

August 15, 2013

Mr. Yoshiki Ogata, General Manager
APWR Promoting Department
Mitsubishi Heavy Industries, Ltd.
16-05, Konan 2-Chome, Minato-Ku
Tokyo, 108-8215 Japan

SUBJECT: SUPPLEMENTAL SAFETY EVALUATION REPORT FOR MITSUBISHI HEAVY INDUSTRIES UNITED STATES – ADVANCED PRESSURIZED WATER REACTOR TOPICAL REPORT NUMBER PQD-HD-19005, REVISION 5, “QUALITY ASSURANCE PROGRAM DESCRIPTION FOR DESIGN CERTIFICATION OF THE UNITED STATES – ADVANCED PRESSURIZED WATER REACTOR”

Dear Mr. Ogata:

By letter dated January 26, 2007, as supplemented by letters dated March 7, 2007 and August 30, 2007, Mitsubishi Heavy Industries, LTD. (MHI), submitted Topical Report PQD-HD-19005, Revision 0, “Quality Assurance Program Description for Design Certification of the United States - Advanced Pressurized Water Reactor (US-APWR),” to the U.S. Nuclear Regulatory Commission (NRC) staff. On October 15, 2007, MHI responded to the NRC staff’s request for additional information and provided Revision 1 of Topical Report PDQ-HD-19005 (PDQ-HD-19005-1).

The NRC staff reviewed PDQ-HD-19005-1, and issued a safety evaluation, by letter dated January 24, 2008, from NRC to Mitsubishi Nuclear Energy Systems, Inc. (MNES) which concluded that PDQ-HD-19005-1 was acceptable for use by MHI for the US-APWR design certification (DC) activities to the extent specified and under the limitations delineated in the topical report.

On May 20, 2013, MHI submitted Revision 5 of Topical Report PDQ-HD-19005 (PDQ-HD-19005-5) to the NRC staff for review and approval. PDQ-HD-19005-5 contained minor revisions to reflect organizational changes within MNES which affected references to the organization used throughout the quality assurance program description. The NRC staff has reviewed this topical report and has found that PDQ-HD-19005-5 is acceptable for use by MHI for the US-APWR DC activities to the extent specified and under the limitations delineated in the topical report. The NRC staff confirmed that the changes incorporated into PDQ-HD-19005-5 were administrative in nature and do not constitute a reduction in the commitments made in the previous NRC-approved version, PDQ-HD-19005-4. Our acceptance applies only to material provided in the subject topical report, and not the US-APWR DC submittal. The NRC staff plans to issue a separate safety evaluation on Chapter 17 of the US-APWR DC application, and will reference, as necessary, this supplemental safety evaluation as it relates to the subject topical report.

Y. Ogata

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If you have any questions regarding this matter, I may be reached at (301) 415-0493 or via e-mail at Tarun.Roy@nrc.gov.

Sincerely,

/RA/

Tarun Roy, Project Manager
Licensing Branch 2
Division of New Reactor Licensing
Office of New Reactors

Docket No. 52-021

cc: See next page

Y. Ogata

-2-

If you have any questions regarding this matter, I may be reached at (301) 415-0493 or via e-mail at Tarun.Roy@nrc.gov.

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(Revised 07/25/2013)

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