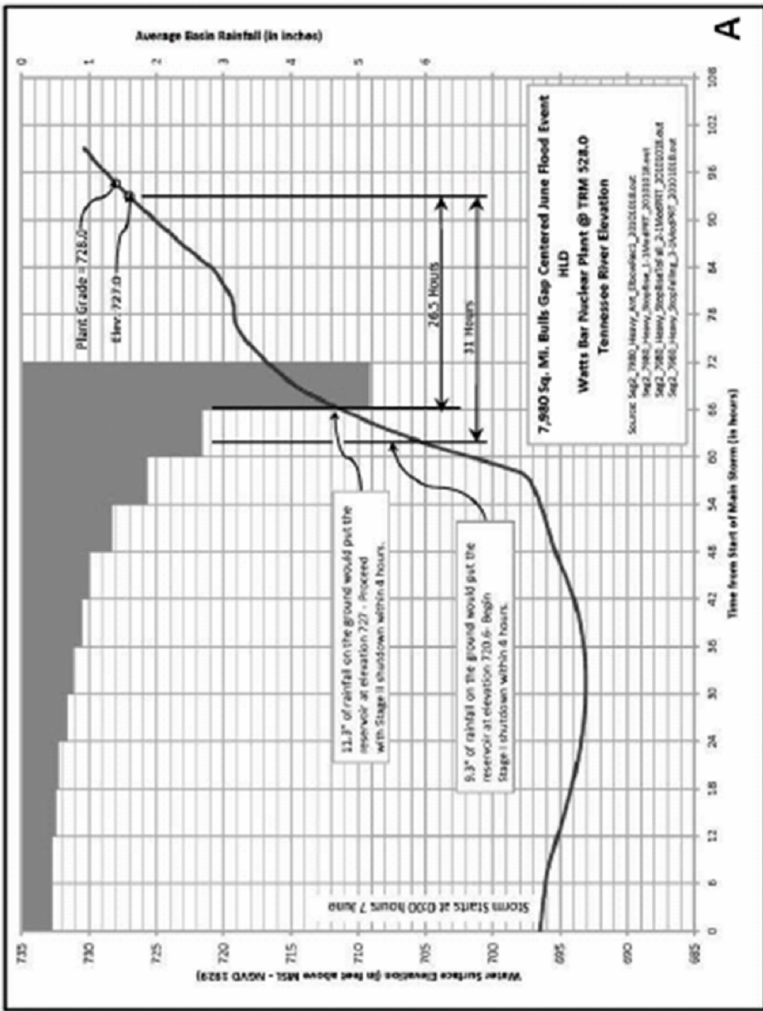
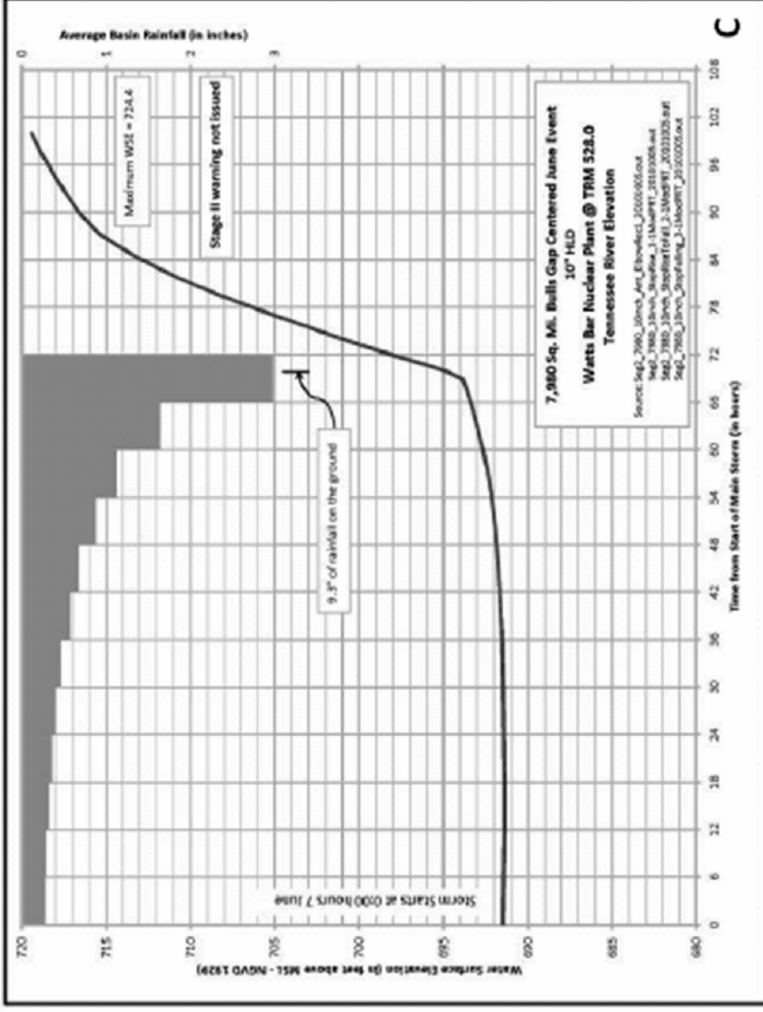


