



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 31, 2013

Mr. Michael J. Pacilio
Senior Vice President
Exelon Generation Company, LLC
President and Chief Nuclear Officer (CNO)
Exelon Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: LASALLE COUNTY STATION, UNITS 1 AND 2, ISSUANCE OF AMENDMENT NOS 207 AND 194 RE: ADMINISTRATIVE CHANGES TO REMOVE OBSOLETE LICENSE CONDITIONS RELATED TO BORAFLEX DEGRADATION OF THE UNIT 2 SPENT FUEL POOL (TAC NOS. ME9799 AND MF2503)

Dear Mr. Pacilio:

The U.S. Nuclear Regulatory Commission (NRC, the Commission) has issued the enclosed Amendment No. 207 to Facility Operating License No. NPF-11 and Amendment No. 194 to Facility Operating License No. NPF-18 for the LaSalle County Station (LSCS), Units 1 and 2, respectively. The amendments are in response to your application dated October 15, 2012, as supplemented by letter dated August 12, 2013.

On December 29, 2011, Exelon Generation Company, LLC, completed installation of NETCO-SNAP-IN® (NETCO) inserts into the LSCS, Unit 2, spent fuel pool (SFP) which satisfied several license conditions. The enclosed amendments remove license conditions which are no longer required to restrict the usage of storage cell locations that credit Boraflex for nuclear criticality safety in the LSCS, Unit 2, SFP. By letter dated August 12, 2013, the licensee requested to add a restriction to Technical Specification (TS) 4.3 "Fuel Storage," to prevent storage of nuclear fuel in cell locations without NETCO inserts for LSCS, Unit 2. Additionally, the enclosed license pages correct typographical and format errors previously introduced by the NRC staff.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Nicholas DiFrancesco".

Nicholas DiFrancesco, Project Manager
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

Enclosures:

1. Amendment No. 207 to NPF-11
 2. Amendment No. 194 to NPF-18
 3. Safety Evaluation
- cc w/encls: Distribution via Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-373

LASALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 207
License No. NPF-11

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Exelon Generation Company, LLC (the licensee), dated October 15, 2012, as supplemented by letter dated August 12, 2013, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 207, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, reading "Travis L. Tate".

Travis L. Tate, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: October 31, 2013



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-374

LASALLE COUNTY STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 194
License No. NPF-18

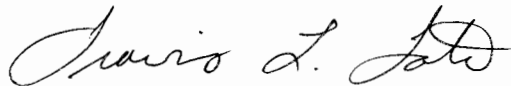
1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Exelon Generation Company, LLC (the licensee), dated October 15, 2012, as supplemented by letter dated August 12, 2013, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, paragraphs 2.C.(30), 2.C.(31), and 2.C.(32) of the Facility Operating License No. NPF-18 are hereby deleted, and paragraph 2.C.(2) of the Facility Operating License No. NPF-18 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 194, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Travis L. Tate, Chief
Plant Licensing Branch III-2
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications and Facility Operating License

Date of Issuance: October 31, 2013

ATTACHMENT TO LICENSE AMENDMENT NOS. 207 AND 194

FACILITY OPERATING LICENSE NOS. NPF-11 AND NPF-18

DOCKET NOS. 50-373 AND 50-374

Replace the following pages of the Facility Operating Licenses and the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

License NPF-11
Page 3

License NPF-18
Page 3
Page 8
Page 8a

TSs
4.0-2

INSERT

License NPF-11
Page 3

License NPF-18
Page 3
Page 8

TSs
4.0-2

Am. 146
01/12/01 (4) Exelon Generation Company, LLC, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

Am. 202
07/21/11 (5) Exelon Generation Company, LLC, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of LaSalle County Station, Units 1 and 2 and such Class B and Class C low-level radioactive waste as may be produced by the operation of Braidwood Station, Units 1 and 2, Byron Station, Units 1 and 2, and Clinton Power Station, Unit 1.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

Am. 198
09/16/10 (1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of full power (3546 megawatts thermal).

