



Exelon Generation®

LaSalle Station

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10 CFR 50.73

RA13-034

June 21, 2013

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

LaSalle County Station, Unit 1
Facility Operating License No. NPF-11
NRC Docket No. 50-373

Subject: Licensee Event Report 2013-004-00 Reactor Pressure Exceeded 150 psi
With Reactor Core Isolation Cooling Inoperable

In accordance with 10 CFR 50.73(a)(2)(i)(B), Exelon Generation Company (EGC), LLC,
is submitting Licensee Event Report Number 2013-004-00 for LaSalle Unit 1.

There are no regulatory commitments in this letter. Should you have any questions
concerning this report, please contact Mr. Guy V. Ford, Regulatory Assurance Manager,
at (815) 415-2800.

Respectfully,

Harold T. Vinyard
Plant Manager
LaSalle County Station

Enclosure: Licensee Event Report

cc: Regional Administrator – NRC Region III
NRC Senior Resident Inspector – LaSalle County Station

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

Estimated burden per response to comply with this mandatory collection request: 80 hours. Reported lessons learned are incorporated into the licensing process and fed back to industry. Send comments regarding burden estimate to the FOIA/Privacy Section (T-5 F53), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by internet e-mail to infocollects.resource@nrc.gov, and to the Desk Officer, Office of Information and Regulatory Affairs, NEOB-10202, (3150-0104), Office of Management and Budget, Washington, DC 20503. If a means used to impose an information collection does not display a currently valid OMB control number, the NRC may not conduct or sponsor, and a person is not required to respond to, the information collection.

1. FACILITY NAME LaSalle County Station, Unit 1	2. DOCKET NUMBER 05000373	3. PAGE 1 OF 3
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4. TITLE
Reactor Pressure Exceeded 150 psig With Reactor Core Isolation Cooling Inoperable

5. EVENT DATE			6. LER NUMBER			7. REPORT DATE			8. OTHER FACILITIES INVOLVED	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NO.	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
04	22	2013	2013	- 004	- 00	06	21	2013	N/A	N/A
									FACILITY NAME	DOCKET NUMBER
									N/A	N/A

9. OPERATING MODE 2	11. THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: <i>(Check all that apply)</i>
10. POWER LEVEL 000	<input type="checkbox"/> 20.2201(b) <input type="checkbox"/> 20.2203(a)(3)(i) <input type="checkbox"/> 50.73(a)(2)(i)(C) <input type="checkbox"/> 50.73(a)(2)(vii) <input type="checkbox"/> 20.2201(d) <input type="checkbox"/> 20.2203(a)(3)(ii) <input type="checkbox"/> 50.73(a)(2)(ii)(A) <input type="checkbox"/> 50.73(a)(2)(vii)(A) <input type="checkbox"/> 20.2203(a)(1) <input type="checkbox"/> 20.2203(a)(4) <input type="checkbox"/> 50.73(a)(2)(ii)(B) <input type="checkbox"/> 50.73(a)(2)(vii)(B) <input type="checkbox"/> 20.2203(a)(2)(i) <input type="checkbox"/> 50.36(c)(1)(i)(A) <input type="checkbox"/> 50.73(a)(2)(iii) <input type="checkbox"/> 50.73(a)(2)(ix)(A) <input type="checkbox"/> 20.2203(a)(2)(ii) <input type="checkbox"/> 50.36(c)(1)(ii)(A) <input type="checkbox"/> 50.73(a)(2)(iv)(A) <input type="checkbox"/> 50.73(a)(2)(x) <input type="checkbox"/> 20.2203(a)(2)(iii) <input type="checkbox"/> 50.36(c)(2) <input type="checkbox"/> 50.73(a)(2)(v)(A) <input type="checkbox"/> 73.71(a)(4) <input type="checkbox"/> 20.2203(a)(2)(iv) <input type="checkbox"/> 50.46(a)(3)(ii) <input type="checkbox"/> 50.73(a)(2)(v)(B) <input type="checkbox"/> 73.71(a)(5) <input type="checkbox"/> 20.2203(a)(2)(v) <input type="checkbox"/> 50.73(a)(2)(i)(A) <input type="checkbox"/> 50.73(a)(2)(v)(C) <input type="checkbox"/> OTHER <input type="checkbox"/> 20.2203(a)(2)(vi) <input checked="" type="checkbox"/> 50.73(a)(2)(i)(B) <input type="checkbox"/> 50.73(a)(2)(v)(D) <input type="checkbox"/> OTHER <div style="text-align: right; font-size: small;">Specify in Abstract below or in NRC Form 366A</div>

12. LICENSEE CONTACT FOR THIS LER

FACILITY NAME Bill Trafton, Operations Director	TELEPHONE NUMBER <i>(Include Area Code)</i> 815-415-2200
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13. COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT

CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANU-FACTURER	REPORTABLE TO EPIX

14. SUPPLEMENTAL REPORT EXPECTED <input type="checkbox"/> YES <i>(If yes, complete 15. EXPECTED SUBMISSION DATE)</i> <input checked="" type="checkbox"/> NO	15. EXPECTED SUBMISSION DATE	MONTH	DAY	YEAR

ABSTRACT *(Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines)*

On April 22, 2013, Unit 1 was in Mode 2, Startup. At approximately 0723 hours CDT, reactor pressure was increased above 150 psig with the Reactor Core Isolation Cooling (RCIC) system isolated and inoperable. Technical Specification (TS) LCO 3.5.3 requires RCIC to be operable in Mode 1, and in Modes 2 and 3 with reactor steam dome pressure greater than 150 psig. TS Required Actions (RA) A.1 and A.2 were entered.