Figure 2.4-110 (Sheet 1) Watts Bar Nuclear Plant Rainfall Flood Warning Time Basis For Safe Shutdown For Plant Flooding - Winter Events



NOTE: Times shown allow 4 hours for communications and forecast computations.

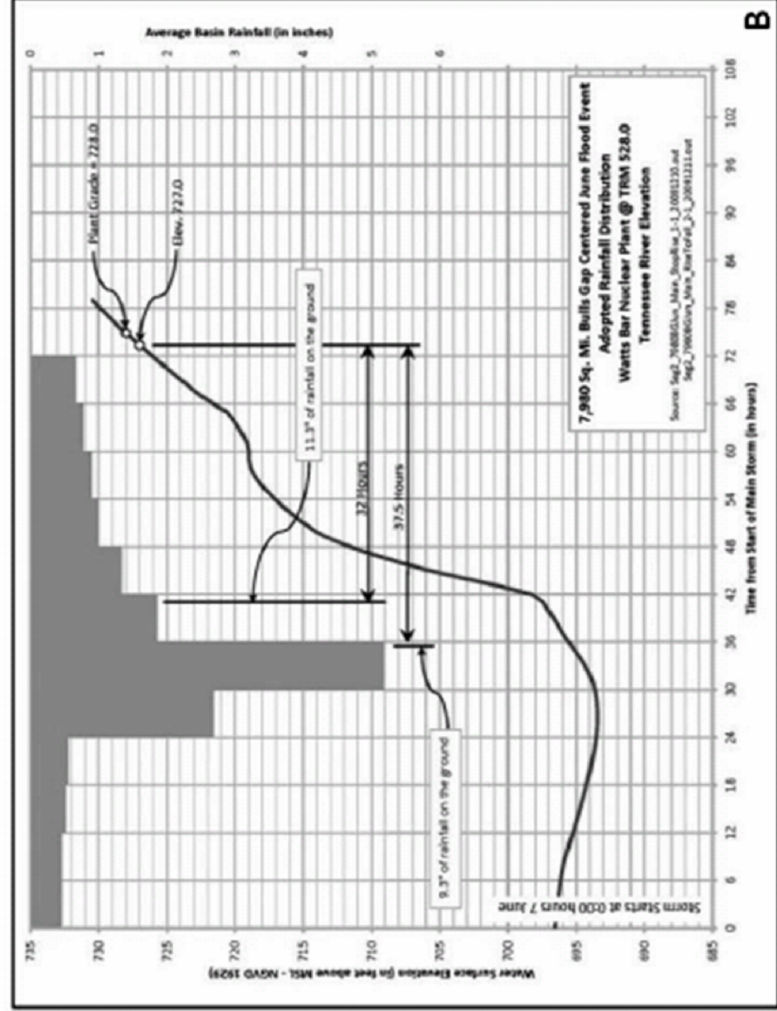


Figure 2.4-110 (Sheet 2) Watts Bar Nuclear Plant Rainfall Flood Warning Time Basis For Safe Shutdown For Plant Flooding - Summer Events