Am. 207
10/31/13 (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 207, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

Am. 194
08/28/09 (3) DELETED

Am. 194
08/28/09 (4) DELETED

Am. 194
08/28/09 (5) DELETED

Am. 194
08/28/09 (6) DELETED

Am. 194
08/28/09 (7) DELETED

Am. 189
07/21/11 (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70 possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of LaSalle County Station, Units 1 and 2, and such Class B and Class C low-level radioactive waste as may be produced by the operation of Braidwood Station, Units 1 and 2, Byron Station, Units 1 and 2, and Clinton Power Station, Unit 1.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

Am. 185
09/16/10 (1) Maximum Power Level
The licensee is authorized to operate the facility at reactor core power levels not in excess of full power (3546 megawatts thermal). Items in Attachment 1 shall be completed as specified. Attachment 1 is hereby incorporated into this license.

Am. 194
10/31/13 (2) Technical Specifications and Environmental Protection Plan
The Technical Specifications contained in Appendix A, as revised through Amendment No. 194, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

Am. 181
08/28/09 (3) DELETED

Am. 181
08/28/09 (4) DELETED

Am. 181
08/28/09 (5) DELETED

Am. 181
08/28/09 (6) DELETED

Am. 181
08/28/09 (7) DELETED

Am. 181
08/28/09 (8) DELETED

Am. 181
08/28/09 (9) DELETED

- (c) The first performance of the periodic measurement of CRE pressure, Specification 5.5.15.d, shall be within 24 months, plus the 6 months allowed by SR 3.0.2, as measured from the date of the most recent successful pressure measurement test, or within 6 months if not performed previously.

Am. 194 (30) DELETED
10/31/13

Am. 194 (31) DELETED
10/31/13

Am. 194 (32) DELETED
10/31/13

Am. 186 (33) The methodology in AREVA NP Inc Report No. ANP-2843(P), "LaSalle Unit 2 Nuclear Power Station Spent Fuel Storage Pool Criticality Safety Analysis with Neutron Absorbing Inserts and Without Boraflex," Revision 1, dated August 2009, as corrected by Attachment 3 to a letter dated June 10, 2010 from P. Simpson to the NRC, shall be used to perform required criticality calculations associated with the storage cells containing NETCO-SNAP-IN[®] inserts.

Am. 192 (34) Use of Global Nuclear Fuel – Americas, LLC, Report, NEDC-33106P, "GEXL97 Correlation for Atrium-10 Fuel," Revision 4, August 2012, for LaSalle Unit 2 shall be limited to the same range of applicability for calculations of Safety Limit Minimum Critical Power Ratios as documented in NRC letter from W.A. Macon, Jr. (NRC) to J. L. Skolds, "LaSalle County Station, Units 1 and 2 – Correction to Issuance of Amendments (TAC Nos. MB9888 and MB9889)," dated January 14, 2004 (ADAMS Accession Number ML040130278).

4.0 DESIGN FEATURES (continued)

4.3 Fuel Storage

4.3.1 Criticality

4.3.1.1 The spent fuel storage racks are designed and shall be maintained with:

- a. $k_{eff} \leq 0.95$ if fully flooded with unborated water, which includes an allowance for uncertainties as described in either: (1) Section 9.1.2 of the UFSAR, or (2) AREVA NP Inc. Report No. ANP-2843(P), "LaSalle Unit 2 Nuclear Power Station Spent Fuel Storage Pool Criticality Safety Analysis with Neutron Absorbing Inserts and Without Boraflex," Revision 1, dated August 2009, for the Unit 2 spent fuel storage racks with rack inserts.
- b. A nominal 6.26 inch center to center distance between fuel assemblies placed in the storage racks.
- c. For Unit 2 only, spent fuel shall only be stored in storage rack cells containing a neutron absorbing rack insert. The neutron absorbing rack inserts shall have a minimum certified ^{10}B areal density greater than or equal to 0.0086 grams $^{10}\text{B}/\text{cm}^2$. The approved inserts are those described in Attachment 4 to the letter from P. Simpson to the NRC, dated October 5, 2009.
- d. The combination of U-235 enrichment and gadolinia loading shall be limited to ensure fuel assemblies have a maximum k-infinity of 0.9185 for all lattices in the top of the assembly, a maximum k-infinity of 0.8869 for all lattices in the intermediate portion of the assembly, and a maximum k-infinity of 0.8843 for all lattices in the bottom of the assembly as determined at 4°C in the normal spent fuel pool in-rack configuration. The bottom, intermediate, and top zones are between 0"-96", 96"-126", and greater than 126" above the bottom of the active fuel.

