



International Agreement Report

Transient Analysis of the Research Reactor MARIA MC Fuel Elements Using RELAP5 Mod 3.3

Prepared by:
M. Dabrowski, P. Domitr, E. Staron

Panstwowa Agencja Atomistyki
(National Atomic Energy Agency)
Krucza 36
00-522 Warszawa, POLAND

A. Calvo, NRC Project Manager

**Office of Nuclear Regulatory Research
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001**

Manuscript Completed: November 2012
Date Published: March 2013

Prepared as part of
The Agreement on Research Participation and Technical Exchange
Under the Thermal-Hydraulic Code Applications and Maintenance Program (CAMP)

Published by
U.S. Nuclear Regulatory Commission

AVAILABILITY OF REFERENCE MATERIALS IN NRC PUBLICATIONS

NRC Reference Material

As of November 1999, you may electronically access NUREG-series publications and other NRC records at NRC's Public Electronic Reading Room at <http://www.nrc.gov/reading-rm.html>. Publicly released records include, to name a few, NUREG-series publications; *Federal Register* notices; applicant, licensee, and vendor documents and correspondence; NRC correspondence and internal memoranda; bulletins and information notices; inspection and investigative reports; licensee event reports; and Commission papers and their attachments.

NRC publications in the NUREG series, NRC regulations, and Title 10, "Energy," in the *Code of Federal Regulations* may also be purchased from one of these two sources.

1. The Superintendent of Documents
U.S. Government Printing Office
Mail Stop SSOP
Washington, DC 20402-0001
Internet: bookstore.gpo.gov
Telephone: 202-512-1800
Fax: 202-512-2250
2. The National Technical Information Service
Springfield, VA 22161-0002
www.ntis.gov
1-800-553-6847 or, locally, 703-605-6000

A single copy of each NRC draft report for comment is available free, to the extent of supply, upon written request as follows:

Address: U.S. Nuclear Regulatory Commission
Office of Administration
Publications Branch
Washington, DC 20555-0001
E-mail: DISTRIBUTION.RESOURCE@NRC.GOV
Facsimile: 301-415-2289

Some publications in the NUREG series that are posted at NRC's Web site address <http://www.nrc.gov/reading-rm/doc-collections/nuregs> are updated periodically and may differ from the last printed version. Although references to material found on a Web site bear the date the material was accessed, the material available on the date cited may subsequently be removed from the site.

Non-NRC Reference Material

Documents available from public and special technical libraries include all open literature items, such as books, journal articles, transactions, *Federal Register* notices, Federal and State legislation, and congressional reports. Such documents as theses, dissertations, foreign reports and translations, and non-NRC conference proceedings may be purchased from their sponsoring organization.

Copies of industry codes and standards used in a substantive manner in the NRC regulatory process are maintained at—

The NRC Technical Library
Two White Flint North
11545 Rockville Pike
Rockville, MD 20852-2738

These standards are available in the library for reference use by the public. Codes and standards are usually copyrighted and may be purchased from the originating organization or, if they are American National Standards, from—

American National Standards Institute
11 West 42nd Street
New York, NY 10036-8002
www.ansi.org
212-642-4900

Legally binding regulatory requirements are stated only in laws; NRC regulations; licenses, including technical specifications; or orders, not in NUREG-series publications. The views expressed in contractor-prepared publications in this series are not necessarily those of the NRC.

The NUREG series comprises (1) technical and administrative reports and books prepared by the staff (NUREG-XXXX) or agency contractors (NUREG/CR-XXXX), (2) proceedings of conferences (NUREG/CP-XXXX), (3) reports resulting from international agreements (NUREG/IA-XXXX), (4) brochures (NUREG/BR-XXXX), and (5) compilations of legal decisions and orders of the Commission and Atomic and Safety Licensing Boards and of Directors' decisions under Section 2.206 of NRC's regulations (NUREG-0750).

DISCLAIMER: This report was prepared under an international cooperative agreement for the exchange of technical information. Neither the U.S. Government nor any agency thereof, nor any employee, makes any warranty, expressed or implied, or assumes any legal liability or responsibility for any third party's use, or the results of such use, of any information, apparatus, product or process disclosed in this publication, or represents that its use by such third party would not infringe privately owned rights.



International Agreement Report

Transient Analysis of the Research Reactor MARIA MC Fuel Elements Using RELAP5 Mod 3.3

Prepared by:
M. Dabrowski, P. Domitr, E. Staron

Panstwowa Agencja Atomistyki
(National Atomic Energy Agency)
Krucza 36
00-522 Warszawa, POLAND

A. Calvo, NRC Project Manager

**Office of Nuclear Regulatory Research
U.S. Nuclear Regulatory Commission
Washington, DC 20555-0001**

Manuscript Completed: November 2012
Date Published: March 2013

Prepared as part of
The Agreement on Research Participation and Technical Exchange
Under the Thermal-Hydraulic Code Applications and Maintenance Program (CAMP)

Published by
U.S. Nuclear Regulatory Commission

ABSTRACT

The new fuel type in the Research Reactor ‘Maria’ is undergoing a test procedure. Thermal-hydraulic calculations using RELAP have been applied in order to cross-check the results obtained using a specialized thermal-hydraulic ‘SN’ code. Nodalization of the fuel element has been developed and calculations have been performed. LOFA and RIA cases have been analysed and compared showing good agreement between the two codes.

CONTENTS

	<u>Page</u>
ABSTRACT	iii
FIGURES.....	vii
TABLES	viii
EXECUTIVE SUMMARY.....	ix
ABBREVIATIONS	xiii
1 INTRODUCTION	1-1
2 FACILITY DESCRIPTION.....	2-1
2.1 MARIA research reactor technical information	2-1
2.2 Fuel channel with fuel element technical information:.....	2-2
3 CODE INPUT AND MODEL DESCRIPTION	3-1
3.1 Nodalization.....	3-1
3.2 Heat structures.....	3-2
3.3 Limitation of the model and errors	3-3
4 RESULTS	4-1
4.1 Steady state evaluation	4-1
4.2 LOFA - Loss of flow in the fuel channel cooling system.....	4-6
4.3 Slow reactivity insertion accident on low power level	4-10
4.4 Fast reactivity insertion accident on low power level	4-13
4.5 Slow reactivity insertion accident during normal operation of reactor.....	4-15
4.6 Fast reactivity insertion accident during reactor normal operation	4-19
5 RUN STATISTICS.....	5-1
6 CONCLUSIONS	6-1
7 REFERENCES	7-1
Appendix A – Technical information on the Research Reactor MARIA	A-1
Appendix B – Input file.	B-1

FIGURES

	<u>Page</u>
Figure 1. Simplified scheme of the Research Reactor 'MARIA' core configuration.....	2-1
Figure 2. Simplified scheme of the fuel channel with an MC fuel element	2-2
Figure 3. Nodalization scheme of the MC fuel element.	3-1
Figure 4. Positioning of mesh points in each fuel pipe – grey colour clad material, red colour - fuel.....	3-2
Figure 5. Placement of measurement points.....	3-2
Figure 6. Temperature distribution on ICIP of each fuel pipe	4-2
Figure 7. Temperature distribution on ECIP of each fuel pipe	4-2
Figure 8. Temperature distribution on ICEP of each fuel pipe	4-3
Figure 9. Temperature distribution on ECEP of each fuel pipe.....	4-3
Figure 10. Water temperature distribution in gaps between fuel pipes. In – filler gap	4-4
Figure 11. Maximum temperature axial distribution in fuel pipe no. 6. Yellow dots – calculation points in fuel pipe, red – fuel, blue – cladding.....	4-4
Figure 12. Heat flux distribution on inner side of fuel pipes.	4-5
Figure 13. Heat flux distribution on outer side of fuel pipes	4-5
Figure 14. Cladding and water temperature changes. SN code: water and fuel element outlet temperature, RELAP5: water I – fuel element outlet temperature, water II - fuel channel outlet temperature	4-8
Figure 15. Cladding and water temperature changes. SN code: water and fuel element outlet temperature, RELAP5: water I – fuel element outlet temperature, water II - fuel channel outlet temperature. Wider scale range.	4-8
Figure 16. ONBR parameter changes.....	4-9
Figure 17. Reactivity changes. RELAP5: total reactivity change; water and fuel: reactivity change due to coolant and fuel temperature reactivity coefficients.....	4-12
Figure 18. Power changes of fuel element for SRIA on low power level.....	4-12
Figure 19. Power changes of fuel element for FRIA on low power level.....	4-14
Figure 20. ONBR parameter changes.....	4-14
Figure 21. Reactivity changes - RELAP5 calculations: i) change of reactivity: total reactivity change, ii) water: change due to coolant temperature reactivity coefficients, iii) fuel: change due to fuel temperature reactivity coefficient.....	4-15

Figure 22. Power changes of fuel element for SRIA on nominal power level.....	4-17
Figure 23. Cladding and water temperature changes. RELAP5 and SN codes: cladding – cladding temperature, water – fuel element outlet temperature.....	4-17
Figure 24. ONBR parameter changes.....	4-18
Figure 25. Reactivity changes. RELAP5 and SN codes: change of reactivity - total re-activity change, water - change due to coolant temperature reactivity coefficients, fuel - change due to fuel temperature reactivity coefficients.....	4-18
Figure 26. Power changes of fuel element for FRIA on nominal power level.....	4-20
Figure 27. ONBR parameter changes.....	4-21
Figure 28. Cladding and water temperature changes. RELAP5 and SN codes: cladding – cladding temperature, water I – fuel element outlet temperature, water II - fuel channel outlet temperature	4-21
Figure 29. Reactivity changes. RELAP5 and SN codes: change of reactivity - total reactivity change, water - change due to coolant temperature reactivity coefficients, fuel - change due to fuel temperature reactivity coefficients.....	4-22
Figure 30. Calculation time for various cases.....	5-1

TABLES

	<u>Page</u>
Table 1. Distribution of heat generation on fuel pipes and comparison of heat generation with content of uranium	3-3
Table 2. Comparison of results for two scenarios	4-6
Table 3. Sequence of events for LOFA.....	4-7
Table 4. Distribution of calculated top temperatures on fuel pipes	4-7
Table 5. Comparison of results for second temperature peak	4-10
Table 6. Comparison of results for SRIA on low power level.....	4-11
Table 7. Comparison of results for FRIA on low power level.....	4-13
Table 8. Comparison of results for SRIA during normal operation	4-16
Table 9. Comparison results for FRIA during normal operation.....	4-19

EXECUTIVE SUMMARY

In accordance with the international agreement on GTRI (Global Threat Reduction Initiative) Poland agreed to change the fuel type in the MARIA research reactor in Świerk from Highly Enriched Uranium (HEU) fuel to Low Enriched Uranium fuel (LEU). The new fuel MC5-485 – in short MC fuel - consists of 19,75% of 235-U. At this stage several new MC elements have been inserted into the reactor core and are under a test procedure.