Figure 2.4-111 Deleted by Amendment 103

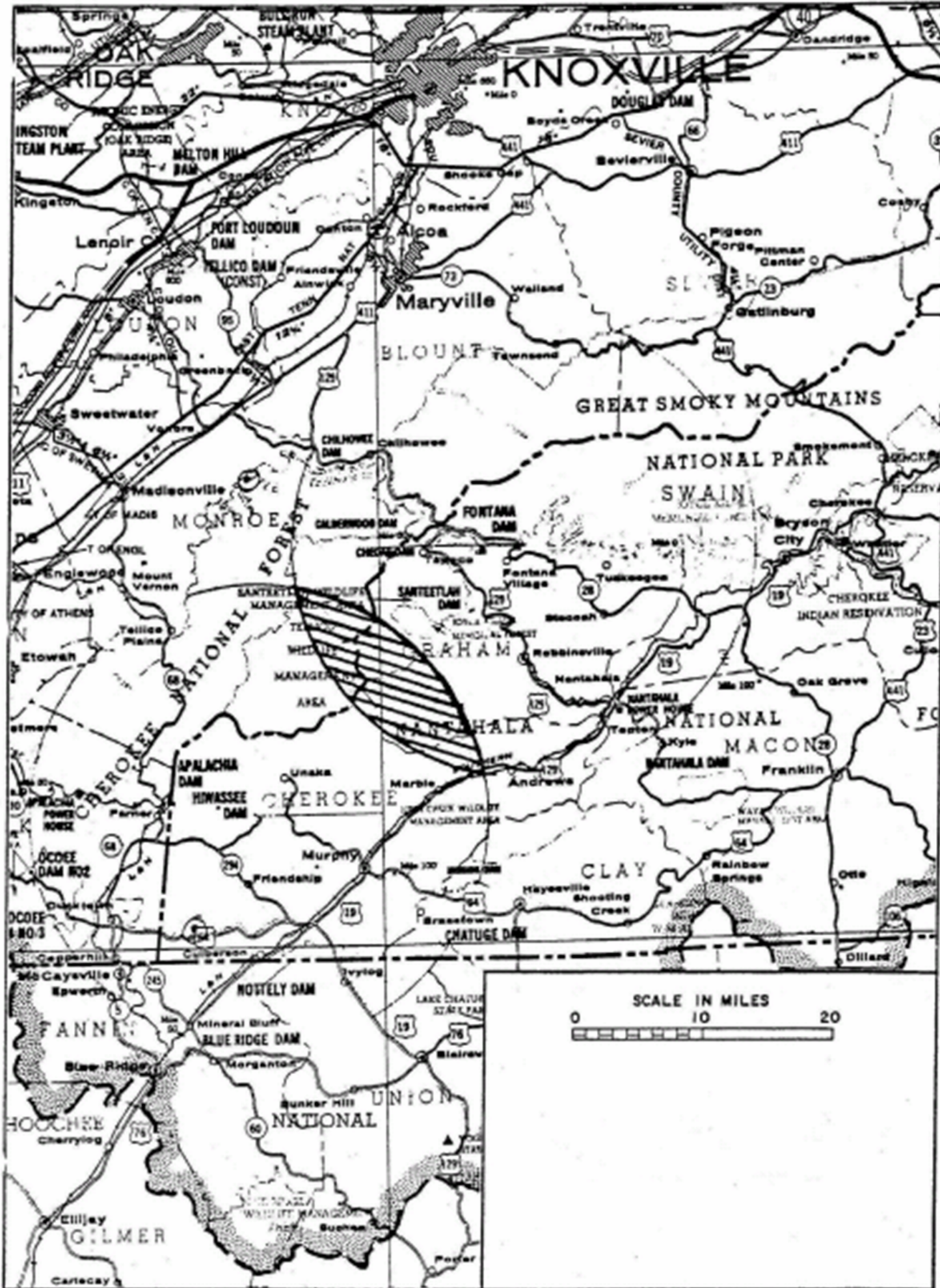


Figure 2.4-112 OBE with Epicenter Within Area Shown



Figure 2.4-113 Deleted by Amendment 107

