



10 CFR 50.71(e) and 10 CFR 50.4(b)(6)

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July 1, 2013

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U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555-0001

Subject:

Duke Energy Carolinas, LLC (Duke Energy)

Oconee Nuclear Station, Units 1, 2, and 3 Docket Nos. 50-269, 50-270, 50-287

Updated Final Safety Analysis Report (UFSAR)

2012 Revision

Pursuant to 10 CFR 50.71(e) and 10 CFR 50.4(b)(6), enclosed is a copy of the 2012 revision of the Updated Final Safety Analysis Report (UFSAR) for Oconee Nuclear Station. The effective date of the revision is December 31, 2012.

The first part of the enclosure provides instructions for inserting this revision into the existing manuals including a list of effective pages required by 10 CFR 50.71(e). Changes made in this revision are indicated by a change bar in the margin of the changed page. The attachment provides a list which summarizes information removed from the UFSAR in this revision. The attachment complies with Regulatory Guide 1.181 (NEI 98-03 Revision 1).

Several changes to the summaries of aging management programs credited for license renewal contained in Chapter 18 are included in this 2012 revision. As required by 10 CFR 54.37(b) and with guidance and clarification provided by an NRC license renewal Frequently Asked Questions (FAQ), aging management reviews supporting changes to these programs are being addressed in a separate submittal.

This submittal document contains no regulatory commitments.

If there are any questions, please contact Susan Perry at (864) 873-4370.

I declare under penalty of perjury that the foregoing is true and correct. Executed on the 1st day of July, 2013.

Sincerely,

Enclosure and Attachment

A 053 NRP2

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XC;

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Regulatory Affairs K. R. Alter T. L. Patterson R. H. Guy S. C. Perry

ONS UFSAR UPDATE INSERTION INSTRUCTIONS

Submittal Due Date: 6/30/13 **Effective Date: 12/31/12**

- 1. Replace List of Effective Pages (LOEP) for Tables and Figures with the 2012 LOEP Update.
- 2. Replace <u>entire</u> text portions (including the Table of Contents, List of Figures, and List of Tables) for each chapter with the updated text portion.
- 3. Update Tables and Figures according to the following instructions.

Note: Tables and Figures from prior year were re-issued in order to remove all revision bars.

	Remove	<u>Insert</u>
Chapter 3	Table 3-68 (7 pgs)	Table 3-68 (8 pgs)
Chapter 4	Table 4-3 (2 pgs) Table 4-4 Table 4-6 Table 4-9 Table 4-10 Table 4-11 Table 4-20	Table 4-3 (2 pgs) Table 4-4 Table 4-6 Table 4-9 Table 4-10 Table 4-11 Table 4-20
	Figure 4-5 Figure 4-6	Figure 4-5 Figure 4-6
Chapter 5	Table 5-16	Table 5-16
Chapter 6	Table 6-7 (7 pgs) Table 6-25 Table 6-26 (2 pgs)	Table 6-7 (7 pgs) Table 6-25 Table 6-26 (2 pgs)
Chapter 7	Table 7-1 Table 7-2 Table 7-3 Table 7-4	Table 7-1 Table 7-2 Table 7-3 Table 7-4
	Figure 7-1 (11 x 17) (1 pg) Figure 7-4 (11 x 17) (1 pg) Figure 7-5 (11 x 17) (1 pg) Figure 7-6 (11 x 17) (1 pg) Figure 7-8 Figure 7-9 Figure 7-10	Figure 7-1 (11 x 17 Sht. 1 of 16 only) (16 pgs) Figure 7-4 (11 x 17) (1 pg) Figure 7-5 (11 x 17 Sht.1of 8 only) (8 pgs) Figure 7-6 (11 x 17) (1 pg) Figure 7-8 Figure 7-9 Figure 7-10
Chapter 8	Figure 8-4 (11 x 17) (3 pgs) Figure 8-5 (11 x 17) (1 pg)	Figure 8-4 (11 x 17) (3 pgs) Figure 8-5 (11 x 17) (1 pg)
Chapter 9	Table 9-6 Table 9-16	Table 9-6 Table 9-16
	Figure 9-24 (11 x 17) (1pg)	Figure 9-24 (11 x 17) (1 pg)
Chapter 11	Table 11-6 (7 pgs)	Table 11-6 (7 pgs)

