



**UNITED STATES  
NUCLEAR REGULATORY COMMISSION**  
REGION II  
245 PEACHTREE CENTER AVENUE NE, SUITE 1200  
ATLANTA, GEORGIA 30303-1257

July 23, 2013

Mr. T. A. Lynch  
Vice President  
Southern Nuclear Operating Company, Inc.  
Joseph M. Farley Nuclear Plant  
P.O. Drawer 470, BIN B500  
Ashford, AL 36312

**SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT – NRC OPERATOR LICENSE  
EXAMINATION REPORT (05000348/2013301 AND 05000364/2013301)**

Dear Mr. Lynch:

During the period June 17 – 19, 2013, the Nuclear Regulatory Commission (NRC) administered operating tests to employees of your company who had applied for licenses to operate the Joseph M. Farley Nuclear Plant. At the conclusion of the tests, the examiners discussed preliminary findings related to the operating tests and the written examination submittal with those members of your staff identified in the enclosed report. The written examination was administered by your staff on June 25, 2013.

One Reactor Operator (RO) and one Senior Reactor Operator (SRO) applicant passed both the operating test and written examination. One SRO applicant failed the written examination. There were nine post-examination comments. These comments and the NRC resolution of these comments are summarized in Enclosure 2. A Simulator Fidelity Report is included in this report as Enclosure 3.

The initial examination submittal was within the range of acceptability expected for a proposed examination. All examination changes agreed upon between the NRC and your staff were made according to NUREG-1021, Operator Licensing Examination Standards for Power Reactors, Revision 9, Supplement 1.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosures will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of the NRC's document

system (ADAMS). ADAMS is accessible from the NRC Website at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room). If you have any questions concerning this letter, please contact me at (404) 997- 4550

Sincerely,

***/RA/***

Malcolm T. Widmann, Chief  
Operations Branch 1  
Division of Reactor Safety

Docket Nos: 50-348 and 50-364  
License Nos: NPF-2 and NPF-8

Enclosures:

1. Report Details
2. Post Exam Comments
3. Simulator Fidelity Report

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Enclosures:

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cc: (See page 3)

PUBLICLY AVAILABLE     
  NON-PUBLICLY AVAILABLE     
  SENSITIVE     
  NON-SENSITIVE  
 ADAMS:  Yes   
 ACCESSION NUMBER: ML13205A329   
  SUNSI REVIEW COMPLETE   
  FORM 665 ATTACHED

OFFICE	RII:DRS	RII:DRS					
SIGNATURE	RA VIA EMAIL						
NAME	CAPEHART	WIDMANN					
DATE	7/15/2013	7/ /2013	7/ /2013	7/ /2013	7/ /2013	7/ /2013	7/ /2013
E-MAIL COPY?	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO	YES NO

cc:

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Letter to T. A. Lynch from Malcolm T. Widmann dated July 23, 2013

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT – NRC OPERATOR LICENSE  
EXAMINATION REPORT (05000348/2013301 AND 05000364/2013301)

**DISTRIBUTION:**

C. Evans, RII EICS

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**U.S. NUCLEAR REGULATORY COMMISSION**

REGION II

Docket No.: 05000348, 05000364

License Nos.: NPF-2, NPF-9

Report No.: 05000348/2013301 AND 05000364/2013301

Licensee: Southern Nuclear Company (SNC), LLC

Facility: Joseph M. Farley Nuclear Plant

Location: Ashford, AL

Dates: Operating Test – June 17 – 19, 2013  
Written Examination – June 25, 2013

Examiners: P. Capehart, Chief Examiner, Senior Operations Engineer  
D. Lanyi, Operations Engineer  
S. Garchow, Senior Operations Engineer  
M. Donithan, Operations Engineer (Under Instruction)

Approved by: Malcolm T. Widmann, Chief  
Operations Branch 1  
Division of Reactor Safety

## **SUMMARY OF FINDINGS**

ER 05000348/2013301 and 05000364/2013301; June 17 – 19, 2013, written exam June 25, 2013; Joseph M. Farley Nuclear Plant; Units 1 and 2 Operator License Examinations.