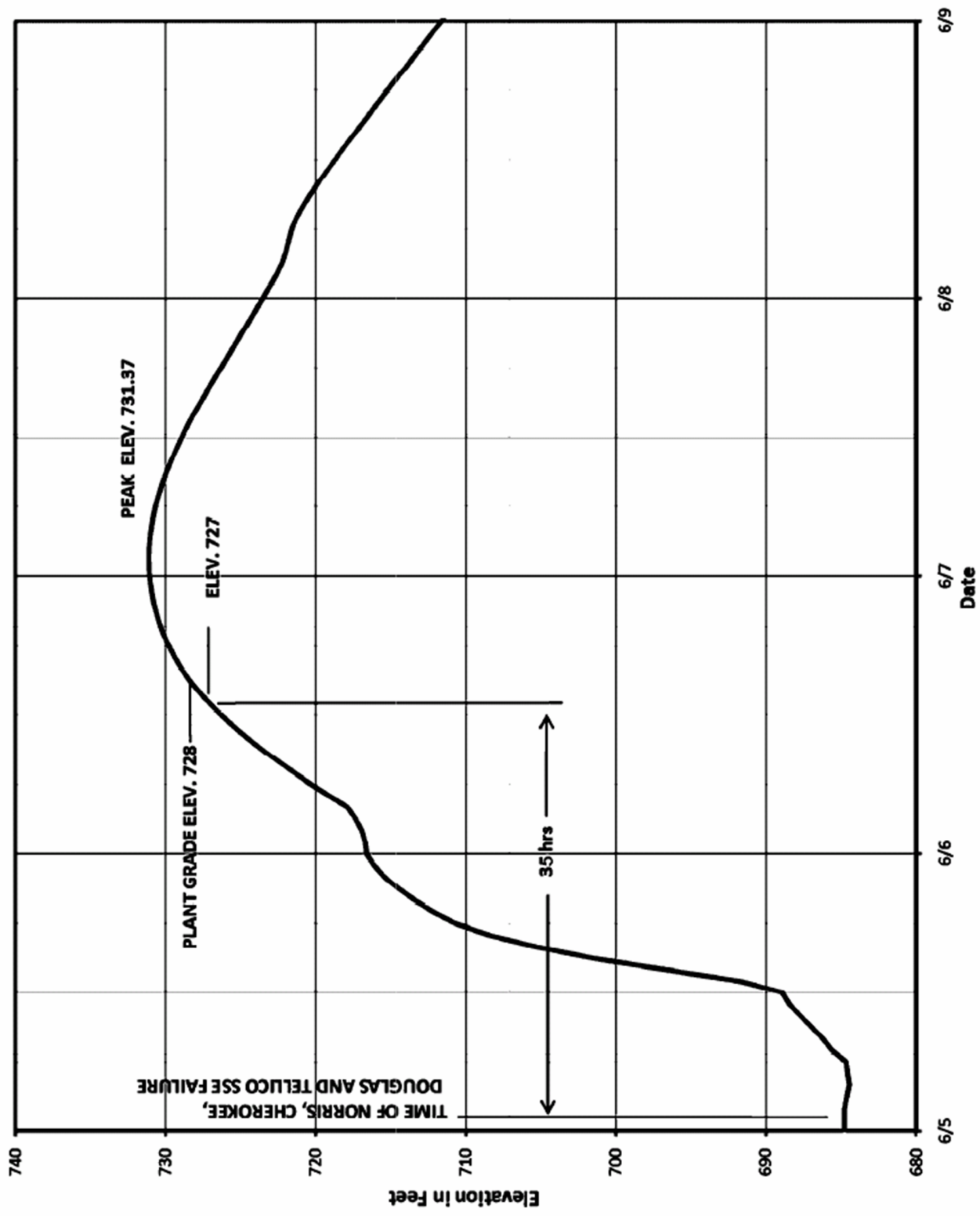


Figure 2.4-114 SSE Failure of Norris, Cherokee, Douglas, and Tellico Dams with 25-Year Flood Failure Wave at Watts Bar Nuclear Plant

