UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 18, 2013

Mr. Joseph E. Pacher
Vice President R.E. Ginna Nuclear Power Plant
R.E. Ginna Nuclear Power Plant, LLC
1503 Lake Road
Ontario, NY 14519

SUBJECT: R.E. GINNA NUCLEAR POWER PLANT - RE: CORRECTION TO AMENDMENT APPROVING THE USE OF ALTERNATE CONTROL AND SHUTDOWN ROD POSITION MONITORING SYSTEM (TAC NO. ME8514)

Dear Mr. Pacher:

By letter dated April 20, 2012, R.E. Ginna Nuclear Power Plant, LLC, (licensee) submitted to U.S. Nuclear Regulatory Commission (NRC) a license amendment request to the R.E. Ginna Nuclear Power Plant (Ginna) Renewed Facility Operating License No. DPR-18. On June 25, 2013, the NRC staff issued Amendment No. 114 to Renewed Facility Operating License No. DPR-18, for Ginna.

The Amendment revised Technical Specification (TS) 3.1.7 to approve the use of an alternative method, other than the current method using the movable incore detectors system, to monitor the position of control rod or shutdown rod, in the event of a malfunction of the microprocessor rod position indication system. The use of this alternative method would reduce the required frequency of flux mapping using the movable incore detector system to determine the position of the control or shutdown rod position that is not being indicated. This will reduce the wear on the movable incore detector system that is also used to complete other required TS surveillances.

The licensee's staff subsequently informed the NRC that the amended TS pages included footers that incorrectly referenced the previous version of those TS pages. The licensee also reported that the amended page 3 of the operating license was missing from Amendment No. 114. As a result, the licensee is unable to implement the Amendment within 30 days from the date of the Amendment, as required.

Enclosure 1 includes revised Amendment No. 114 to Renewed Facility Operating License No. DPR-18 that reflects an implementation period of 60 days. Enclosure 2 includes page 3 of the operating license and the corrected TS pages.

If you have any questions, please contact me at (301) 415-1476 or email Mohan.Thadani@nrc.gov .

Sincerely,

Mohan C. Thadani, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-244

Enclosures:
1. Revised Amendment No. 114 to Renewed License No. DPR-18
2. Corrected TS Pages

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R.E. GINNA NUCLEAR POWER PLANT, LLC

DOCKET NO. 50-244

R.E. GINNA NUCLEAR POWER PLANT

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 114
Renewed License No. DPR-18

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:

   A. The application for amendment filed by the R.E. Ginna Nuclear Power Plant, LLC (the licensee) dated April 20, 2012, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-18 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 114, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

Douglas V Pickett for

Robert Beall, Acting Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the License and Technical Specifications

Date of Issuance: July 18, 2013

(b) Pursuant to the Act and 10 CFR Part 70, to possess and use four (4) mixed oxide fuel assemblies in accordance with the RG&E's application dated December 14, 1979 (transmitted by letter dated December 20, 1979), as supplemented February 20, 1980 and March 5, 1980;

(3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

(1) Maximum Power Level

Ginna LLC is authorized to operate the facility at steady-state power levels up to a maximum of 1775 megawatts (thermal).

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 114, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Fire Protection

(a) The licensee shall implement and maintain in effect all fire protection features described in the licensee's submittals referenced in and as approved or modified by the NRC's Fire Protection Safety Evaluation (SE) dated February 14, 1979, and

3.1 REACTIVITY CONTROL SYSTEMS

3.1.7 Rod Position Indication

LCO 3.1.7 The Microprocessor Rod Position Indication (MRPI) System and the Demand Position Indication System shall be OPERABLE.

APPLICABILITY: MODE 1,
MODE 2 with $K_{eff} \geq 1.0$.

ACTIONS

- NOTE -

Separate Condition entry is allowed for each inoperable MRPI per group and each demand position indicator per bank.

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	One MRPI per group inoperable for one or more groups.	A.1	Verify the position of the rods with inoperable position indicators by using movable incore detectors.	Once per 8 hours
		OR		
		A.2	Reduce THERMAL POWER to ≤ 50% RTP.	8 hours
		OR		

CONDITION	REQUIRED ACTION	COMPLETION TIME
	- NOTE - Rod position monitoring by Actions A.3.1 and A.3.2 may be applied to only one inoperable rod position indicator and shall only be allowed until an entry into MODE 5	
	A.3.1 Verify the position of the non-indicating rod indirectly by using movable incore detectors.	8 hours AND Once within 8 hours of rod control system indication of potential rod movement AND Once per 31 days thereafter
	AND	
	A.3.2 Review the parameters of the rod control system for indications of rod movement for the rod with an inoperable position indicator.	16 hours AND Once per 8 hours thereafter
B. One or more rods with inoperable position indicators have been moved > 24 steps in one direction since the last determination of the rod's position.	B.1 Verify the position of the rods with inoperable position indicators by using movable incore detectors. OR	4 hours
	B.2 Reduce THERMAL POWER to ≤ 50% RTP.	8 hours

CONDITION		REQUIRED ACTION		COMPLETION TIME
C.	One demand position indicator per bank inoperable for one or more banks.	C.1.1	Verify by administrative means all MRPIs for the affected banks are OPERABLE. AND	Once per 8 hours
		C.1.2	Verify the most withdrawn rod and the least withdrawn rod of the affected banks are ≤ 12 steps from the OPERABLE demand position indicator for that bank. OR	Once per 8 hours
		C.2	Reduce THERMAL POWER to ≤ 50% RTP.	8 hours
D.	Required Action and associated Completion Time of Condition A, Condition B or Condition C not met.	D.1	Be in MODE 2 with K_{eff} < 1.0.	6 hours
E.	More than one MRPI per group inoperable for one or more groups. OR More than one demand position indicator per bank inoperable for one or more banks.	E.1	Enter LCO 3.0.3.	Immediately

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.1.7.1	Verify each MRPI agrees within 12 steps of the group demand position for the full indicated range of rod travel.	Prior to reactor criticality after each removal of the reactor head

If you have any questions, please contact me at (301) 415-1476 or email Mohan.Thadani@nrc.gov .

Sincerely,
/ra/

Mohan C. Thadani, Senior Project Manager
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-244

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