Nuclear Regulatory Commission (NRC) examiners conducted an initial examination in accordance with the guidelines in Revision 9, Supplement 1, of NUREG-1021, "Operator Licensing Examination Standards for Power Reactors." This examination implemented the operator licensing requirements identified in 10 CFR §55.41, §55.43, and §55.45, as applicable.

Members of the Joseph M. Farley Nuclear Plant staff developed both the operating tests and the written examination.

The NRC administered the operating tests during the period June 17 – 19, 2013. Members of the Joseph M. Farley Nuclear Plant training staff administered the written examination on June 25, 2013. One Reactor Operator (RO) and one Senior Reactor Operator (SRO) applicant passed both the operating test and written examination. One SRO applicant failed the written examination. Two applicants were issued licenses commensurate with the level of examination administered.

There were nine post-examination comments.

No findings were identified.



## REPORT DETAILS

### 4. OTHER ACTIVITIES

#### a. Inspection Scope

Members of the Joseph M. Farley Nuclear Plant staff developed both the operating tests and the written examination. All examination material was developed in accordance with the guidelines contained in Revision 9, Supplement 1, of NUREG-1021, "Operator Licensing Examination Standards for Power Reactors." The NRC examination team reviewed the proposed examination. Examination changes agreed upon between the NRC and the licensee were made per NUREG-1021 and incorporated into the final version of the examination materials.

The NRC reviewed the licensee's examination security measures while preparing and administering the examinations in order to ensure compliance with 10 CFR §55.49, "Integrity of examinations and tests."

The NRC examiners evaluated one Reactor Operator (RO) and two Senior Reactor Operator (SRO) applicants using the guidelines contained in NUREG-1021. The examiners administered the operating tests during the period June 17 – 19, 2013. Members of the Joseph M. Farley Nuclear Plant training staff administered the written examination on June 25, 2013. Evaluations of applicants and reviews of associated documentation were performed to determine if the applicants, who applied for licenses to operate the Joseph M. Farley Nuclear Plant, met the requirements specified in 10 CFR Part 55, "Operators' Licenses."

#### b. Findings

No findings were identified. The NRC determined, using NUREG-1021, that the licensee's initial examination submittal was within the range of acceptability expected for a proposed examination.

One RO applicant and one SRO applicant passed both the operating test and written examination. One SRO applicant passed the operating test but did not pass the written examination. One RO applicant and one SRO applicant were issued licenses.

Copies of all individual examination reports were sent to the facility Training Manager for evaluation of weaknesses and determination of appropriate remedial training.

The licensee submitted nine post-examination comments. A copy of the final written examination and answer key, with all changes incorporated, may be accessed not

earlier than June 26, 2015, in the ADAMS system (ADAMS Accession Number(s) ML13191A886 and ML13191A891).

4OA6 Meetings, Including Exit

Exit Meeting Summary

On June 19, 2013, the NRC Chief Examiner discussed the operating test with Mr. Tom Lynch, Site Vice President, and members of the Joseph M. Farley Nuclear Plant staff. The examiners asked the licensee if any of the examination material was proprietary. No proprietary information was identified.

## KEY POINTS OF CONTACT

### Licensee personnel

Brian Reed, Operations Training Supervisor  
Gary Ohmstede, Lead Instructor – Exam Development  
Aaron Forsha, Operations Instructor  
Stanley Jackson, Operations Instructor  
Billy Thornton, Operations Instructor  
Chris Thornell, Operations Director  
David Reed, Operations Support Manager  
Barbara Taylor, Performance Improvement Supervisor  
Rick Vaught, Maintenance Director

### NRC personnel

Newton Lacy, Operations Engineer (Trainee)  
David Lanyi, Operations Engineer

## Facility Comments and NRC Resolution

A complete text of the licensee's post-examination comments can be found in ADAMS under Accession Number ML13191A894.