(continued)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 207 TO FACILITY OPERATING LICENSE NO. NPF-11
AND AMENDMENT NO. 194 TO FACILITY OPERATING LICENSE NO. NPF-18
EXELON GENERATION COMPANY, LLC
LASALLE COUNTY STATION, UNITS 1 AND 2
DOCKET NOS. 50-373 AND 50-374

1.0 INTRODUCTION

By application dated October 15, 2012 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML12290A111), as supplemented by letter dated August 12, 2013 (ADAMS Accession No. ML13224A231), Exelon Generation Company, LLC (the licensee) requested changes to the facility operating license for LaSalle County Station (LSCS), Unit 2. The proposed changes would remove License Conditions 2.C.(30), 2.C.(31), and 2.C.(32) which controlled the spent fuel pool (SFP) configuration in order to address the degradation of Boraflex material until NETCO-SNAP-IN[®] (NETCO) inserts could be installed.

The proposed changes would remove these license conditions because, following the installation of NETCO inserts on December 29, 2011, they are no longer required to restrict the use of storage cell locations that credited Boraflex in the nuclear criticality analysis for the LSCS, Unit 2, SFP. The licensee also requested a change to Technical Specification (TS) 4.3.1.1.c to require that spent fuel shall only be stored in storage rack cells containing the approved neutron absorbing rack insert.

Additionally, the enclosed license pages correct typographical and format errors previously introduced by the NRC staff in LSCS license amendments. Specifically, the enclosed NPF-18 license page 8 deletes "(30)" which was inadvertently added at the end of License Condition 2.C.(29), and relocates License Condition 2.C.(34) to page 8. Enclosed NPF-11 license page 3, and NPF-18 license pages 3 and 8, add the amendment approval number and date for the corresponding revision to the existing license conditions for clarity and consistency.

The staff received no supplemental information after the revised initial proposed finding of no significant hazards consideration was published on September 3, 2013 (78 FR 54283).

Enclosure

2.0 REGULATORY EVALUATION

The regulatory requirements related to this application are as follows.

Title 10 of the *Code of Federal Regulations* (10 CFR), Part 50, Appendix A, Criterion 62, "Prevention of criticality in fuel storage and handling," which requires that criticality in the fuel storage and handling system shall be prevented by physical systems or processes, preferably by use of geometrically safe configurations.

10 CFR 50.68(b)(1), which requires that:

Plant procedures shall prohibit the handling and storage at any one time of more fuel assemblies than have been determined to be safely subcritical under the most adverse moderation conditions feasible by unborated water.

10 CFR 50.68(b)(4), which requires that:

If no credit for soluble boron is taken, the k-effective of the spent fuel storage racks loaded with fuel of the maximum fuel assembly reactivity must not exceed 0.95, at a 95 percent probability, 95 percent confidence level, if flooded with unborated water. If credit is taken for soluble boron, the k-effective of the spent fuel storage racks loaded with fuel of the maximum fuel assembly reactivity must not exceed 0.95, at a 95 percent probability, 95 percent confidence level, if flooded with borated water, and the k-effective must remain below 1.0 (subcritical), at a 95 percent probability, 95 percent confidence level, if flooded with unborated water.

10 CFR 50.36(c)(4), which requires that:

Design features to be included are those features of the facility such as materials of construction and geometric arrangements, which, if altered or modified, would have a significant effect on safety and are not covered in categories described in paragraphs (c) (1), (2), and (3) of this section.

10 CFR 50.71(e), which requires that each licensee shall update periodically its updated final safety analysis report (UFSAR) to assure that the information included in this report contains the latest information developed.

The LSCS, Unit 2, SFP does not contain soluble boron. Therefore, the regulatory requirement that is applicable to it is that the LSCS, Unit 2, SFP effective neutron multiplication factor (keff) must remain less than or equal to 0.95, at a 95 percent probability, 95 percent confidence level (95/95), if flooded with unborated water.