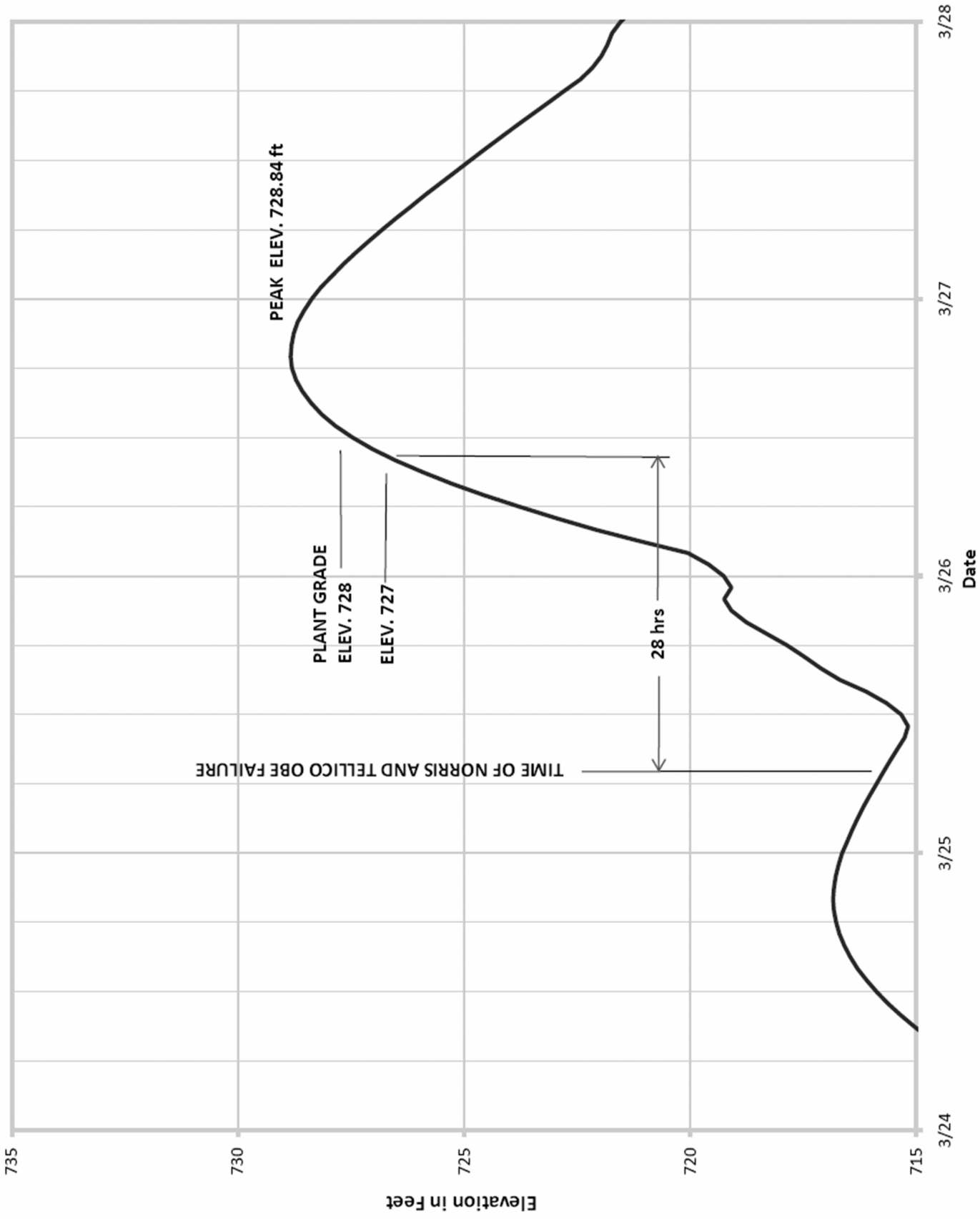


Figure 2.4-115 OBE Failure of Norris and Tellico Dams with 1/2 PMF Event Failure Wave at Watts Bar Nuclear Plant