ONS UFSAR UPDATE INSERTION INSTRUCTIONS

Submittal Due Date: 6/30/13 **Effective Date: 12/31/12**

	Remove	<u>Insert</u>
Chapter 13	Figure 13-1	Figure 13-1
_	Figure 13-3	Figure 13-3
	Figure 13-4	Figure 13-4
	Figure 13-7	Figure 13-7 (deleted pg)
		Figure 13-8 (new)
Chapter 15	***************************************	Tab 15.18
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	Table 15-47	Table 15-47
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OCONEE UFSAR - 2012 UPDATE

List of Effective Pages (LOEP) for Tables

The purpose of this list is to assure that the pages in the Tables section of your manual match the most recent issue, as well as to show a full accounting of all tables, including those that have been deleted. The earliest effective date, 12/31/00, was used when all tables were re-issued.

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Chapter 1		
-	1.1	Kan Datas in Osamas History
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12/31/08	2-7	Frequency of Tropical Cyclones in Georgia, South Carolina and North Carolina Plus Coastal Waters
12/31/08	2-8	Mean Monthly Thunderstorm Days and Thunderstorms for Nuclear Plant Site
12/31/08	2-9	Duration and Frequency (in Hours) of Calm and Near-Calm Winds Average of Three Locations* (1/59 - 12/63)
12/31/08	2-10	Annual Surface Wind Rose For Greenville, South Carolina (1/59 – 12/63)*
12/31/08	2-11	Percent Frequency of Wind Speeds at Various Hours Through the Day - Greenville, S. C. (1/59 - 12/63)*
12/31/08	2-12	Duration and Frequency of Calm and Near-Calm Winds Average of Three Locations* (1/59 - 12/63)
12/31/08	2-13	Percentage Distribution of Athens, Georgia Annual Winds at 0630 Eastern Standard Time (800-1300 Feet Above Ground)
12/31/08	2-14	Percentage Distribution of Athens, Georgia Annual Winds at 0630 Eastern Standard Time (2300-2800 Feet Above Ground)
12/31/08	2-15	Average Wind Direction Change with Height, Athens, Georgia, by Lapse Rates in the Lowest 50 Meters-Two Years of Record * •