Prior to this test, thermal-hydraulic effects have been calculated by the operator using a specialized computer code called 'SN'. The SN code is capable of performing thermal-hydraulic calculations of the fuel elements behaviour and has been verified and validated for the research reactor MARIA only.

The Polish National Atomic Energy Agency (PAA) has decided to try to cross-check the results of the analysis using the RELAP5 code although it is aware that the results of the analysis at this stage might need further elaboration and additional calculations. However an important aim is to gain experience in performing analysis using RELAP5 and to judge whether it is applicable and to what extent to the analysis of the research reactor.

The National Centre for Nuclear Research who is the operator of the research reactor MARIA has provided the results of two types of accidents: a Reactivity Initiated Accident (RIA) and Loss of Flow Accident (LOFA). The analysis of RIA was divided into four scenarios: two for reactor low power level operation (possible during start-up of the reactor or during work with low power level – 10 kW) and two for normal operation with maximum allowed power. Both cases were performed for slow and fast reactivity insertion. The SN code models the fuel element only. It does not include the possibility to model the whole reactor together with the safety systems. As a consequence of this limitation no analyses of the Loss Of Coolant Accident (LOCA) have been supplied as yet. Using RELAP should change this and enable the regulatory body to perform a LOCA analysis as well.

The Regulatory Body has cross checked the analyses made and presented by the operator and has provided the comparison of results in this report. Due to other limitations of the SN code, the comparison is rather limited. Only a small number of parameters is included in the SN output. Thus, only those could be compared.

The presented results of calculations do not include errors and sensitivity studies. However the scenario analysed for the purpose of this document was made with conservative assumptions. The model used for this calculation was limited to a model of the fuel element only. Other elements of the reactor systems were not included. As a consequence it was necessary to:

- omit influence of reactivity coefficients of the coolant in reactor pool and beryllium blocks positioned around the analysed fuel channel; the omitted thermal reactivity coefficients are positive but during postulated accidents, the increase of the temperature of the beryllium blocks and water of pool is very low,
- omit heat transfer between the fuel channel and water of the reactor pool; the influence of this factor is very small due to the very small difference in temperatures between

water in the outer part of the fuel channel and the water in the pool (i.e. heated water flows in the internal part of fuel channel).

Following an introduction in chapter 1, the report covers a short description of the Research Reactor ‘Maria’ including the fuel channel and the fuel element presented in chapter 2. Additional information concerning the RR Maria is supplied in Appendix A.

The nodalization model is presented in chapter 3 and includes a description of the fuel channel and heat structures. The input data file is attached in Appendix B.

The results of the calculations are presented in chapter 4 mostly on figures. They begin with the steady state evaluation which is followed by a loss of flow in the fuel channel analysis. Further analyses cover the slow reactivity insertion accident on low power level, fast reactivity insertion accident on low power level, slow reactivity insertion accident during reactor normal operation and finally fast reactivity insertion accident during reactor normal operation.

In chapter 5 a brief information on run statistics is presented.

Finally a summary of the results is presented in chapter 6.

As already mentioned, initial calculations were performed for steady-state conditions. The newly created model of the reactor with the new fuel elements for RELAP was used and compared with the results of steady state calculations received from the SN code. The limitations of the SN code does not allow to perform detailed comparisons however peak cladding temperatures and heat flux were compared.

The steady state was calculated for the reactor normal operation deviated from full power.

The conditions that were used in the simulation were chosen to simulate work of the fuel channel during normal operation with safety parameters equal to warning signals. In order to have such conditions the mass flow was reduced to 90% of nominal value and thermal power was increased to 110% of the nominal value.

The results received were satisfactory.

Loss of flow in the fuel channel cooling system is possible in case of the loss of power in the pump system. As a result a sudden decrease of pressure occurs in the cooling channel that leads to the initiation of the emergency signal and shutdown of the reactor. Furthermore, after the loss of flow a rapid increase of fuel clad temperature and cooling temperature in gaps between fuel pipes occurs. According to the scenario a few seconds after the loss of power in the pump system, the work of a backup pump should be initiated powered by accumulator batteries however operating at a lower speed – 1500 r/s. After the reactor shutdown and before the flow is stabilized another small temperature jump can be detected.

The highest outer clad temperature calculated with the RELAP5 code was 432K (outer layer temperature – 440K). The temperature calculated with the SN code was 433.4K. Both codes calculated the highest temperature in the same place – on the internal side of pipe number 6.

In case when the operator undertakes incorrect actions or a malfunction of the reactivity regulation system occurs, it is possible that as a result, positive reactivity is inserted into the core with the maximum reactivity speed 0.04\$/s. This scenario was checked for low power level conditions. This scenario has the lowest impact on the fuel elements. The temperature peaks of

water and clad are very small - smaller than fluctuations of temperature during normal operation of the reactor.

One of the important scenarios addressed in the supplementary document is the fast inflow of cold water into the core resulting in a sudden increase of reactivity. The negative reactivity coefficients lead to the increase of power in the fuel elements. The scenario analysis was performed for low power. The results received were satisfactory.

The slow reactivity insertion accident during normal operation was very similar to the slow reactivity insertion accident on low power. Also here the results were satisfactory.

A break of control rods is another scenario analysed. The break of control rods leads to fast insertion of positive reactivity into the reactor. It is assumed that the maximum insertion of positive reactivity is equal to 1.5\$ with the top speed of insertion 4\$/s.

The main differences in the results between RELAP and the SN code are due to the different models of reactor kinetics. RELAP5 uses the point kinetic model and this model was applied to the RIA analysis. In case of the SN code a more pessimistic model was used.

Summarizing the analysed cases it can be stated that most of the results of the simulation for the research reactor MARIA obtained using the RELAP5 code, showed that there are no big differences between the results achieved by the validated SN and the RELAP5 codes. Only in one case (RIA with fast reactivity insertion with maximum power level) there is the opportunity to exceed the lowest allowed level of the ONBR parameter. Exceeding this parameter can cause the onset of boiling of the coolant in the gap between the fuel pipes. In the analysed case, the duration of time when ONBR is lower than 1.2 is very short (about 0.35s) and should not cause any damage to the fuel and clad material. In the fast RIA scenario, the power jump during the insertion of positive reactivity was stopped by the negative reactivity coefficients (fuel and coolant) before insertion of the safety and control rods into the core could be initiated.

In other simulated scenarios, no opportunity to exceed the ONBR parameter or temperature limit of clad – 452K occurred.

In the analysed LOFA scenario, the calculations showed that enough cooling capability is provided even when only one pump is working with a lower rotation speed than the normally required two pumps.

The content of this report does not cover all possible accident conditions that should be analysed for the research reactor. Further analysis will be necessary. A comparison of the SN and RELAP5 codes showed that RELAP5 can be used in the safety analysis for the research reactor MARIA.

ABBREVIATIONS

ECEP	external side of clad on the external side of pipe
ECIP	external side of clad on the internal side of pipe
HEU	highly enriched Uranium
ICEP	internal side of clad on the external side of pipe
ICIP	internal side of clad on the internal side of pipe
LEU	Low enriched Uranium
LOFA	Loos of Flow Accident
NCBJ	Narodowe Centrum Badań Jądrowych (National Centre for Nuclear Research)
ONBR	Onset of Nucleate Boiling Ratio
PAA	Państwowa Agencja Atomistyki (National Atomic Energy Agency of Poland)
PAR	automatic regulation rod
PK	control rods
PB	safety rods
REP	moving type fuel elements
RIA	Reactivity Initiated Accident
RR	research reactor
SN	name of computer code used for RR ‘Maria’ calculations

1 INTRODUCTION

Fulfilling the international agreement on GTRI (Global Threat Reduction Initiative) Poland agreed to change the fuel type in the MARIA research reactor in Świerk from Highly Enriched Uranium (HEU) fuel to Low Enriched Uranium fuel (LEU). The new fuel MC5-485 consists of 19,75% 235-U. At this stage several new MC elements have been inserted into the reactor core and are under a test procedure.

Prior to this test, thermal-hydraulic effects have been calculated by the operator using a specialized computer code called 'SN'. The SN code is capable of performing thermal-hydraulic calculations of the fuel elements behaviour and has been verified and validated for the research reactor MARIA only.

The Polish National Atomic Energy Agency (PAA) has decided to try to cross-check the results of the analysis using the RELAP5 code although it is aware that the results of the analysis at this stage might need further elaboration and additional calculations. However an important aim is to gain experience in performing analysis using RELAP5 and to judge whether it is applicable and to what extent to the analysis of the research reactor.

The National Centre for Nuclear Research who is the operator of the research reactor MARIA has provided the results of two types of accidents: a Reactivity Initiated Accident (RIA) and Loss of Flow Accident (LOFA). The results of the analyses are part of a supplementary document¹ to the Safety Analysis Report. The analysis of RIA was divided into four scenarios: two for reactor low power level operation (possible during start-up of the reactor or during work with low power level – 10 kW) and two for normal operation with maximum allowed power. Both cases were performed for slow and fast reactivity insertion. The SN code models the fuel element only. It does not include the possibility to model the whole reactor together with the safety systems. As a consequence of this limitation no analyses of the Loss Of Coolant Accident (LOCA) have been supplied as yet. Using RELAP should change this and enable the regulatory body to perform a LOCA analysis as well.

The Regulatory Body has cross checked the analyses made and presented by the operator in the supplementary document and has provided the comparison of results in this report. Due to other limitations of the SN code, the comparison is rather limited. Only a small number of parameters is included in the SN output. Thus, only those could be compared.

