August 9, 2013

NRC 2013-0072
10 CFR 54

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

Point Beach Nuclear Plant, Unit 2
Docket 50-301
Renewed License No. DPR-27

10 CFR 50.55a Request, Relief Request RR-4L2
Inservice Inspection Impracticality
Examination Limitations Due to Configuration
Fourth Ten-Year Inservice Inspection Program Interval
Response to Request for Additional Information

References: (1) NextEra Energy Point Beach, LLC letter to NRC, dated March 19, 2013, 10 CFR 50.55a Request, Relief Request RR-4L2 Inservice Inspection Impracticality Examination Limitations Due to Configuration Fourth Ten-Year Inservice Inspection Program Interval (ML13079A141)

(2) NRC electronic mail to NextEra Energy Point Beach, LLC, dated July 3, 2012, Point Beach Nuclear Plant, Unit 2 – Draft RAI Regarding Relief Request RR-4L2 (TAC No. MF1146)

NextEra Energy Point Beach, LLC (NextEra) requested in Reference (1) that the Nuclear Regulatory Commission (NRC) grant relief from the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (B&PV Code), Section XI, 1998 Edition through 2000 Addenda requirement that the inner radius sections of all nozzles at terminal ends of piping runs be volumetrically examined. Relief is requested on the basis that alternative methods will provide an acceptable level of quality and safety.

Specifically, NextEra requested relief because the geometry of the Unit 2 steam generator main steam outlet nozzles nozzle design does not result in a true inner radius with the bored flow restrictor holes. Therefore, no meaningful examination could be performed.

Via Reference (2), the NRC determined additional information was required to enable the staff's continued review of the Relief Request RR-4L2. The Enclosure to this letter contains the response to the request for additional information in Reference (2).

This letter contains no new commitments and no changes to existing commitments.

Very truly yours,

NextEra Energy Point Beach, LLC

Larry Meyer
Site Vice President

Enclosure

cc: Administrator, Region III, USNRC
Project Manager, Point Beach Nuclear Plant, USNRC
Resident Inspector, Point Beach Nuclear Plant, USNRC
Mr. Mike Verhagan, Department of Commerce, State of Wisconsin

ENCLOSURE 1

NEXTERA ENERGY POINT BEACH, LLC
POINT BEACH NUCLEAR PLANT, UNIT 2

10 CFR 50.55a REQUEST, RELIEF REQUEST RR-4L2
INSERVICE INSPECTION IMPRACTICALITY
EXAMINATION LIMITATIONS DUE TO CONFIGURATION
FOURTH TEN-YEAR INSERVICE INSPECTION PROGRAM INTERVAL
RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION

The NRC staff determined that additional information was required (Reference 1) to enable the continued review of the Relief Request RR-4L2 (Reference 2). The following information is provided by NextEra Energy Point Beach, LLC (NextEra) in response to the NRC staff's request.

RAI

By letter dated March 19, 2013 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML13079A141), NextEra Energy Point Beach, LLC (the licensee) submitted relief request RR-4L2 covering a steam generator nozzle inner radius examination from the requirements of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code), Section XI, Rules for Inservice Inspection of Nuclear Power Plant Components for Point Beach Nuclear Power Plant, Unit 2. The request for relief applies to the fourth 10-year inservice inspection (ISI) interval, in which the licensee adopted the 1998 Edition through the 2000 Addenda of ASME Code Section XI as the code of record.

In accordance with 10 CFR 50.55a(g)(5)(iii), the licensee has submitted the subject request for relief for limited examination as the geometry of the component is not compatible with the ASME Code figures.

The NRC staff has reviewed and evaluated the information provided by the licensee, and determined that the following information is needed in order to complete its review:

1. *The drawing of the flow restrictor provided appears to show that the flow restrictor is constructed of clad ferritic steel, but this is not explicitly stated in the relief request. Please describe the materials of construction for the flow restrictor.*

Response

a. The Steam Nozzle Flow Limiter shown in Figure 4L2-1 is fabricated from the following components:

Steam Outlet Nozzle	SA-508 Class 3A
Venturi Inserts (7)	SB-564 (Inconel 690)

A 0.22" thick weld overlay of Inconel 52 was deposited on the Steam Outlet Nozzle. The Venturi Inserts are welded to this overlay with Inconel 52.

b. See below Drawing for additional details:

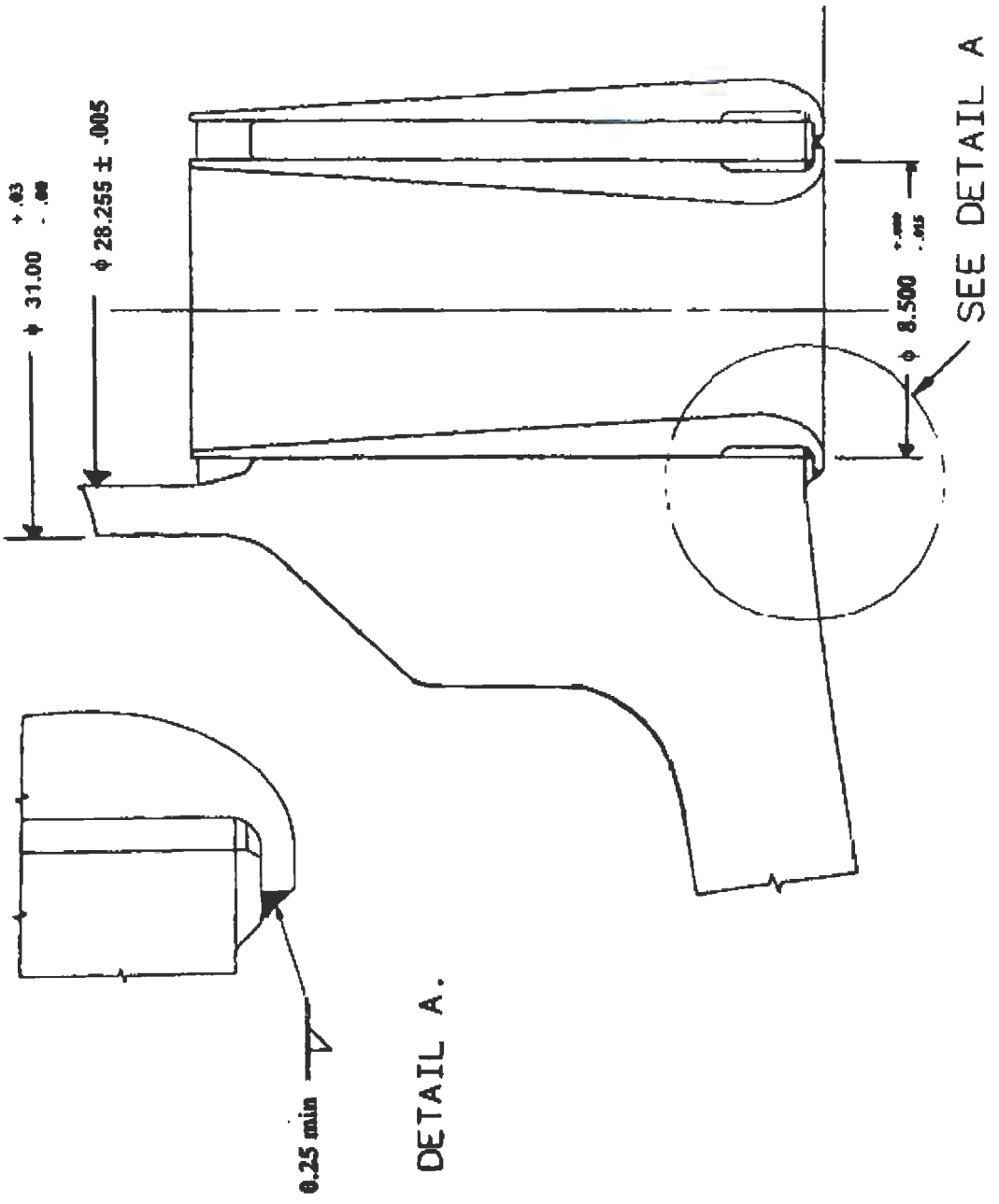

Figure 3-2. Steam Outlet Nozzle and Venturi Insert Assembly Details

References

(1) NRC electronic mail to NextEra Energy Point Beach, LLC, dated July 3, 2012, Point Beach Nuclear Plant, Unit 2 – Draft RAI Regarding Relief Request RR-4L2 (TAC No. MF1146)

(2) NextEra Energy Point Beach, LLC letter to NRC, dated March 19, 2013, 10 CFR 50.55a Request, Relief Request RR-4L2 Inservice Inspection Impracticality Examination Limitations Due to Configuration Fourth Ten-Year Inservice Inspection Program Interval (ML13079A141)