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12/31/08	2-16	67.5î Sector Wind Direction Persistence Duration (in Hours) Greenville, S. C. WBAS
12/31/08	2-17	112.5î Sector Wind Direction Persistence Duration (in Hours) (Greenville, S. C. WBAS)
12/31/08	2-18	Surface Temperature (iF) Clemson, S. C. (68 Years of Record)*
12/31/08	2-19	Surface Precipitation (Inches) Clemson, S. C. (71 Years of Record)**
12/31/08	2-20	Precipitation - Wind Statistics - Greenville, S. C. 1959-1963 (By Precipitation Intensities)*
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12/31/08	2-22	Pasquill Stability Category and Supplemental Data for Greenville, S. C.
12/31/08	2-23	Average Temperature Difference (°F) at Minimum Temperature Time*
12/31/08	2-24	Joint Frequency Distribution of Wind Speed and Wind Direction for each Stability Class, for Greenville-Spartanburg, South Carolina for 1975 (Pages 1-8)
12/31/08	2-25	Joint Frequency Distribution of Wind Speed and Wind Direction for each Stability Class, for Greenville-Spartanburg, South Carolina for 1968-1972 (Pages 1-8)
12/31/08	2-26	Joint Frequencies of Wind Direction and Speed by Stability Class (March 1970-March 1972 (Pages 1-6)
12/31/08	2-27	Joint Frequency Tables of Wind Direction and Speed by Atmospheric Stability - Low and High Level (January 1975 - December 1975) (Pages 1-14)
12/31/08	2-28	Composite Poorest Diffusion Conditions Observed for Each Hour of Day (Based on 30 Months of Data)
12/31/08	2-29	Dispersion Factors Used for Accident and Routine Operational Analyses X/Q
12/31/08	2-30	Determining Appropriate Dispersion Factors (Pages 1-2)
12/31/08	2-31	Oconee Nuclear Station X/Q at Critical Receptors to 5 Miles* (Depleted by Dry Deposition) (Pages 1-2)
12/31/08	2-32	Oconee Nuclear Station D/Q at Critical Receptors to 5 Miles*
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	2-45	Deleted Per 2008 Update
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12/31/10	12/31/00	11-1	3" Liquid Waste Discharge
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13-2 Deleted Per 1999 Update 12/31/12 13-3 Nuclear Generation Department 12/31/12 13-4 Nuclear Generation - Oconee Nuclear Site 12/31/00 13-5 "At the Controls" Definition - Unit 1 & 2 12/31/00 13-6 "At the Controls" Definition - Unit 3 13-7 Deleted Per 2012 Update 12/31/12 13-8 Nuclear & PMC Organizational Structure	Chapter 13		
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·	12/31/05	15-1	Startup Accident
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12/31/05	15-4	Startum Aggidant
		Startup Accident
12/31/05	15-5	Startup Accident
12/31/05	15-6	Startup Accident Palated Pay 1008 Undete
	15-7	Deleted Per 1998 Update
	15-8	Deleted Per 1998 Update
	15-9	Deleted Per 1998 Update
10/21/02	15-10	Deleted Per 1998 Update
12/31/03	15-11	Rod Withdrawal at Power Accident - Peak RCS Pressure Analysis Power
12/31/03	15-12	Rod Withdrawal at Power Accident - Peak RCS Pressure Analysis RCS Temperatures
12/31/03	15-13	Rod Withdrawal at Power Accident - Peak RCS Pressure Analysis Pressurizer Level
12/31/03	15-14	Rod Withdrawal at Power Accident - Peak RCS Pressure Analysis RCS Pressure
12/31/03	15-15	Rod Withdrawal at Power Accident - Core Cooling Capability Analysis Power
12/31/03	15-16	Rod Withdrawal at Power Accident - Core Cooling Capability Analysis RCS Temperatures
12/31/03	15-17	Rod Withdrawal at Power Accident - Core Cooling Capability Analysis Pressurizer Level
12/31/08	15-18	Cold Water Accident - RCS Flow
12/31/08	15-19	Loss of Coolant Flow Accidents - Four RCP Coastdown From Four RCP Initial Conditions Analysis – RCS Flow
12/31/08	15-20	Loss of Coolant Flow Accidents - Four RCP Coastdown From Four RCP Initial Conditions Analysis - Power
12/31/08	15-21	Loss of Coolant Flow Accidents - Four RCP Coastdown From Four RCP. Initial Conditions Analysis - RCS Temperature
12/31/08	15-22	Loss of Coolant Flow Accidents - Four RCP Coastdown From Four RCP Initial Conditions Analysis – Pressurizer Level
12/31/08	15-23	Loss of Coolant Flow Accidents - Four RCP Coastdown From Four RCP Initial Conditions Analysis – RCS Pressure
12/31/08	15-24	Loss of Coolant Flow Accidents - Four RCP Coastdown From Four RCP Initial Conditions Analysis - DNBR
12/31/03	15-25	Loss of Coolant Flow Accidents – Two RCP Coastdown From Four RCP Initial Conditions Analysis – RCS Flow
12/31/10	15-26	Control Rod Misalignment Accidents - Dropped Rod Analysis - Neutron Power
12/31/10	15-27	Control Rod Misalignment Accidents - Dropped Rod Analysis – RCS Temperatures
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12/31/08	15-30	Rod Ejection Accident - BOC Three RCPs - Power
12/31/08	15-31	Rod Ejection Accident – BOC HZP – Power
12/31/08	15-31	Rod Ejection Accident – EOC Four RCPs – Power
12/31/08	15-32	Rod Ejection Accident – EOC Three RCPs – Power
12/31/08	15-34	Rod Ejection Accident – EOC HZP – Power
12/31/08	15-34	Rod Ejection Accident – BOC Four RCPs – Core Power Distribution
12/31/08	15-35	Rod Ejection Accident – BOC Three RCPs – RCS Pressure
12/31/06	15-30	Deleted Per 1999 Update
	15-37	Deleted Per 1999 Update
	15-39	Deleted Per 1999 Update
12/31/03	15-40	Steam Line Break Accident - With Offsite Power – Steam Line Pressure
12/31/03	15-41	Steam Line Break Accident - With Offsite Power - Break Flowrate
12/31/03	15-42	Steam Line Break Accident – With Offsite Power – RCS Temperature
12/31/03	15-43	Steam Line Break Accident – With Offsite Power – Reactivity
12/31/00	15-44	LOCA - Large Break Analysis Code Interfaces
12.31.00	15-45	Deleted Per 2000 Update
	15-46	Deleted Per 1990 Update
	15-47	Deleted Per 1997 Update
	15-48	Deleted Per 1997 Update
	15-49	Deleted Per 2000 Update
12/31/08	15-50	LOCA - Peak Cladding Temperature vs. Break Size for LBLOCA Spectrum
	15-51	Deleted Per 1997 Update
	15-52	Deleted Per 1995 Update
	15-53	Deleted Per 1995 Update
	15-54	Deleted Per 1995 Update
	15-55	Deleted Per 1995 Update
	15-56	Deleted Per 1995 Update
	15-57	Deleted Per 1995 Update
	15-58	Deleted Per 1995 Update
	15-59	Deleted Per 1995 Update
	15-60	Deleted Per 1995 Update
	15-61	Deleted Per 1995 Update
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Effective Date	Figure No.	Figure Title
	15-62	Deleted Per 1995 Update
	15-63	Deleted Per 1995 Update
	15-64	Deleted Per 1995 Update
	15-65	Deleted Per 1995 Update
	15-66	Deleted Per 1995 Update
	15-67	Deleted Per 1995 Update
	15-68	Deleted Per 1995 Update
	15-69	Deleted Per 1995 Update
	15-70	Deleted Per 1995 Update
	15-71	Deleted Per 1995 Update
	15-72	Deleted Per 1995 Update
	15-73	Deleted Per 1995 Update
	15-74	Deleted Per 1995 Update
	15-75	Deleted Per 1995 Update
	15-76	Deleted Per 1995 Update
	15-77	Deleted Per 1995 Update
	15-78	Deleted Per 1995 Update
	15-79	Deleted Per 1995 Update
12/31/00	15-80	MHA - Integrated Direct Dose
	15-81	Deleted Per 1995 Update
	15-82	Deleted Per 2000 Update
	15-83	Deleted Per 1995 Update
	15-84	Deleted Per 2000 Update
	15-85	Deleted Per 2000 Update
	15-86	Deleted Per 1997 Update
	15-87	Deleted Per 2000 Update
	15-88	Deleted Per 1995 Update
12/31/03	15-89	Post-Accident Hydrogen Control – Reactor Building Arrangement
	15-90	Deleted Per 1995 Update
	15-91	Deleted Per 1995 Update
	15-92	Deleted Per 1995 Update
	15-93	Deleted Per 1995 Update
	15-94	Deleted Per 1995 Update