3.0 TECHNICAL EVALUATION

As part of granting a previous licensee amendment issued on January 28, 2011 (ADAMS Accession No. ML110250051), the staff evaluated and approved (1) the use of NETCO rack inserts constructed of Rio Tinto Alcan composite material; (2) the licensee's associated Long-

Term Surveillance Program; and (3) the criticality safety analysis methodology provided in License Condition 2.C.(33) and AREVA NP Inc, Report No. ANP 2843(P), "LaSalle Unit 2 Nuclear Power Station Spent Fuel Storage Pool Criticality Safety Analysis with Neutron Absorbing Inserts and Without Boraflex," Revision 1, dated August 2009, as corrected by Attachment 3 to a letter dated June 10, 2010 (ADAMS Accession No. ML101650228), from P. Simpson to the NRC.

Subsequently, by letter dated January 6, 2012 (ADAMS Accession No. ML120060432), the licensee notified the NRC that the NETCO insert installation had been completed on December 29, 2011. The NRC staff finds that completion of the NETCO insert installation placed LSCS in compliance with TS 4.3.1.1.c and satisfied the requirements of License Condition 2.C.(32). As explained below, the NRC staff technical review confirmed that the equivalent level of restriction was maintained to preventing the storage of nuclear fuel in spent fuel pool cells without NETCO inserts, and ensured that the staff's conclusions established in the amendment approval dated January 28, 2011, remain satisfied.

3.1 License Conditions C.2.(30) and C.2.(31)

The License Conditions C.2.(30) and C.2.(31) were established to restrict the use of cell locations without NETCO inserts based on the B10 areal density with degraded boraflex material for the LSCS, Unit 2, SFP.

Once NETCO inserts were installed in these locations on December 29, 2011, the restrictions for locations which credited boraflex for neutron absorption became obsolete. Instead, the NETCO inserts and the criticality safety analysis contained in License Condition 2.(C).33 ensure that the regulatory requirements remain satisfied.

Additionally, by letter dated August 12, 2013, the licensee proposed to change TS 4.3.1.1.c to state that spent fuel shall only be stored in storage rack cells containing a neutron absorbing rack insert.

The NRC staff finds that, combined with the installation of NETCO inserts, the requirements of License Condition C.2.(33), and the requested amendment to TS 4.3.1.1.c, the deletion of License Conditions C.2.(30) and C.2.(31) is acceptable because nuclear fuel is not stored in cell locations without NETCO inserts during normal operations, or inspection and maintenance activities.

3.2 License Condition C.2.(32)

License Condition C.2.(32) required the installation of NETCO inserts in all accessible storage rack cells for the LSCS, Unit 2, SFP no later than December 31, 2014. The licensee completed the installation required by this license condition on December 29, 2011. Therefore, the staff finds that License Condition C.2.(32) is obsolete and may be removed.

3.3 Revisions to TS 4.3.1.1.c

By letter dated August 12, 2013, the licensee requested to revise TS 4.3.1.1.c to state that spent fuel shall only be stored in storage rack cells containing a neutron absorbing rack insert. The NETCO neutron absorbing rack insert design was evaluated and approved for this use by the NRC staff in January 28, 2011 (ADAMS Accession No. ML110250051). The requested revision to TS 4.3.1.1.c prevents the loading of nuclear fuel in cells which contain only Boraflex. Additionally, the licensee committed to removing criticality safety methodologies and surveillance programs for Boraflex material from its Updated Final Safety Analysis Report (UFSAR).

The NRC staff finds that these revisions of TS 4.3.1.1.c and the UFSAR address the effects of Boraflex degradation because the licensing and design basis will now rely only on the NETCO inserts for criticality safety. Furthermore, the revision to TS 4.3.1.1.c ensures that 10 CFR 50.36(c)(4) is satisfied regarding the inclusion in the TS of design features, which, if altered or modified, would have a significant effect on safety.

3.3 Removal of Obsolete Criticality Safety Methodology and License Condition C.2.(33)

The staff confirmed that the licensee does not intend to maintain as part of its licensing basis the criticality methodologies which credit Boraflex. By letter dated August 12, 2013, the licensee stated that it planned to remove the criticality methodology for Boraflex from its UFSAR. The licensee also stated that this was being tracking in its 10 CFR Part 50, Appendix B Corrective Action System. The staff finds that this licensee response is satisfactory because it is consistent with the requirements in 10 CFR 50.71(e) to maintain an updated UFSAR, which includes the removal from the UFSAR of obsolete information such as criticality methodologies that are no longer utilized.