¹ Aneks 2012/I do Eksplotacyjnego Raportu Bezpieczeństwa Reaktora MARIA Konwersja rdzenia reaktora MARIA na paliwo MC, July 2012.

2 FACILITY DESCRIPTION

2.1 MARIA research reactor technical information

The Maria research reactor is a pool type research reactor with 20-25 closed fuel channels in the core, with a beryllium moderator and graphite reflector. The designed maximum thermal power is 30 MW. The reactor is placed about 30 km from Warsaw in Świerk-Otwock and is operated by the National Centre for Nuclear Research (NCBJ). More technical information including a description of the primary safety systems is provided in Appendix A.

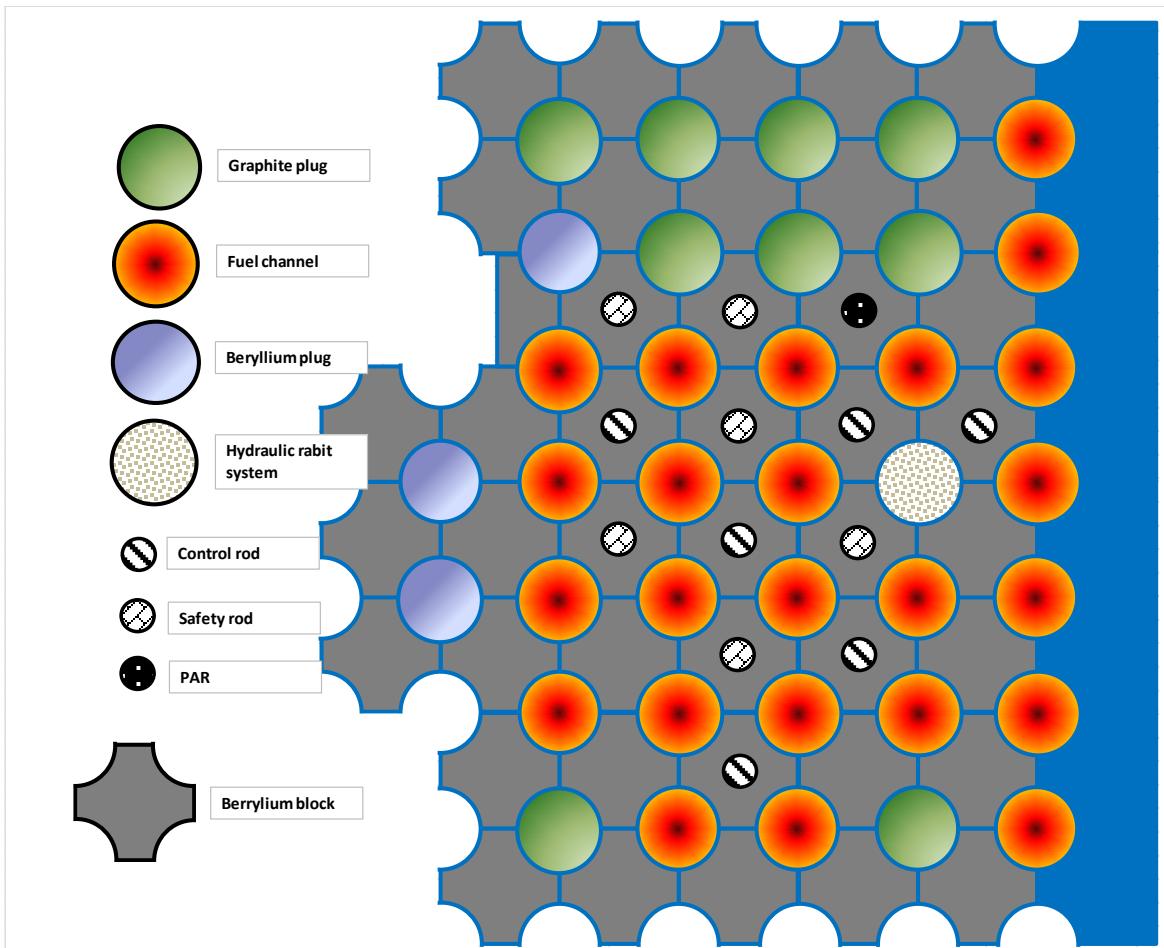


Figure 1. Simplified scheme of the Research Reactor 'MARIA' core configuration

2.2 Fuel channel with fuel element technical information:

This report assumes that only the new type of fuel – the French LEU fuel MC5-485 with 19.75% of 235U is used. The fuel element is a pipe in pipe construction, consisting of 5 pipes with fuel (Fig.2 with numbers from 2 to 6) and filler. The fuel is a dispersion U3Si2-Al with AG3NE alloy as a clad material and SAW alloy (aluminium alloy type) as an outer tube of the fuel channel. Water in the fuel element flows through 6 gaps (numbered from 1 to 6 on the picture) and through a gap in the centre of the fuel element. Fuel elements work with various thermal power levels depending on the position in the core. Usually, fuel elements work in the centre of the core with thermal power at the level of about 1.2-1.6 MW and in the peripheral position of the core with power about 0.3-1.2 MW. The maximum allowed thermal power – the safety limit for this fuel is 1.8 MW and it is forbidden for the reactor to exceed the limit in any conditions.

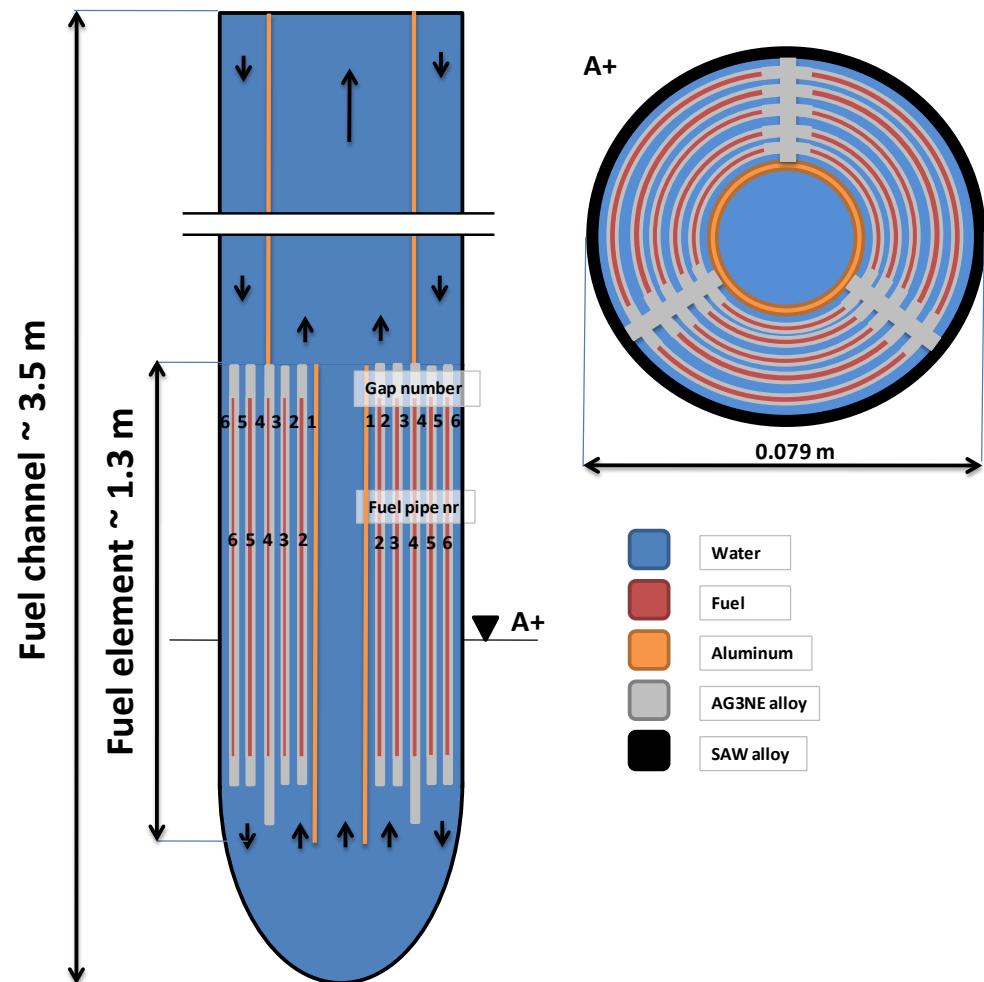


Figure 2. Simplified scheme of the fuel channel with an MC fuel element

3 CODE INPUT AND MODEL DESCRIPTION

3.1 Nodalization

In order to cross-check the analysis it was necessary to prepare a model of the fuel channel with a fuel element consisting of 5 fuel pipes and a bypass in centre. The nodalization is presented on Fig.3. Water flowing in the channel is directed first to the outer part of channel (component 10), then split into three gaps between fuel pipes (components 20, 25 and 30). Next, the water mixes and changes direction of flow (in component 35, 40 and 45) and goes up through the rest of the gaps and bypass (components 50, 55, 60 and 80). Next, the water flows to the exit of the channel through the inner part of the channel (component 70).