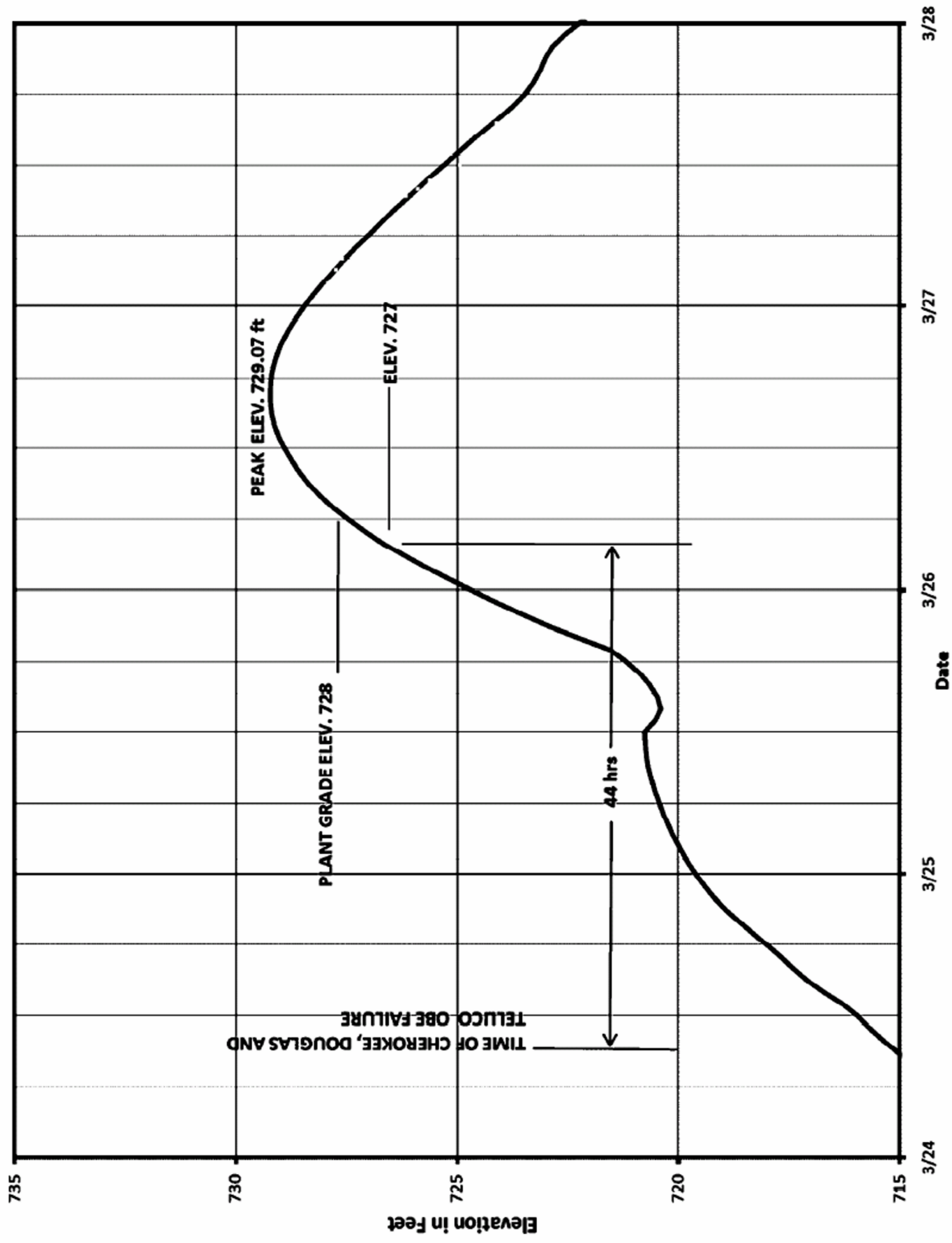


Figure 2.4-116 OBE Failure of Cherokee, Douglas and Tellico Dams with 1/2 PMF Event Failure Wave at Watts Bar Nuclear Plant