The criticality safety methodology for LSCS, Unit 2, is described in License Condition C.2.(33), which states:

(33) The methodology in AREVA NP Inc, Report No. ANP-2843(P) "LaSalle Unit 2 Nuclear Power Station Spent Fuel Storage Pool Criticality Safety Analysis with Neutron Absorbing Inserts and Without Boraflex," Revision 1, dated August 2009, as corrected by Attachment 3 to a letter dated June 10, 2010 from P. Simpson to the NRC, shall be used to perform required criticality calculations associated with the storage cells containing NETCO-SNAP-IN® inserts.

The staff previously found this methodology to be acceptable in an amendment dated January 28, 2011 (ADAMS Accession No. ML110250051). Since this acceptable criticality safety methodology is part of the license, and since the Boraflex criticality safety methodology will be removed from the license and licensing basis, the staff finds that the removal of License Conditions C.2.(30), C.2.(31), and C.2.(32) is acceptable because the future crediting of Boraflex material in safety analyses is prevented. Therefore, the staff concludes that the proposed revisions to the license and TSs will ensure that the criticality safety limits continue to meet the requirements set forth in 10 CFR 50.68(b)(1) and 50.68(b)(4).

3.4 Conclusion

Based upon the discussion presented above in Sections 3.1, 3.2, and 3.3, the NRC staff finds that the removal of License Conditions C.2.(30), C.2.(31), and C.2.(32), and the modification of TS 4.3.1.1.c, are acceptable. These changes to the LSCS license for the Unit 2 SFP do not alter the basis of the prior amendment approval dated January 28, 2011, and its nuclear fuel criticality safety conclusions. The change to TS 4.3.1.1.c ensures that the licensee maintains restrictions to prevent nuclear fuel storage in cell locations without NETCO inserts and prevents future reliance on Boraflex for neutron absorption in criticality safety analyses. Therefore, the staff finds that the requirements of 10 CFR Sections 50.68(b)(1), 50.68(b)(4), and 50.36(c)(4) continue to be met.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 FINAL NO SIGNIFICANT HAZARDS CONSIDERATION DETERMINATION (NSHCD)

On September 3, 2013, the Commission issued a proposed finding that the amendment request involves no significant hazards consideration, along with an opportunity to comment upon that finding by October 3, 2013, and an opportunity to request a hearing by November 4, 2013 (78 FR 54283). No comments were received on the proposed no significant hazards consideration. The Commission has determined to issue this amendment prior to the November 4, 2013, deadline for hearing requests. Accordingly, the Commission has made a final determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment does not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), an evaluation of the issue of no significant hazards consideration is presented below:

1. Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

Response: No.

The proposed change removes License Conditions within the LSCS Unit 2 Operating License related to interim configurations of the SFP during the installation of the NETCO-SNAP-IN® inserts and the required completion date for installation. The proposed change also revises TS Section 4.3.1 to clarify that for the Unit 2 SFP, spent fuel shall only be stored in storage rack cells containing a neutron absorbing rack insert. All changes proposed by [the licensee] in this license amendment request are administrative in nature because they remove License Conditions that have either been satisfied or that are no longer applicable, and the revision to TS Section 4.3.1 ensures spent fuel is stored only in cells that contain inserts. There are no physical changes to the facilities,

nor any changes to the station operating procedures, limiting conditions for operation, or limiting safety system settings.

Therefore, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

Response: No.

The proposed change removes License Conditions within the LSCS Unit 2 Operating License related to interim configurations of the SFP during the installation of the NETCO-SNAP-IN® inserts and the required completion date for installation. The proposed change also revises TS Section 4.3.1 to clarify that for the Unit 2 SFP, spent fuel shall only be stored in storage rack cells containing a neutron absorbing rack insert. There are no changes to the SFP criticality analysis associated with the proposed change. No physical changes to the plant are proposed, and there are no changes to the manner in which the plant is operated. Rather, the proposed change is administrative because it involves removing License Conditions that have either been satisfied or that are no longer applicable, and the revision to TS Section 4.3.1 ensures spent fuel is stored only in cells that contain inserts.