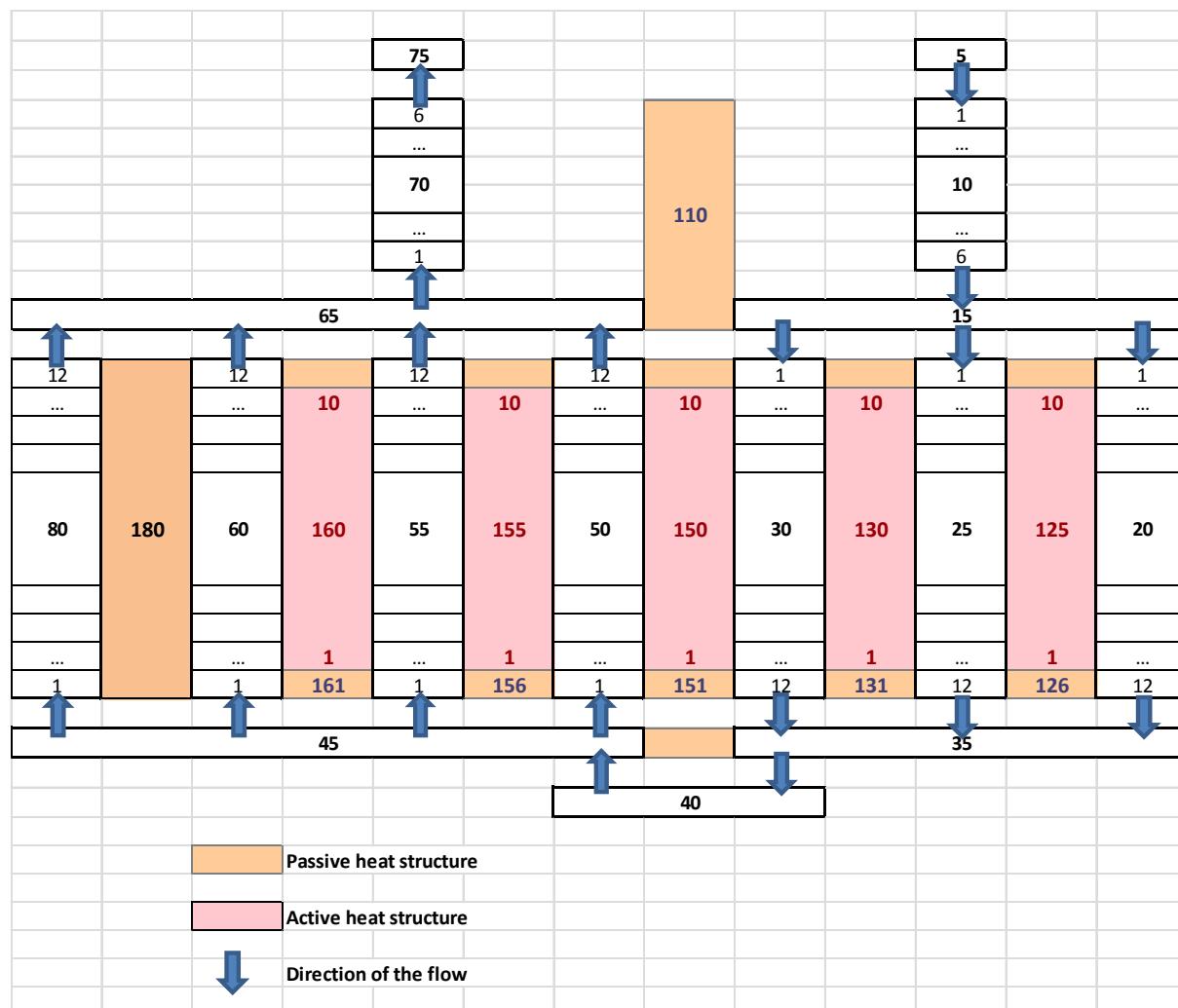


Figure 3. Nodalization scheme of the MC fuel element.

3.2 Heat structures

Two types of heat structures - active and passive were used for modelling heat exchange. Passive heat structures without heat generation are used for modelling heat transfer between the outer and inner part of the fuel channel (node no. 110 on Figure 3), non-filled fuel pipe parts (161, 156, 151, 131, 126) and filler (180). Active heat structures are used for modelling fuel pipes with heat generation (160, 155, 150, 130, 125 consisting of 10 cells). Each cell in the active heat structure was divided into 17 meshes as is visualised on Figure 4. In case of passive structures each cell was divided into less than 10 meshes.

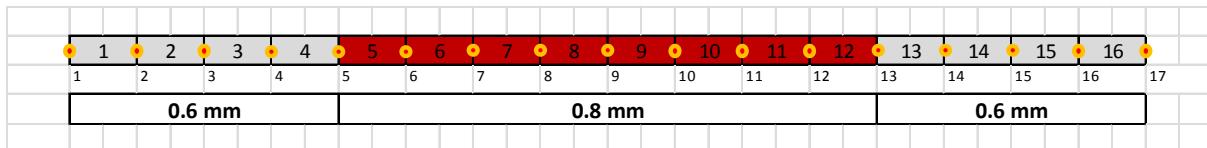


Figure 4. Positioning of mesh points in each fuel pipe – grey colour clad material, red colour - fuel

For the purpose of this document it was required to check the temperature in four points:
External side of Clad on the Internal side of Pipe (ECIP) – mesh nr 1,
Internal side of Clad on the Internal side of Pipe (ICIP) – mesh nr 5,
Internal side of Clad on the External side of Pipe (ICEP) – mesh nr 13,
External side of Clad on the External side of Pipe (ECEP) – mesh nr 17.
The placement of the four points is shown on Figure 5.

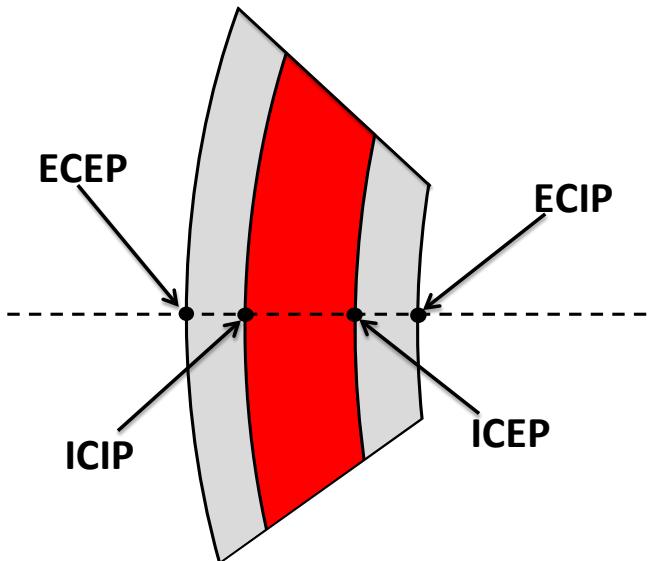


Figure 5. Placement of measurement points

The heat generation distribution model was based on two factors: (1) heat generation in each

pipe calculated by the computer code MCNP² and (2) axial heat generation based on the cosine model with a slight shift downwards in order to simulate partial insertion of control rods into the core during operation.

Table 1. Distribution of heat generation on fuel pipes and comparison of heat generation with content of uranium

Pipe	Heat generation [%]	Mass of U-235 [%]
2	11.0	11.15
3	14.1	15.58
4	18.1	20.00
5	23.8	24.43
6	33.0	28.84

3.3 Limitation of the model and errors

A model of the analysed objects, as well as data and calculations should always include an errors analysis that influence the final results. However the scenario analysed for the purpose of this document was made with conservative assumptions in the initial condition and did not include the influence of errors and sensitivity studies. The model used for this calculation was limited to a model of the fuel element only. Other elements of the reactor systems were not included. As a consequence it was necessary to:

- omit influence of reactivity coefficients of the coolant in reactor pool and beryllium blocks positioned around the analysed fuel channel; the omitted thermal reactivity coefficients are positive but during postulated accidents, the increase of the temperature of the beryllium blocks and water of pool is very low,
- omit heat transfer between the fuel channel and water of the reactor pool; the influence of this factor is very small due to the very small difference in temperatures between water in the outer part of the fuel channel and the water in the pool (i.e. heated water flows in the internal part of fuel channel).

2 Calculated by NCBJ in document: *Aneks 2009/I do Eksplotacyjnego Raportu Bezpieczeństwa Reaktora MARIA Badania paliwa MC*

4 RESULTS

4.1 Steady state evaluation

Initial calculations were performed for steady-state conditions. The newly created model of the reactor with the new fuel elements for RELAP was used and compared with the results of steady state calculations received from the SN code. The limitations of the SN code do not allow to perform detailed comparisons, however peak cladding temperatures and heat flux were compared.

The steady state was calculated for the reactor normal operation deviated from full power.

Steady-state conditions

The conditions that were used in the simulation were chosen to simulate work of the fuel channel during normal operation with safety parameters equal to warning signals. In order to have such conditions the mass flow was reduced to 90% of nominal value and thermal power was increased to 110% of the nominal value.

Initial conditions:

- Cooling channel inlet water temperature $T_{in}=318.15K$
- Mass flow $Q=7.43\text{kg/s}$ ($27\text{m}^3/\text{h}$ for density of water: 990kg/m^3) (90% of nominal mass flow)
- Pressure at the fuel channel inlet $p=1.7\text{MPa}$
- Fuel channel pressure drop $\Delta p=0.59\text{MPa}$
- Fuel element thermal power 1.98MW (110% of nominal value)

Results

Temperature distribution on fuel pipes

The results of calculations are presented on the following figures – from Figure 6 to Figure 13.

The temperature distribution on the internal side of the fuel pipe as below:

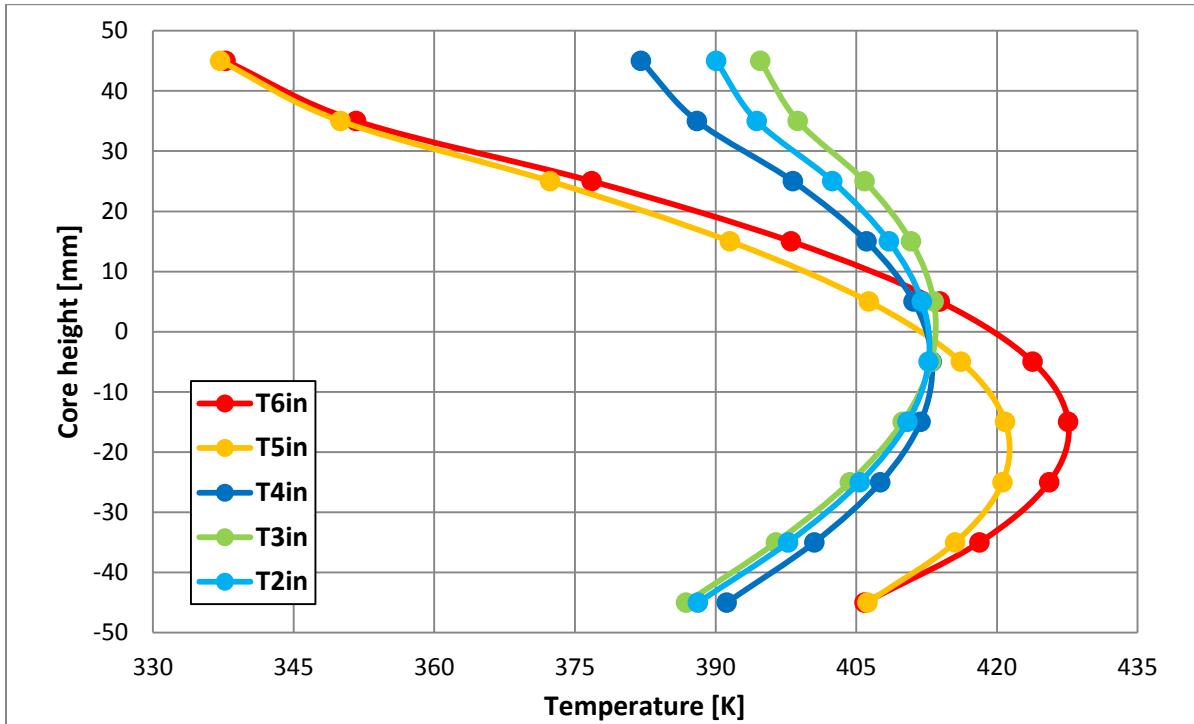


Figure 6. Temperature distribution on ICIP of each fuel pipe

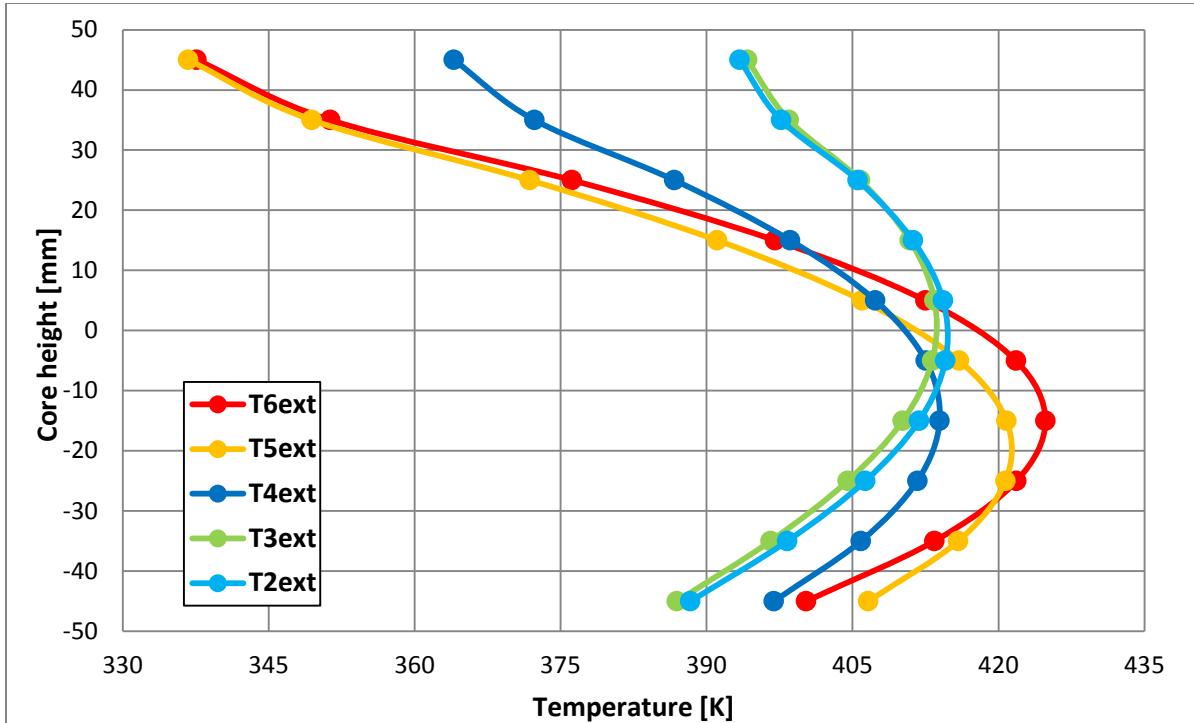


Figure 7. Temperature distribution on ECIP of each fuel pipe

On the outer side of fuel pipe as below:

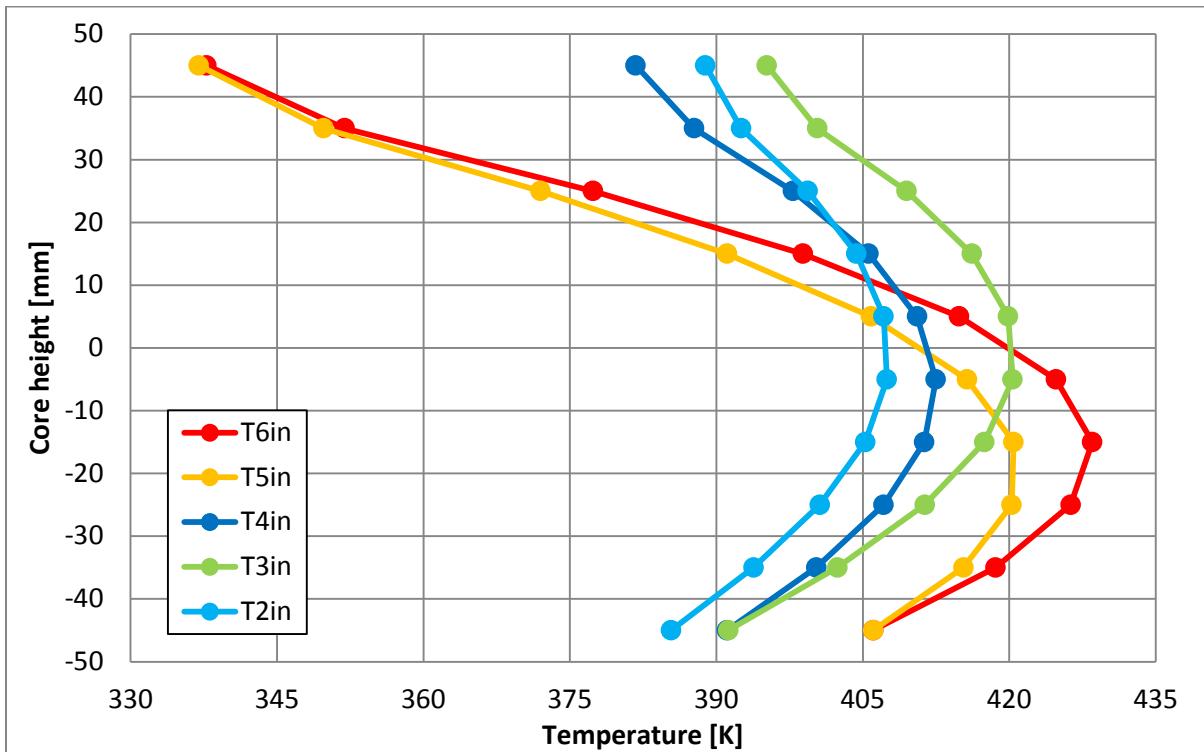


Figure 8. Temperature distribution on ICEP of each fuel pipe

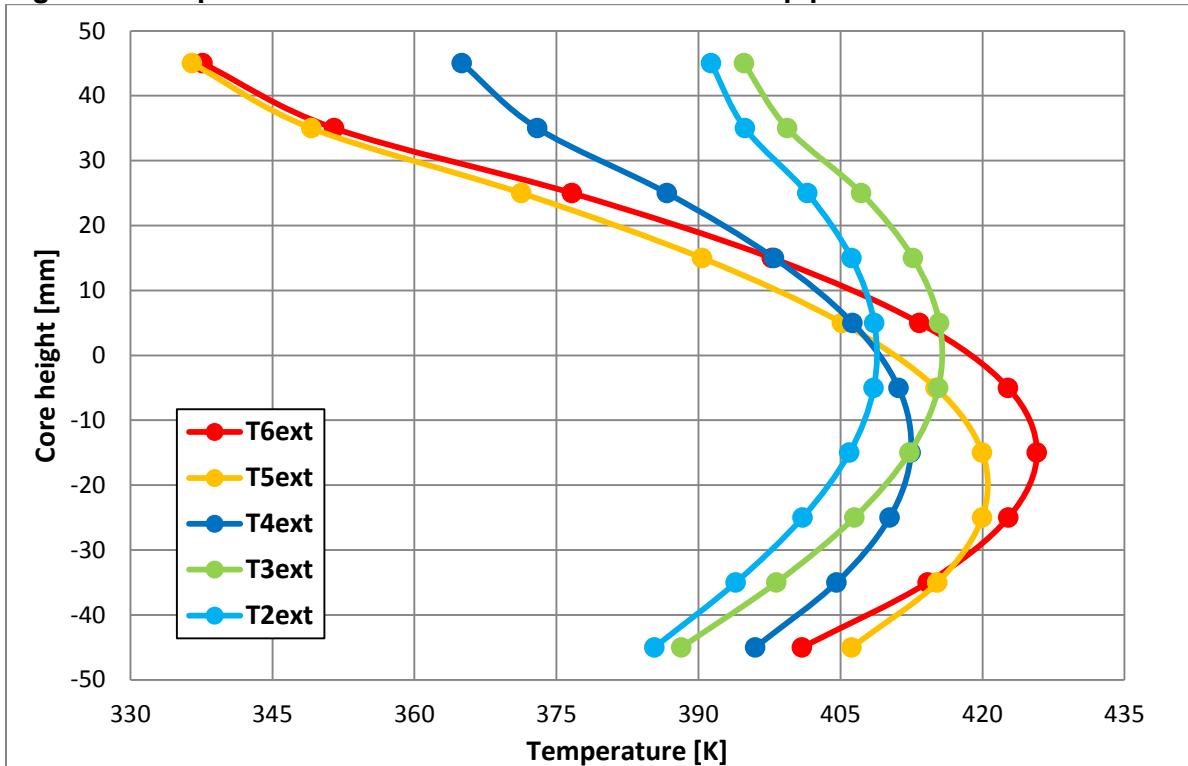


Figure 9. Temperature distribution on ECEP of each fuel pipe

Temperature distribution of water between pipes in gaps as below:

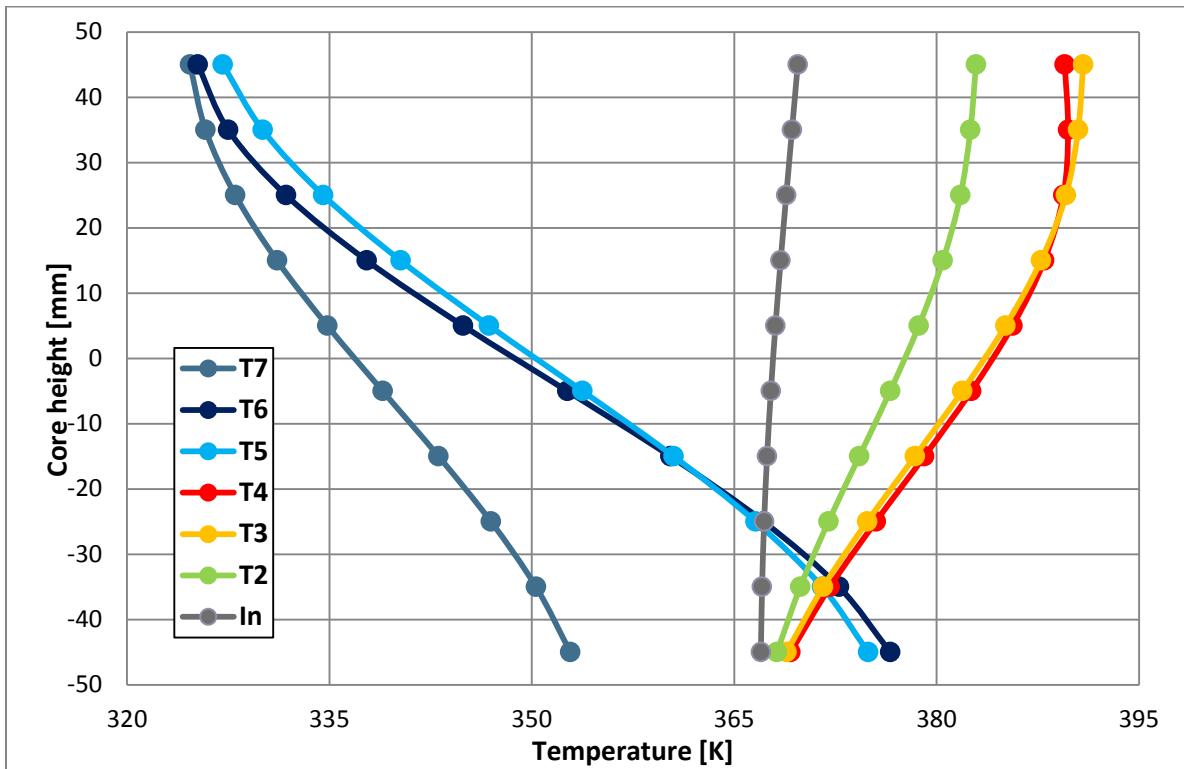


Figure 10. Water temperature distribution in gaps between fuel pipes. In – filler gap

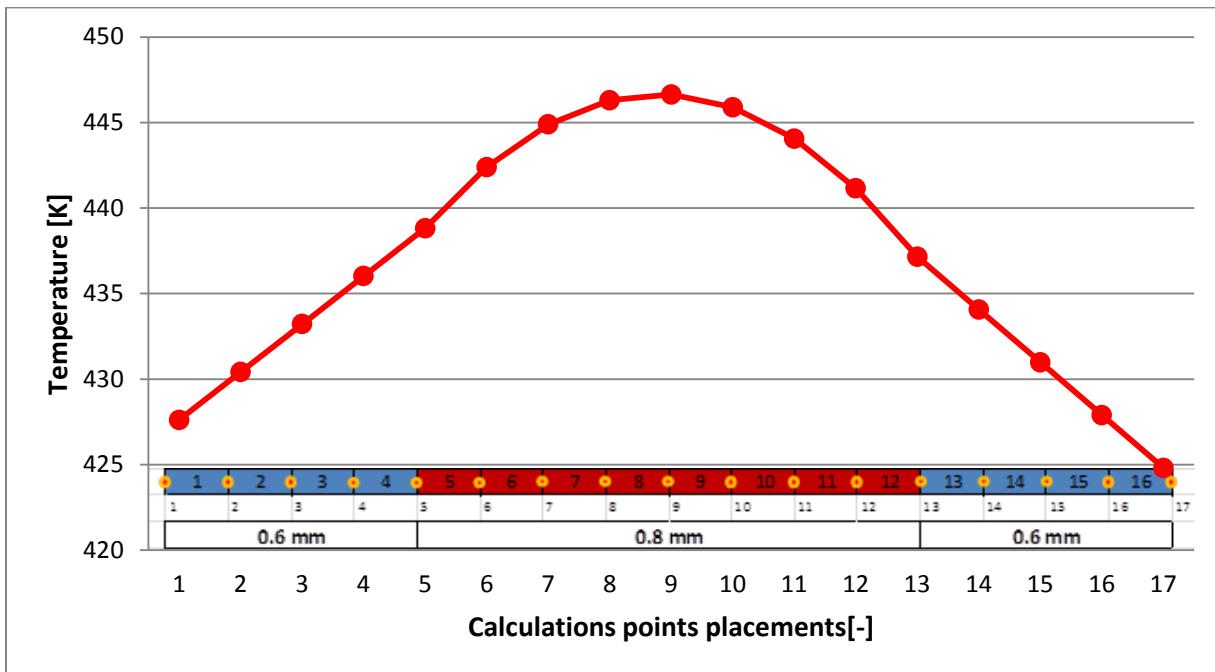


Figure 11. Maximum temperature axial distribution in fuel pipe no. 6. Yellow dots – calculation points in fuel pipe, red – fuel, blue – cladding

Heat flux distribution on outer side of fuel pipes

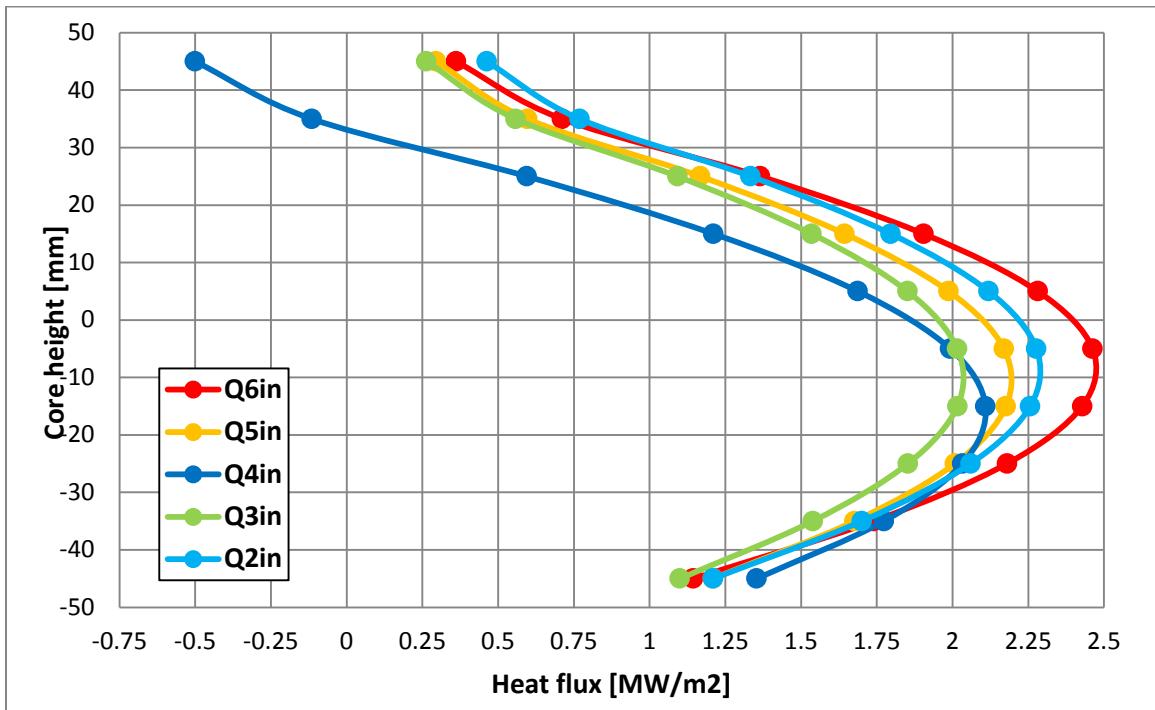


Figure 12. Heat flux distribution on inner side of fuel pipes.