Effective Date	Figure No.	Figure Title
	15-95	Deleted Per 1995 Update
	15-96	Deleted Per 1995 Update
	15-97	Deleted Per 1995 Update
	15-98	Deleted Per 1995 Update
	15-99	Deleted Per 1995 Update
	15-100	Deleted Per 1995 Update
	15-101	Deleted Per 1995 Update
	15-102	Deleted Per 1995 Update
	15-103	Deleted Per 1995 Update
	15-104	Deleted Per 1995 Update
	15-105	Deleted Per 1995 Update
	15-106	Deleted Per 1995 Update
	15-107	Deleted Per 1995 Update
	15-108	Deleted Per 1995 Update
	15-109	Deleted Per 1995 Update
	15-110	Deleted Per 2001 Update
	15-111	Deleted Per 2003 Update
12/31/08	15-112	LOCA kW/ft Limits vs. Core Elevation for MK-B10T Fuel
12/31/03	15-113	Rod Withdrawal at Power Accident – Core Cooling Capability Analysis RCS Pressure
12/31/08	15-114	Rod Withdrawal at Power Accident - Core Cooling Capability Analysis DNBR
12/31/08	15-115	Cold Water Accident - Core Average Temperature
12/31/08	15-116	Cold Water Accident - Power
12/31/08	15-117	Cold Water Accident - Cold Leg Temperature
12/31/08	15-118	Cold Water Accident – RCS Pressure
12/31/03	15-119	Loss of Coolant Flow Accidents – Two RCP Coastdown from Four RCP Initial Conditions Analysis – Power
12/31/03	15-120	Loss of Coolant Flow Accidents – Two RCP Coastdown from Four RCP Initial Conditions Analysis – RCS Temperature
12/31/03	15-121	Loss of Coolant Flow Accidents – Two RCP Coastdown from Four RCP Initial Conditions Analysis – Pressurizer Level
12/31/03	15-122	Loss of Coolant Flow Accidents – Two RCP Coastdown from Four RCP Initial Conditions Analysis – RCS Pressure
12/31/08	15-123	Loss of Coolant Flow Accidents – Two RCP Coastdown from Four RCP Initial Conditions Analysis – DNBR

