



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

August 8, 2013

Mr. David A. Heacock  
President and Chief Nuclear Officer  
Dominion Nuclear Connecticut, Inc.  
Innsbrook Technical Center  
5000 Dominion Boulevard  
Glen Allen, VA 23060-6711

SUBJECT: MILLSTONE POWER STATION, UNIT 3 - REQUEST FOR ADDITIONAL  
INFORMATION REGARDING AMENDMENT FOR A REVISED CALCULATED  
PEAK CONTAINMENT INTERNAL PRESSURE (TAC NO. MF1731)

Dear Mr. Heacock:

By letter dated April 25, 2013, Dominion Nuclear Connecticut, Inc. (the licensee), submitted a License Amendment Request to revise the calculated peak containment internal pressure ( $P_a$ ) for the Design Basis Loss-Of-Coolant Accident described in Technical Specification Section 6.8.4F, "Containment Leakage Rate Testing Program."

The U.S. Nuclear Regulatory Commission staff has reviewed the information provided by the licensee and has determined that the enclosed request for additional information (RAI) is needed in order to complete the review. A response to this RAI is requested to be provided within 45 days of the date of this letter.

If you have any questions regarding this matter, please contact me at 301-415-4125.

Sincerely,

A handwritten signature in cursive script that reads "James Kim".

James Kim, Project Manager  
Plant Licensing Branch I-1  
Division of Operating Reactor Licensing  
Office of Nuclear Reactor Regulation

Docket No. 50-423

Enclosure:  
As stated

cc w/encl: Distribution via Listserv

REQUEST FOR ADDITIONAL INFORMATION  
LICENSE AMENDMENT REQUEST  
FOR A REVISED CALCULATED PEAK CONTAINMENT INTERNAL PRESSURE  
DOMINION NUCLEAR CONNECTICUT, INC.  
MILLSTONE POWER STATION UNIT 3  
DOCKET NO. 50-423

1. Please verify that no change is needed in Technical Specification 3.6.1.4, "Containment Pressure," in order to maintain the calculated peak containment internal pressure ( $P_a$ ) below the design limit. Please indicate the containment operating pressure used as the initial condition for the Design-Basis Accident Loss-Of-Coolant-Accident analysis performed.
2. Please verify that this issue is not related to the recent EPITOME computer modeling errors discovered by Westinghouse.
3. Please state if the Appendix J Type B and C test procedures do not require revision upon approval of this proposed LAR. The American National Standards Institute (ANSI) 56.8-1994, Section 3.3.2 requires that Type B and C testing be performed at a pressure not less than  $P_a$  (except for airlock door seals, which may have a lower pressure specified) and not more than 1.1 times  $P_a$  when a higher differential pressure results in increased sealing. Please discuss the site procedures for Type B and C testing, and provide a discussion on the requirement that the testing be performed within a range of pressures that, with the revised  $P_a$ , will continue to be within the range of pressures required by ANSI 56.8-1994.
4. Please describe the steps that will be taken following approval of this amendment with regard to Appendix J Type A testing procedures.

Enclosure

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Dear Mr. Heacock:

By letter dated April 25, 2013, Dominion Nuclear Connecticut, Inc. (the licensee), submitted a License Amendment Request (LAR) to revise the calculated peak containment internal pressure (Pa) for the Design Basis Loss-Of-Coolant Accident (DBLOCA) described in TS Section 6.8.4F, "Containment Leakage Rate Testing Program."

The U.S. Nuclear Regulatory Commission staff has reviewed the information provided by the licensee and has determined that the enclosed request for additional information (RAI) is needed in order to complete the review. A response to this RAI is requested to be provided within 45 days of the date of this letter.

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*/ra/*

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**ADAMS Accession No.: ML1328A310**

\*via memo dated July 26, 2013

OFFICE	LPL1-1/PM	LPL1-1/LA	DSS/SCVB/BC	LPL1-1/BC
NAME	JKim	KGoldstein	RDennig*	RBeall(A)
DATE	8/8/13	8/7/13	7/26/13	8/8/13

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