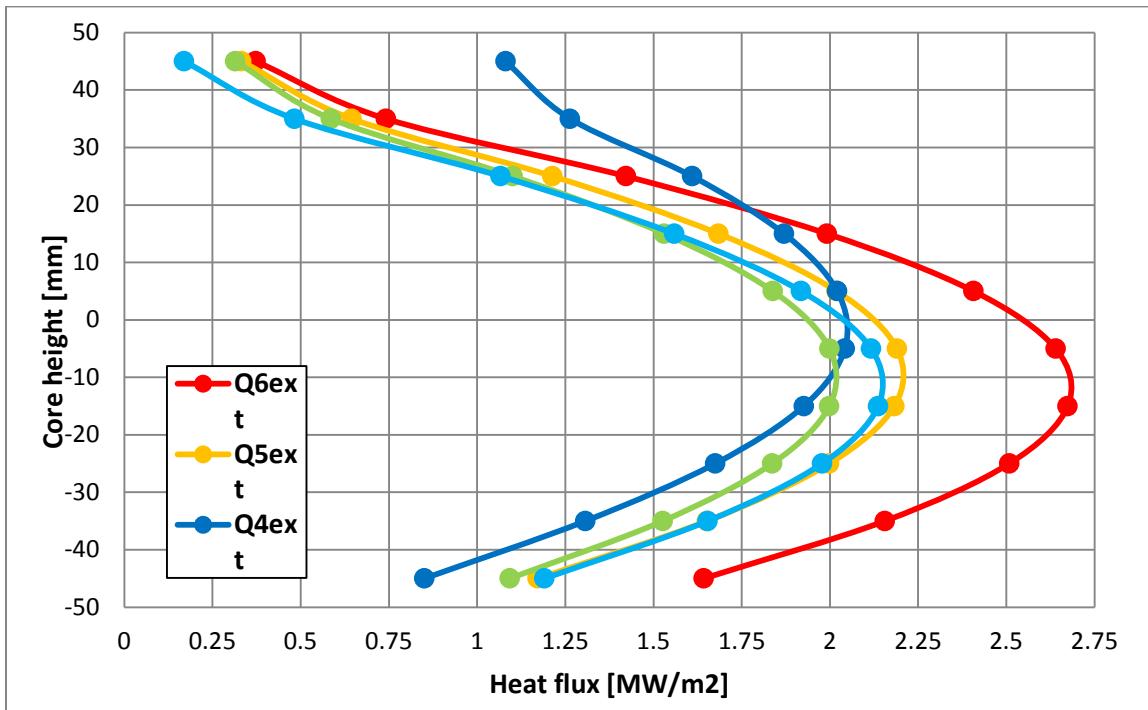


Figure 13. Heat flux distribution on outer side of fuel pipes

Comparison of results

A comparison of two calculations is presented in Table 2. The main results of the RELAP5 calculations with inlet temperature $T_{in}=318.15K$ presented in the chapter above are compared with the results received using the SN code. Additionally one more calculation using RELAP5 was performed for the same conditions except for the inlet temperature which was changed to $T_{in}=323.15K$

Table 2. Comparison of results for two scenarios

	RELAP5	RELAP5 ($T_{in}=323.1K$)	SN
Maximum peak clad temperature on ECIP of fuel pipe [K]	427.6	431.0	425.6
Maximum peak coolant temperature [K]	390.9	396.2	no data
Maximum heat flux [MW/m ²]	2.67	2.68	2.63
Maximum temperature in the fuel centre point [K]	446.6	450.0	no data

4.2 LOFA - Loss of flow in the fuel channel cooling system

Accident conditions

Loss of flow in the fuel channel cooling system is possible in case of the loss of power in the pump system. As a result a sudden decrease of pressure occurs in the cooling channel that leads to the initiation of the emergency signal and shutdown of the reactor. Furthermore, after the loss of flow a rapid increase of fuel clad temperature and cooling temperature in gaps between fuel pipes occurs. According to the scenario a few seconds after the loss of power in the pump system, the work of a backup pump should be initiated with the power supply from accumulator batteries and with lower speed – 1500 r/s. After the reactor shutdown and before the flow is stabilized another small temperature jump can be detected.

Initial and boundary conditions:

Initial conditions:

- Cooling channel Inlet water temperature $T_{in}=318.15K$
- Mass flow $Q=8.25\text{kg/s}$ ($30\text{m}^3/\text{h}$ for density of water: 990kg/m^3)
- Pressure at the fuel channel inlet $p_{in}=1.7\text{MPa}$
- Fuel channel pressure drop $\Delta p=0.59\text{MPa}$
- Fuel element thermal power 1.8MW

Boundary conditions:

- Alarm signal – signal from decrease pressure to 1.4MPa
- Signal delay time $t_1=0.2\text{s}$
- Effective time of safety and control rods insertion $t_2=0.15\text{s}$
- Inserted negative reactivity (from rods PK and PB) $\varrho=-5.5\$$ (-2\$ PK, -3.5\$ PB – with the

- assumption that the rod with the highest weight failed)
- Loss of flow modelled as in the Supplement³
 - Temperature reactivity coefficients: $\alpha_f = -0.25\phi/\text{deg}$, $\alpha_w = -2.02\phi/\text{deg}$

Results

The sequence of events:

Table 3. Sequence of events for LOFA

Time [s]	
0.00	Pump trip, loss of flow
0.40	Alarm signal p=1.4Mpa
0.75	Beginning of the negative reactivity insertion from PK and PB
0.78	Peak Clad Temperature on ICIP of fuel pipe nr 6 - 440.38 K
0.79	Peak Clad Temperature on ECIP of fuel pipe nr 6 - 432.07 K
0.89	Peak Coolant Temperature in the gap between fuel pipe nr 3 and nr 4 – 394.07 K
6.32	Second Peak Clad Temperature on ECIP of fuel pipe nr 6 with 2 cooling pumps – 381.56 K
13.90	Second Peak Clad Temperature on ECIP of fuel pipe nr 6 with 1 cooling pump – 399 K

Table 4. Distribution of calculated top temperatures on fuel pipes

Pipe nr	2		3		4		5		6	
RELAP5 component	160		155		150		130		125	
Side of pipe	In.	Out.								
Max. temperature on the outer layer of clad	413	415	413	413	416	418	426	426	432	430
Max. temperature on the inner layer of clad	421	422	420	420	423	424	433	433	440	439

³ Aneks 2012/I do Eksplatacyjnego Raportu Bezpieczeństwa Reaktora MARIA Konwersja rdzenia reaktora MARIA na paliwo MC, July 2012.

Comparison of results

Clad temperature

The highest outer clad temperature calculated with the SN code was 433.4K, and temperature calculated with RELAP5 code was 432K (outer layer temperature – 440K). Both codes calculated the highest temperature in the same place – on the internal side of pipe no 6.

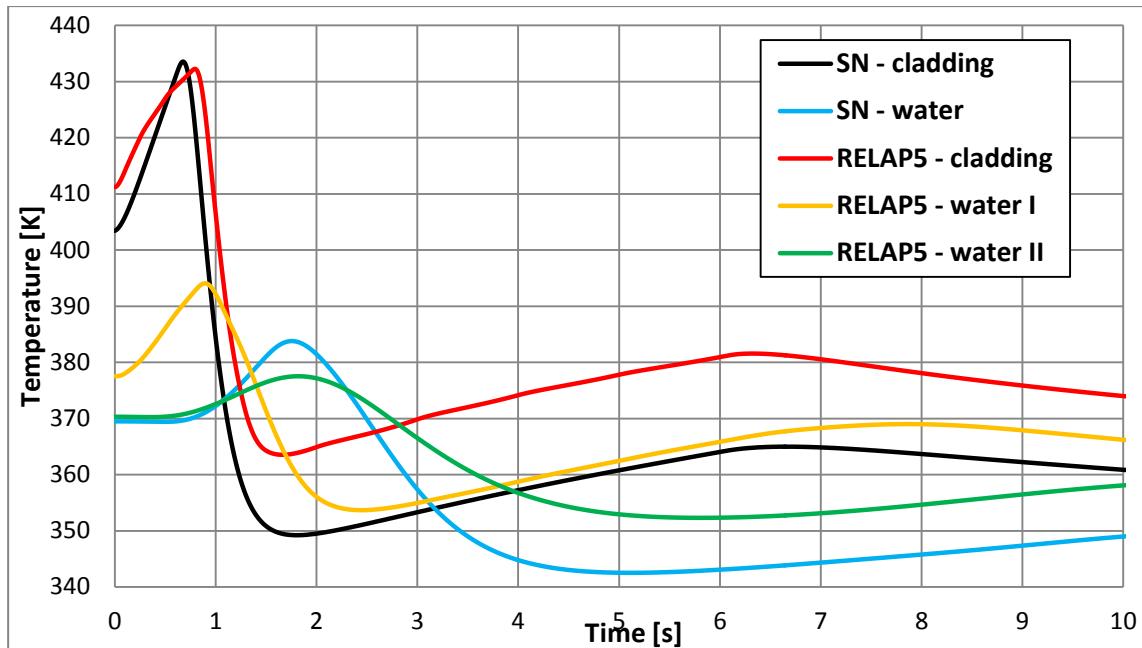


Figure 14. Cladding and water temperature changes. SN code: water and fuel element outlet temperature, RELAP5: water I – fuel element outlet temperature, water II - fuel channel outlet temperature

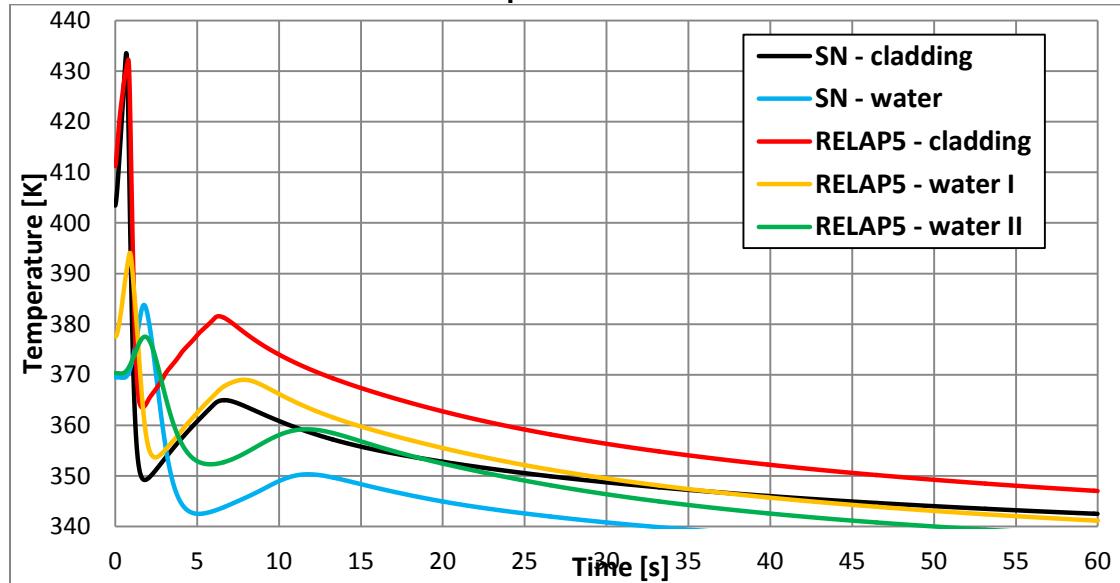


Figure 15. Cladding and water temperature changes. SN code: water and fuel element outlet temperature, RELAP5: water I – fuel element outlet temperature, water II - fuel channel outlet temperature. Wider scale range.