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12/31/03	15-127	Loss of Coolant Flow Accidents – One RCP Coastdown from Three RCP Initial Conditions Analysis – Pressurizer Level
12/31/03	15-128	Loss of Coolant Flow Accidents - One RCP Coastdown from Three RCP Initial Conditions Analysis - RCS Pressure
12/31/08	15-129	Loss of Coolant Flow Accidents - One RCP Coastdown from Three RCP Initial Conditions Analysis - DNBR
12/31/03	15-130	Loss of Coolant Flow Accidents – Locked Rotor From Four RCP Initial Conditions Analysis – RCS Flow
12/31/03	15-131	Loss of Coolant Flow Accidents – Locked Rotor From Four RCP Initial Conditions Analysis – Power
12/31/03	15-132	Loss of Coolant Flow Accidents - Locked Rotor From Four RCP Initial Conditions Analysis - RCS Temperature
12/31/03	15-133	Loss of Coolant Flow Accidents – Locked Rotor From Four RCP Initial Conditions Analysis – Pressurizer Level
12/31/03	15-134	Loss of Coolant Flow Accidents – Locked Rotor From Four RCP Initial Conditions Analysis – RCS Pressure
12/31/08	15-135	Loss of Coolant Flow Accidents – Locked Rotor From Four RCP Initial Conditions Analysis – DNBR
12/31/08	15-136	Loss of Coolant Flow Accidents – Locked Rotor From Three RCP Initial Conditions Analysis – RCS Flow
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12/31/08	15-138	Loss of Coolant Flow Accidents – Locked Rotor From Three RCP Initial Conditions Analysis – RCS Temperatures
12/31/08	15-139	Loss of Coolant Flow Accidents – Locked Rotor From Three RCP Initial Conditions Analysis – Pressurizer Level
12/31/08	15-140	Loss of Coolant Flow Accidents – Locked Rotor From Three RCP Initial Conditions Analysis – RCS Pressure
12/31/11	15-141	Loss of Coolant Flow Accidents – Locked Rotor From Three RCP Initial Conditions Analysis – DNBR
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12/31/10	15-143	Control Rod Misalignment Accidents - Dropped Rod - RCS Pressure

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12/31/10	15-144	Control Rod Misalignment Accidents – Dropped Rod – DNBR
12/31/10	15-145	Turbine Trip Accident – Steam Generator Pressure
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12/31/08	15-147	Turbine Trip Accident – Ressurizer Level
12/31/08	15-148	Turbine Trip Accident – RCS Pressure
12/31/08	15-149	Turbine Trip Accident – Power
12/31/08	15-150	Steam Generator Tube Rupture – Power
12/31/11	15-151	Steam Generator Tube Rupture – Break Flow
12/31/11	15-151	Steam Generator Tube Rupture – RCS Pressure
12/31/11	15-152	Steam Generator Tube Rupture – Pressurizer Level
12/31/11	15-154	Steam Generator Tube Rupture – Pressurizer Level Steam Generator Tube Rupture – Steam Generator Pressure
12/31/11	15-154	Steam Generator Tube Rupture – Steam Generator Level
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12/31/03	15-157	Steam Line Break Accident – With Offsite Power – RCS Pressure
12/31/03	15-158	Steam Line Break Accident – With Offsite Power – Core Inlet Flow
12/31/03	15-160	Deleted per 2003 Update
12/31/03	15-161	Steam Line Break Accident – Without Offsite Power – Steam Line Pressure
12/31/03	15-162	Steam Line Break Accident – Without Offsite Power – RCS Temperatures
12/31/03	15-163	Steam Line Break Accident – Without Offsite Power – RCS Flow
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12/31/03	15-168	Small Steam Line Break – Steam Mass Flows
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12/31/03	15-172	Small Steam Line Break – RCS Hot Leg Pressure
		LOCA kW/ft Limits vs. Core Elevation for MK-B11 and MK-B11A Fuel
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12/31/03	15-175	Oconee – No CHRS Flow
	15-176	Deleted Per 2001 Update

