

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

August 6, 2013

Mr. Kevin Walsh, Site Vice President c/o Michael Ossing Seabrook Station NextEra Energy Seabrook, LLC P.O. Box 300 Seabrook, NH 03874

SUBJECT:

SEABROOK STATION, UNIT 1 - REQUEST FOR ADDITIONAL INFORMATION

FOR THE 2012 STEAM GENERATOR TUBE INSPECTIONS

(TAC NO. MF0940)

Dear Mr. Walsh:

By letter dated December 31, 2012 (Agencywide Documents Access and Management System Accession No. ML13008A160), NextEra Energy Seabrook, LLC, submitted its 2012 Steam Generator Tube Inspections, performed during the 15th refueling outage.

The U.S. Nuclear Regulatory Commission staff has been reviewing the submittal dated December 31, 2012, and determined that additional information is necessary to complete its review. The draft questions were sent via electronic transmission on July 17, 2013, to Mr. Michael Ossing, of your staff. The draft questions were sent to ensure that the questions were understandable, the regulatory basis for the questions was clear, and to determine if the information was previously docketed. On July 23, 2013, Mr. Ossing indicated that the licensee will submit a response within 45 days of the date of this letter. The request for additional information is enclosed.

If you have questions, you can contact me at 301-415-3100 or at John.Lamb@nrc.gov.

Sincerely

Joha G. Lamb, Senior Project Manager

Plant Licensing Branch I-2

Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket No. 50-443

Enclosure:

Request for Additional Information

cc w/ enclosure: Distribution via ListServ

REQUEST FOR ADDITIONAL INFORMATION

2012 STEAM GENERATOR TUBE INSPECTIONS

NEXTERA ENERGY SEABROOK, LLC.

SEABROOK STATION, UNIT 1

DOCKET NO. 50-443

By letter dated December 31, 2012 (Agencywide Documents Access and Management System Accession No. ML13008A160), NextEra Energy Seabrook, LLC submitted its 2012 Steam Generator (SG) Tube Inspections, performed during the 15th refueling outage (RFO) at Seabrook Station, Unit 1 (Seabrook).

The U.S. Nuclear Regulatory Commission staff has determined that the following additional information is necessary to complete its review.

REQUEST FOR ADDITIONAL INFORMATION

- 1. Provide the effective full-power years of SG operation for the last three RFOs.
- 2. In the Seabrook Final Safety Analysis Report (FSAR), the Main Steam Line Break accident (Section 15.1) assumes leakage of 500 gallons per day (gpd) through the faulted SG, and 940 gpd through the remaining three SGs, for a total leakage of 1440 gpd, which is also equal to 1 gallon per minute (gpm).

Section 6.7.6.k.b.2. of the Seabrook Technical Specifications (TSs) states, in part, "[t]he primary to secondary accident induced leakage rate for any design basis accident, other than a SG tube rupture, shall not exceed the leakage rate assumed in the accident analysis in terms of total leakage rate for all SGs and leakage rate for an individual SG. Leakage is not to exceed 1 gpm total or 500 gpd through any one SG." [emphasis added]

Section 9.0 of the letter dated December 31, 2012, states, in part, "[b]ecause the predicted accident induced leakage from *any* of the Seabrook SGs is less [than] 1 gpm, the leakage criteria for condition monitoring are met." [emphasis added]

a. Please clarify the sentence in the letter dated December 31, 2012, given the wording in the TSs and FSAR. Please confirm that the Accident Induced Leakage Performance Criteria were met during the prior cycle.

August 6, 2013

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Sincerely.

/ra/

John G. Lamb, Senior Project Manager Plant Licensing Branch I-2 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

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**via memo

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