



Prairie Island Nuclear Generating Plant
1717 Wakonade Drive East
Welch, MN 55089

July 25, 2013

L-PI-13-076
10 CFR 50.90

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Prairie Island Nuclear Generating Plant, Unit 2
Docket 50-306
Renewed License No. DPR-60

Supplement to License Amendment Request for Exception to Technical Specification
5.5.14 Testing Requirements Associated With Steam Generator Replacement (TAC No.
ME9141)

In a letter to the U.S. Nuclear Regulatory Commission (NRC) dated July 25, 2012 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML12207A523), the Northern States Power Company, a Minnesota corporation doing business as Xcel Energy (hereafter "NSPM"), submitted a license amendment request (LAR) to revise Technical Specification (TS) 5.5.14, "Containment Leakage Rate Testing," for Prairie Island Nuclear Generating Plant (PINGP), Unit 2. The proposed change would except PINGP Unit 2 from the requirements of Regulatory Guide (RG) 1.163 as specified in TS 5.5.14 for post-modification containment leakage rate testing associated with steam generator replacement. This exception is being requested to avoid performing an unnecessary integrated leakage rate test (ILRT).

As suggested by NRC Staff, NSPM is enclosing a revised TS markup to clarify that the proposed exception only applies to the upcoming Unit 2 steam generator replacements that are commencing in Fall 2013.

The enclosure to this letter provides the proposed TS change.

NSPM submits this information in accordance with the provisions of 10 CFR 50.90.

The supplemental information provided in this letter does not impact the conclusions of the Determination of No Significant Hazards Consideration and Environmental Assessment presented in the July 25, 2012 submittal.

A copy of this submittal is being forwarded to the designated State of Minnesota official pursuant to 10 CFR 50.91(b)(1).

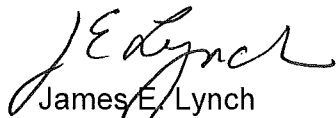
If there are any questions or if additional information is needed, please contact Glenn Adams at 612-330-6777.

Summary of Commitments

This letter contains no new commitments or revisions to existing commitments.

I declare under penalty of perjury that the foregoing is true and correct.

Executed on 07/25/2013



James E. Lynch
Site Vice President, Prairie Island Nuclear Generating Plant
Northern States Power Company – Minnesota

Enclosure

cc: Regional Administrator, Region III, USNRC
Project Manager, Prairie Island Nuclear Generating Plant, USNRC
Resident Inspector, Prairie Island Nuclear Generating Plant, USNRC
State of Minnesota

ENCLOSURE

Marked-Up Technical Specification Page

The enclosed markup illustrates the net changes for the proposed amendment, as applied to the renewed operating license (Unit 1 Amendment 206, Unit 2 Amendment 193).

1 page follows

5.0-28

5.5 Programs and Manuals (continued)

5.5.14 Containment Leakage Rate Testing Program

- a. A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995, as modified by the following exceptions:
1. Unit 1 and Unit 2 (steam generator (SG) replacement commencing Fall 2013) areis excepted from post-modification integrated leakage rate testing requirements associated with SGsteam generator replacement.
 2. Exception to NEI 94-01, "Industry Guideline for Implementing Performance-Based Option of 10 CFR 50, Appendix J", Section 9.2.3, to allow the following:
 - (i). The first Unit 1 Type A test performed after December 1, 1997 shall be performed by December 1, 2012.
 - (ii). The first Unit 2 Type A test performed after March 7, 1997 shall be performed by March 7, 2012.
- b. The peak calculated containment internal pressure for the design basis loss of coolant accident is less than the containment internal design pressure, P_a , of 46 psig.
- c. The maximum allowable primary containment leakage rate, L_a , at P_a , shall be 0.15% of primary containment air weight per day. For pipes connected to systems that are in the auxiliary building special ventilation zone, the total leakage shall be less than 0.06% of primary containment air weight per day at pressure P_a . For pipes connected to systems that are exterior to both the shield building and the auxiliary building special ventilation zone, the total leakage past isolation valves shall be less than 0.006% of primary containment air weight per day at pressure P_a .