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12/31/03	15-179	BOL LBLOCA Limit Case – 2.506 – ft, Mk11(M5) Break Mass Flow Rates
12/31/03	15-180	BOL LBLOCA Limit Case – 2.506 – ft, Mk11(M5) Hot Channel Mass Flow Rates
12/31/03	15-181	BOL LBLOCA Limit Case – 2.506 – ft, Mk11(M5) Core Flooding Rate
12/31/03	15-182	BOL LBLOCA Limit Case – 2.506 – ft, Mk11(M5) HP Fuel & Clad Temperatures at Ruptured Location
12/31/03	15-183	BOL LBLOCA Limit Case – 2.506 – ft, Mk11(M5) HP Fuel & Clad Temperatures at Peak Unruptured Location
12/31/03	15-184	BOL LBLOCA Limit Case – 2.506 – ft, Mk11(M5) HC Filtered Heat Transfer Coefficients
12/31/03	15-185	BOL LBLOCA Limit Case – 2.506 – ft, Mk11(M5) Quench Front Advancement
12/31/03	15-186	0.15 ft2 CLPD, 102% of 2568 MWt SBLOCA - Pressure
12/31/03	15-187	0.15 ft2 CLPD, 102% of 2568 MWt SBLOCA – Break and ECCS Mass Flow Rates
12/31/03	15-188	0.15 ft2 CLPD, 102% of 2568 MWt SBLOCA - Hot Channel Levels
12/31/03	15-189	0.15 ft2 CLPD, 102% of 2568 MWt SBLOCA - Peak Cladding Temperature
12/31/03	15-190	0.15 ft2 CLPD, 102% of 2568 MWt SBLOCA – HC Vapor Temperature at Core Exit
12/31/03	15-191	0.15 ft2 CLPD, 102% of 2568 MWt SBLOCA - HC Heat Transfer Coefficient
12/31/03	15-192	0.15 ft2 CLPD, 77% of 2568 MWt SBLOCA - Pressure
12/31/03	15-193	0.15 ft2 CLPD, 77% of 2568 MWt SBLOCA - Break and ECCS Mass Flow Rates
12/31/03	15-194	0.15 ft2 CLPD, 77% of 2568 MWt SBLOCA - Hot Channel Levels
12/31/03	15-195	0.15 ft2 CLPD, 77% of 2568 MWt SBLOCA - Peak Cladding Temperature
12/31/03	15-196	0.15 ft2 CLPD, 77% of 2568 MWt SBLOCA – HC Vapor Temperature at Core Exit
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12/31/08	15-198	2.506-ft Mark-B-HTP Mixed-Core Bol LBLOCA Case - Reactor Vessel Upper Plenum Pressure
12/31/08	15-199	2.506-ft Mark-B-HTP Mixed-Core BOL LBLOCA Case - Break Mass Flow Rates
12/31/08	15-200	2.506-ft Mark-B-HTP Mixed-Core BOL LBLOCA Case - Hot Channel Mass Flow Rates
12/31/08	15-201	2.506-ft Mark-B-HTP Mixed-Core BOL LBLOCA Case - Core Flooding Rate
12/31/08	15-202	2.506-ft Mark-B-HTP Mixed-Core BOL LBLOCA Case - HP Fuel & Clad Temperatures at Ruptured Location