### Question # 32, 039K1.09

#### Comment:

A "minimum" could be confusing as the minimum power level that the R-70s indicate in gpd is 20%.

#### Facility Comment:

This does not change the correct answer. Will evaluate clarity of the stem and change exam bank if necessary.

#### NRC Resolution:

No change required.

### Question # 33, 045K1.19

**Comment:** Distracter A would also be correct if power is less than 40% for less than 260 seconds.

#### Facility Comment:

Appendix E applies in this situation. The applicant is creating unstated conditions and applying them to the question. Will evaluate clarity of the stem and change exam bank if necessary.

#### NRC Resolution:

No change required.

### Question # 40, 059G2.2.44

#### Comment:

Later in the procedure, if SFGP suction pressure is not stabilized above 300 psig, the procedure directs the reduction in turbine load. It does not disallow the use of AOP-17.1.

#### Facility Comment:

This does not change the correct answer. ARP-KB4 directs the starting of the standby condensate pump and is an entry condition for AOP-13. AOP-13 is entered at Step 6 and Step 6.1 checks that the SFGP suction pressure stabilizes above 275 psig or directs the operator to perform a rapid load reduction. The check of SFGP suction pressure > 300 psig is well past

time 30 seconds in the question statement at Step 6.6. Since the stem indicates that the SGFP suction pressure stabilizes at Step 6.1, a load reduction is NOT required at time 30 seconds. Will evaluate clarity of the stem and change exam bank if necessary.

**NRC Resolution:**

No change required.

**Question # 47, 064K6.07**

**Comment:**

The 2<sup>nd</sup> question statement should use the work “maximum” instead of “minimum”.

**Facility Comment:**

This does not change the correct answer. Will evaluate clarity of the stem and change exam bank if necessary.

**NRC Resolution:**

No change required.

**Question # 61, G2.1.5**

**Comment:**

The Fire Protection Administrator (FPA) is not designated as the Shift Communicator in the stem of the question. NRC Chief Examiner allowed the licensee to tell all applicants that the FPA is designated as the Shift Communicator for the shift.

**Facility Comment:**

The stem of the question will be modified to ensure that the FPA is clearly designated as the Shift Communicator for the shift.

**NRC Resolution:**

No change required.

**Question # 79, 008AG2.2.22**

**Comment:**

A mode change is allowed after a risk assessment is performed.

**Facility Comment:**

Appendix E applies as a risk evaluation is not discussed in the stem.

**NRC Resolution:**

No change required.

**Question # 83, 051AG2.1.7**

**Comment:**

Actions in AOP-8.9 will not be performed due to AOP-3 being the priority procedure.

**Facility Comment:**

Both AOP-8.0 and AOP-3.0 are still in effect at the stated time of the question.

**NRC Resolution:**

No change required.

**Question # 85, 055EG2.2.12**

**Comment:**

There is not enough information to determine the status of offsite power operability.

**Facility Comment:**

Appendix E applies as the stem clearly states that offsite power is available and there is no information provided that would cause the applicant to question the operability of offsite power.

**NRC Resolution:**

No change required.

**Question # 88, 064G2.2.44**

**Comment:**

Applicant states the LCO 3.8.9 should be entered and not LCO 3.8.1.

**Facility Comment:**

LCO 3.8.1 and 3.8.9 are required to be entered. The question asks whether or not a required action statement of LCO 3.8.1 is required to be implemented.

**NRC Resolution:**

No change required.

## **SIMULATOR FIDELITY REPORT**

Facility Licensee: Joseph M. Farley Nuclear Plant

Facility Docket Nos.: 05000348, 05000364

Operating Test Administered: June 17 – 19, 2013

This form is to be used only to report observations. These observations do not constitute audit or inspection findings and, without further verification and review in accordance with Inspection Procedure 71111.11 are not indicative of noncompliance with 10 CFR 55.46. No licensee action is required in response to these observations.

No simulator fidelity or configuration issues were identified.