April 1, 2013

U.S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555-0001 Serial No. 13-199 SPS Lic JCP Docket Nos. 50-280 50-281 License Nos. DPR-32 DPR-37

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Gentlemen:

VIRGINIA ELECTRIC AND POWER COMPANY SURRY POWER STATION UNITS 1 AND 2 ANNUAL CHANGES, TESTS, AND EXPERIMENTS REPORT REGULATORY COMMITMENT EVALUATION REPORT

Virginia Electric and Power Company submits the annual report of Changes, Tests, and Experiments pursuant to 10 CFR 50.59(d)(2) and Regulatory Commitment Changes identified in Commitment Evaluation Summaries implemented at Surry Power Station during 2012. Attachment 1 provides a description and summary of the Regulatory Evaluations. There were no Regulatory Commitment Changes in 2012.

Should you have any questions regarding this report, please do not hesitate to contact me at (757) 365-2003.

Very/truly yours,

Douglas C. Lawrence, Direction Station Safety & Licensing Surry Power Station

Attachment

Commitments made in this letter: None.

 cc: United States Nuclear Regulatory Commission, Region II Marquis One Tower, Suite 1200
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> NRC Senior Resident Inspector Surry Power Station

IE47 NRC

Serial No. 13-199 10 CFR 50.59 Report Page 1 of 2

Attachment 1

Surry Units 1 & 2

2012 - 10 CFR 50.59 Changes, Tests and Experiments

12-001 Regulatory Evaluation

05/08/12

<u>Description:</u> Regulatory Evaluation 12-001 reviewed reconfiguring the Reactor Coolant Pump (RCP) stator oil collection trays to control the flow of water/effluent to the Reactor Building Sump.

<u>Summary:</u> During an investigation into the cause of water leakage from the Unit 2 RCP Oil Collection Tanks, it was discovered that condensation forming on the RCP motor stator coolers was being collected by the newly installed RCP Stator Oil Collection Trays. This water flow eventually filled and overflowed the RCP oil collection tanks. As a result, the stator oil collection trays will be reconfigured to a housekeeping device which will divert the flow of condensation to the Control Rod Drive Mechanism Fan Cooling Coil drain lines. This will provide a flow path to the containment sump for the collected condensation in lieu of the existing floor drains in the loop room as existed before the tray modification.

The review determined the fire hazard evaluations for the reactor containment building as detailed in the Fire protection Safety Evaluation Report would not be altered by this modification and would bound the modification. Therefore, the evaluation concluded the modification could be implemented without prior NRC review and approval.

12-002 Regulatory Evaluation

05/10/12

<u>Description</u>: Regulatory Evaluation 12-002 reviewed the Surry Unit 1 reactor vessel closure head (RVCH) assessment performed in support of the implementation of Westinghouse 15x15 Upgrade Fuel design in Surry 1 Cycle 25.

<u>Summary:</u> This 10CFR50.59 Evaluation was performed for the assessment of the Surry Unit 1 RVCH for introduction of 15x15 Upgrade Fuel design at Surry Power Station. The estimated stresses for several components increased, which reduces the amount of margin to the allowable values of the ASME Boiler and Pressure Vessel Code. Part II-1 through Part II-6 of the 10CFR50.59 Evaluation confirmed the assessment of the RVCH has no impact on the associated design functions. All design basis limit fission product barriers associated with the reactor coolant system stress boundaries were shown to be met and there was no change in the method of evaluation. Therefore, the assessment of the Surry Unit 1 RVCH could be implemented without prior NRC review and approval.

Serial No. 13-199 10 CFR 50.59 Report Page 2 of 2

Attachment 1

Surry Units 1 & 2

2012 - 10 CFR 50.59 Changes, Tests and Experiments

12-003, Rev. 1 Regulatory Evaluation

09/06/12

<u>Description:</u> Regulatory Evaluation 12-003, Rev. 1 reviewed the functionality of a HI-HI Consequence Limiting Safeguards (CLS) relay with a non-functional unlatch coil.

<u>Summary:</u> The Unit 1 Train B HI-HI CLS relay is energized and latched from automatic signals to initiate the HI-HI CLS functions. The unlatching of the relay is normally done manually from a pushbutton in the Main Control Room (MCR). The unlatch coil in the relay is not functional and will not unlatch the relay from the MCR pushbutton. Testing has demonstrated the relay can be consistently and safely unlatched locally at the relay. An operability determination was developed to document the acceptability of the use of local reset.

The safety related function of the relay as described in the UFSAR is to automatically initiate HI-HI CLS. This accident mitigation function is not adversely affected by the change in the CLS manual reset function. The HI-HI CLS function is an accident mitigation function and is not an accident initiator. There are no accidents associated with actual or inadvertent actuation of CLS. The initiation of HI-HI CLS and the subsequent automatic actions are not affected by changing the location of the HI-HI CLS reset function. Therefore, HI-HI-CLS system will function as required to mitigate any credited accidents. This change only affects the ability to reset HI-HI CLS. Therefore, the operability determination could be implemented without prior NRC review and approval.