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12/31/08	15-206	2.506-ft Mark-B-HTP Mixed-Core MOL LBLOCA Case - HP Fuel & Clad Temperatures at Unruptured Location
12/31/08	15-207	102% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Spectrum Analysis
12/31/08	15-208	0.15 ft ² CLPD, 102% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Case-Pressure
12/31/08	15-209	0.15 ft ² CLPD, 102% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Case-Break and ECCS Mass Flow Rates
12/31/08	15-210	0.15 $\rm ft^2$ CLPD, 102% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Case-Hot Channel Levels
12/31/08	15-211	0.15 ft ² CLPD, 102% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Case-Peak Cladding Temperature
12/31/08	15-212	0.15 ft ² CLPD, 102% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Case-HC Vapor Temperature at Core Exit
12/31/08	15-213	77% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Spectrum Analysis
12/31/08	15-214	0.075 ft ² CLPD, 77% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Case-Pressure
12/31/08	15-215	0.075 ft ² CLPD, 77% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Case-Break and ECCS Mass Flow Rates
12/31/08	15-216	0.075 ft ² CLPD, 77% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Case-Hot Channel Levels
12/31/08	15-217	0.075 ft ² CLPD, 77% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Case-Peak Cladding Temperature
12/31/08	15-218	0.075 ft ² CLPD, 77% of 2568 MWt, Mark-B-HTP Mixed-Core SBLOCA Case-HC Vapor Temperature at Core Exit
12/31/11	15-219	Mark-B-HTP Full-Core BOL LBLOCA - Reactor Vessel Upper Plenum Pressure
12/31/11	15-220	Mark-B-HTP Full-Core BOL LBLOCA - Break Mass Flow Rates
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12/31/11	15-222	Mark-B-HTP Full-Core BOL LBLOCA - Core Flooding Rates
12/31/11	15-223	Mark-B-HTP Full-Core BOL LBLOCA - Hot Pin Fuel & Clad Temperatures at Ruptured Location
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12/31/11	15-227	102% of 2568 MWt, Full Core Mark-B-HTP SBLOCA Break Spectrum Analysis
12/31/11	15-228	0.15ft ² CLPD, 102% of 2568 MWt, Full Core Mark-B-HTP SBLOCA - Pressure
12/31/11	15-229	0.15ft ² CLPD, 102% of 2568 MWt, Full Core Mark-B-HTP SBLOCA - Break and ECCS Mass Flow Rates
12/31/11	15-230	0.15ft ² CLPD, 102% of 2568 MWt, Full Core Mark-B-HTP SBLOCA - RV Collapsed Liquid Level & Hot Channel Mixture Level
12/31/11	15-231	0.15ft ² CLPD, 102% of 2568 MWt, Full Core Mark-B-HTP SBLOCA - Hot Pin Peak Clad Temperature
12/31/11	15-232	0.15ft ² CLPD, 102% of 2568 MWt, Full Core Mark-B-HTP SBLOCA - Hot Channel Vapor Temperature at Core Exit

ATTACHMENT

SUMMARY OF INFORMATION REMOVED FROM REVISION 22 OF THE UFSAR

Attachment Oconee Nuclear Station UFSAR 2012 Update List of Removed Items

UFSAR Table 7-3

Notes 4 & 5 concerning Low Pressure Service Water (LPSW) valves 1055 & 1061, and LPSW-1054 & -1062 respectively, were deleted as a result of the Engineered Safeguards Protective System/Reactor Protective System modification.

UFSAR Section 9.5.1

Section 9.5.1 was re-written as a result of National Fire Protection Association (NFPA) 805 implementation. Sections 9.5.1.3 - 9.5.1.7.4 were removed/replaced by NFPA 805 information.

UFSAR Section 9.6.4.6.1

During re-write resulting from NFPA 805 implementation, information related to cold shutdown was deleted.

UFSAR Section 9.6.5

During re-write resulting from NFPA 805 implementation, unnecessary information concerning Appendix R events was deleted.

UFSAR Section 9.6.6

During re-write resulting from NFPA 805 implementation, references that were no longer applicable were deleted.

Table 15-16

Irrelevant entries, associated with the Large Main Steam Line Break and Small Main Steam Line Break, were deleted as a result of the Table update to include calculated Total Effective Dose Equivalent (TEDE) doses in accordance with AST methodology.

UFSAR Section 18.3.1.2

Deleted paragraph that referenced EA-03-009 since it no longer applies.

UFSAR Table 18-1

The "Boric Acid Corrosion Program" entry that was listed after "Flow Accelerated Corrosion Program" was deleted; the "Boric Acid Corrosion Program" was then placed in the correct location in the Table.