At approximately 0815 hours, it was recognized that increasing reactor pressure greater than 150 psig with RCIC isolated and inoperable was a violation of TS LCO 3.0.4, which requires that entry into another Mode with an LCO not met shall only be made when the associated actions allow continued operation for an unlimited period of time. This was not the case for TS 3.5.3, and thus TS LCO 3.0.4 was not satisfied. Control rod withdrawals were stopped, and control rod insertion commenced in order to reduce reactor pressure and bring the Unit back into compliance with TS. At 0900 hours RCIC was declared operable, and control rod insertion was stopped.

The cause of this event was a weakness in Normal Unit Startup procedure LGP-1-1 in that there was no specific procedure step to verify RCIC operability prior to exceeding 150 psig. LGP-1-1 does contain a limitation stating that RCIC operability is required prior to exceeding 150 psig but this would have been more effective if it had been a specific procedural step. In addition, there was a lack of recognition of the impact of TS LCO 3.0.4 by Operations with respect to RCIC operability. Corrective actions include reviewing the event with all licensed operators and revisions to the Normal Unit Startup procedure LGP-1-1.

**LICENSEE EVENT REPORT (LER)
CONTINUATION SHEET**

1. FACILITY NAME	2. DOCKET	6. LER NUMBER			3. PAGE
LaSalle County Station, Unit 1	05000373	YEAR	SEQUENTIAL NUMBER	REV NO.	2 OF 3
		2013	- 004	- 00	

NARRATIVE

LaSalle County Station Units 1 is a General Electric Company Boiling Water Reactor with 3546 Megawatts Rated Core Thermal Power.

A. CONDITION PRIOR TO EVENT:

Unit(s): 1	Event Date: April 22, 2013	Event Time: 0723 CDT
Reactor Mode(s): 2	Mode(s) Name: Startup	Power Level: 0%

B. DESCRIPTION OF EVENT:

On April 22, 2013, Unit 1 was in Mode 2, Startup. At approximately 0723 hours CDT, reactor pressure was increased above 150 psig with the Reactor Core Isolation Cooling (RCIC)[BN] system isolated and inoperable. Technical Specification (TS) LCO 3.5.3 requires RCIC to be operable in Mode 1, and in Modes 2 and 3 with reactor steam dome pressure greater than 150 psig. TS Required Actions (RA) A.1 and A.2 were entered and preparations were begun to warm RCIC and place it in service.

At approximately 0815 hours, it was recognized that increasing reactor pressure greater than 150 psig with RCIC isolated and inoperable was a violation of TS LCO 3.0.4, which requires that entry into another Mode with an LCO not met shall only be made when the associated actions allow continued operation for an unlimited period of time. This was not the case for TS 3.5.3, and thus TS LCO 3.0.4 was not satisfied. Control rod withdrawals were stopped, and control rod insertion commenced in order to reduce reactor pressure and bring the Unit back into compliance with TS. At 0900 hours RCIC was declared operable, and control rod insertion was stopped.

This event is reportable under 10 CFR 50.73(a)(2)(i)(B) as an event or condition prohibited by the plant's Technical Specifications.

C. CAUSE OF EVENT:

The cause of this event was a weakness in Normal Unit Startup procedure LGP-1-1 in that there was no specific procedure step to verify RCIC operability prior to exceeding 150 psig. LGP-1-1 does contain a limitation stating that RCIC operability is required prior to exceeding 150 psig but this would have been more effective if it had been a specific procedural step. In addition, there was a lack of recognition of the impact of TS LCO 3.0.4 by Operations with respect to RCIC operability.

D. SAFETY ANALYSIS:

The safety significance of this event was minimal. High Pressure Core Spray (HPCS)[HG] was verified operable when reactor pressure exceeded 150 psig at 0723 hours in accordance with TS 3.5.3 RA A.1. RCIC was inoperable with reactor pressure in excess of 150 psig for approximately 97 minutes, which is significantly less than the TS 3.5.3 RA A.2 Completion Time of 14 days.

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NARRATIVE

E. CORRECTIVE ACTIONS:

- LGP-1-1 was revised to add sign offs to verify TS compliance prior to changing Modes or entering specified Conditions.
- This event will be reviewed with all licensed operators to reinforce the requirements of TS LCO 3.0.4 when changing Modes or entering specified conditions.

F. PREVIOUS OCCURRENCES:

A review found no previous occurrences in the past ten years of violating TS 3.0.4 by exceeding 150 psig reactor pressure with RCIC inoperable.

G. COMPONENT FAILURE DATA:

There were no component failures associated with this event.