Therefore, the proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. Does the proposed change involve a significant reduction in a margin of safety?

Response: No.

The proposed change removes License Conditions within the LSCS Unit 2 Operating License related to interim configurations of the SFP during the installation of the NETCO-SNAP-IN® inserts and the required completion date for installation. The proposed change also revises TS Section 4.3.1 to clarify that for the Unit 2 SFP, spent fuel shall only be stored in storage rack cells containing a neutron absorbing rack insert. Plant safety margins are established through limiting conditions for operation, limiting safety system settings, and safety limits specified in Technical Specifications. The proposed change does not alter these established safety margins. The proposed change does not alter the criticality analysis for the SFP and does not affect the SFP criticality safety margin. The proposed change is administrative because it involves removing License Conditions that have either been satisfied or that are no longer applicable, and the revision to TS Section 4.3.1 ensures spent fuel is stored only in cells that contain inserts.

Therefore, the proposed change does not involve a significant reduction in a margin of safety.

Based on the above evaluation, the NRC staff concludes that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff has made a final determination that no significant hazards consideration is involved for the proposed amendment and that the amendment should be issued as allowed by the criteria contained in 10 CFR 50.91.

6.0 ENVIRONMENTAL CONSIDERATION

The amendments change the requirements with respect to the installation or use of facility components located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has made a final determination that no significant hazards consideration is involved for the proposed amendment as discussed above in Section 5.0. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

7.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: N. DiFrancesco

Date: October 31, 2013

Mr. Michael J. Pacilio
 Senior Vice President
 Exelon Generation Company, LLC
 President and Chief Nuclear Officer (CNO)
 Exelon Nuclear
 4300 Winfield Road
 Warrenville, IL 60555

SUBJECT: LASALLE COUNTY STATION, UNITS 1 AND 2, ISSUANCE OF AMENDMENT NOS 207 AND 194 RE: ADMINISTRATIVE CHANGES TO REMOVE OBSOLETE LICENSE CONDITIONS RELATED TO BORAFLEX DEGRADATION OF THE UNIT 2 SPENT FUEL POOL (TAC NOS. ME9799 AND MF2503)

Dear Mr. Pacilio:

The U.S. Nuclear Regulatory Commission (NRC, the Commission) has issued the enclosed Amendment No. 207 to Facility Operating License No. NPF-11 and Amendment No. 194 to Facility Operating License No. NPF-18 for the LaSalle County Station (LSCS), Units 1 and 2, respectively. The amendments are in response to your application dated October 15, 2012, as supplemented by letter dated August 12, 2013.

On December 29, 2011, Exelon Generation Company, LLC, completed installation of NETCO-SNAP-IN® (NETCO) inserts into the LSCS, Unit 2, spent fuel pool (SFP) which satisfied several license conditions. The enclosed amendments remove license conditions which are no longer required to restrict the usage of storage cell locations that credit Boraflex for nuclear criticality safety in the LSCS, Unit 2, SFP. By letter dated August 12, 2013, the licensee requested to add a restriction to Technical Specification (TS) 4.3 "Fuel Storage," to prevent storage of nuclear fuel in cell locations without NETCO inserts for LSCS, Unit 2. Additionally, the enclosed license pages correct typographical and format errors previously introduced by the NRC staff.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

Nicholas DiFrancesco, Project Manager
 Plant Licensing Branch III-2
 Division of Operating Reactor Licensing
 Office of Nuclear Reactor Regulation

Docket Nos. 50-373 and 50-374

Enclosures:

1. Amendment No. 207 to NPF-11
2. Amendment No. 194 to NPF-18
3. Safety Evaluation

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OFFICE	LPL3-2/PM	LPL3-2/LA	DSS/SRXB	DSS/STSB	OGC	LPL3-2/BC	LPL3-2/PM
NAME	NDiFrancesco	SRohrer	CJackson	RElliot	JWachutka (NLO)	TTate	NDiFrancesco
DATE	09/17/13	09/18/13	10/08/13	10/08/13	10/07/13	10/31/13	10/31/13

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