ONBR⁴ parameter

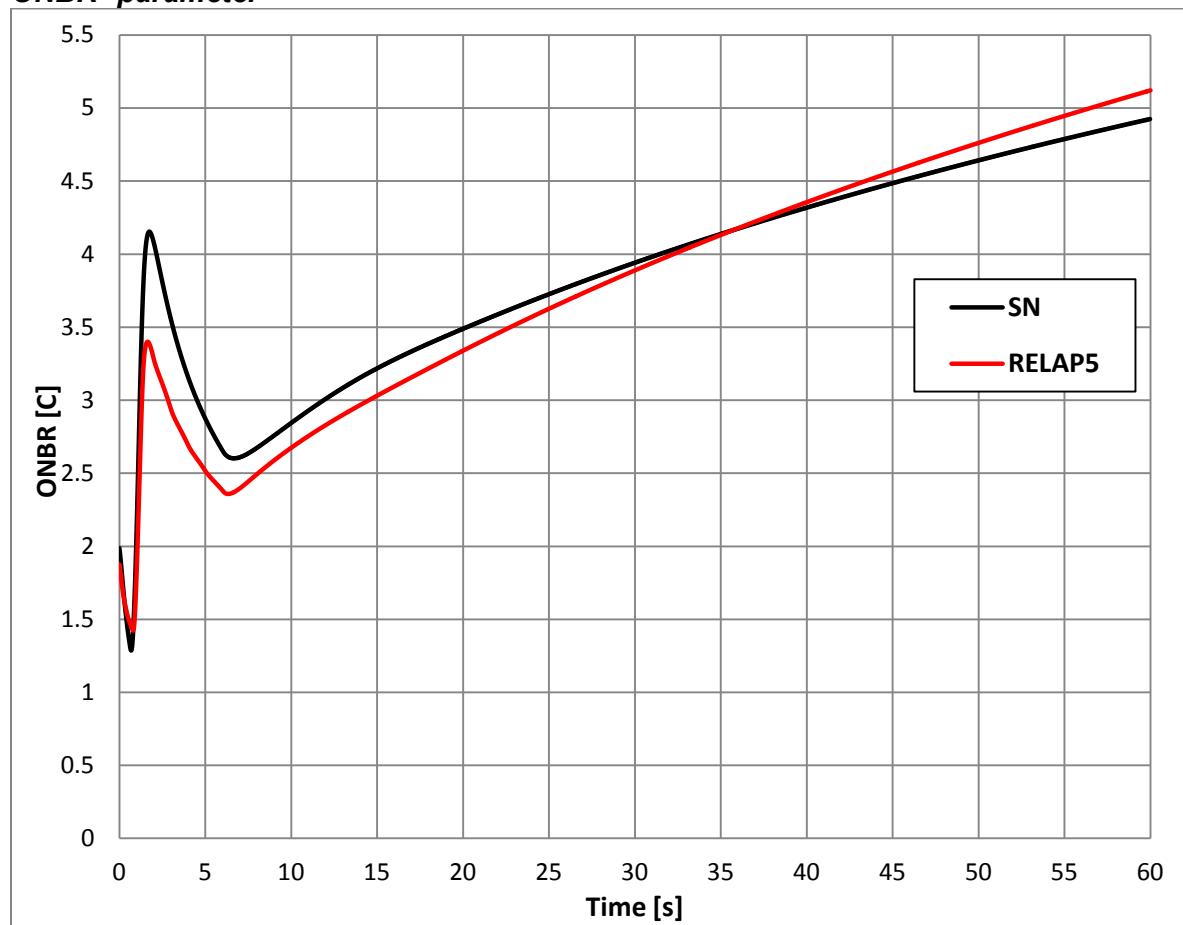


Figure 16. ONBR parameter changes

4 **ONBR (Onset of Nuclear Boiling Ratio)** should be greater than 1.2, where:

$$\text{ONBR} = (T_{\text{ONB}} - T_{\text{in}}) / (T_{\text{wall}} - T_{\text{in}})$$

$$T_{\text{ONB}} = T_{\text{sat}} + 0.182 * q^{0.35} / p^{0.23} \text{ (Forester-Greif correlations)}$$

T_{in} - inlet coolant temperature,

T_{wall} - the clad surface temperature,

T_{sat} - saturation coolant temperature,

q - heat flux [W/m^2],

p - pressure [bar].

Second clad temperature maximum (depending on the number of cooling pumps)

Table 5. Comparison of results for second temperature peak

Parameter	SN		RELAP5	
	1 pump	2 pumps	1 pump	2 pumps
Maximum outer temperature of clad [K]	387.25	364.98	399.0	381.56
Minimum ONBR	1.74	2.60	1.84	2.34

Differences in results

Differences in the results calculated by RELAP5 and the SN code arose due to many factors and not necessarily related only to nodalization. One of the factors - is a different flow model. In case of the SN code a more pessimistic method was used – coolant flow from fuel channel occurs at the same time in all gaps between fuel pipes. In case of RELAP5 a more realistic model was used. The water first flows down into the three gaps then gets mixed and next fills the rest of the gaps between the fuel pipes. The second example of differences is another model of decay heat. In case of RELAP5 decay heat was calculated by the code from kinetics and time of operation, whereas in the case of the SN code - decay heat was calculated on the basis of the old type of fuel measurements.

4.3 Slow reactivity insertion accident on low power level

Accident conditions

In case when the operator undertakes incorrect actions or a malfunction of the reactivity regulation system occurs, it is possible that as a result, positive reactivity is inserted into the core with the maximum reactivity speed 0.04\$/s. This scenario was checked for low power level conditions.

Initial conditions:

- Cooling channel Inlet water temperature $T_{in}=313.15K$
- Mass flow $Q=8.268\text{kg/s}$ ($30\text{m}^3/\text{h}$ for density of water: 992.2kg/m^3)
- Pressure at the fuel channel inlet $p_{in}=1.7\text{MPa}$
- Fuel channel pressure drop $\Delta p=0.59\text{MPa}$
- Fuel element thermal power 10kW

Boundary conditions:

- Alarm signal – signal from increase of power to 480% - in this case to 48kW
- Signal delay time $t_1=0.2\text{s}$
- Effective time of safety and control rods insertion $t_2=0.15\text{s}$
- Insertion positive reactivity from accident $t=0\text{s}$ $\varrho=0.04\$/\text{s}$, $\varrho_{max}=1.5\$$

- Inserted negative reactivity (from rods PK and PB) $\rho = -5.5\$$ (-2\\$ PK, -3.5\\$ PB)
- Temperature reactivity coefficients: $a_f = -0.23\phi/\text{deg}$, $a_w = -1.93\phi/\text{deg}$

Comparison of results

Table 6. Comparison of results for SRIA on low power level

Sequence of events	SN		RELAP5	
	Time [s]	Value	Time [s]	Value
Start of positive reactivity insertion	0.0	-	0.0	-
Alarm signal 480% of power	12.3	-	12.48	-
Start of negative reactivity insertion	no data	-	12.83	-
Maximum power	12.7	53kW	12.83	52.7kW
Maximum peak cladding temperature on ECIP of fuel pipe nr 6	no data	316.15K	12.85	316.14K
Maximum peak cladding temperature on ICIP of fuel pipe nr 6	no data	no data	12.85	316.44K
Minimum ONBR	no data	about 50	12.85	53.8
Maximum peak coolant temperature in gap nr 3 between fuel pipe nr 3 and 4	no data	no data	12.9	314.72K

This scenario has the lowest impact on the fuel elements. The temperature peaks of water and clad are very small - smaller than fluctuations of temperature during normal operation of the reactor.

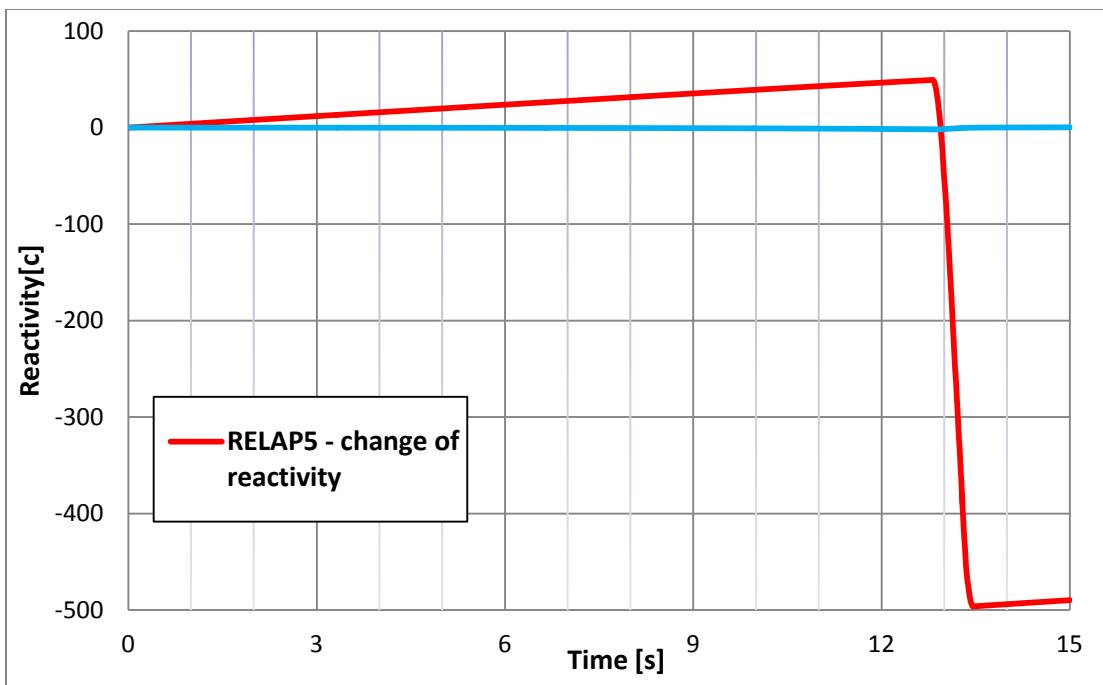


Figure 17. Reactivity changes. RELAP5: total reactivity change; water and fuel: reactivity change due to coolant and fuel temperature reactivity coefficients

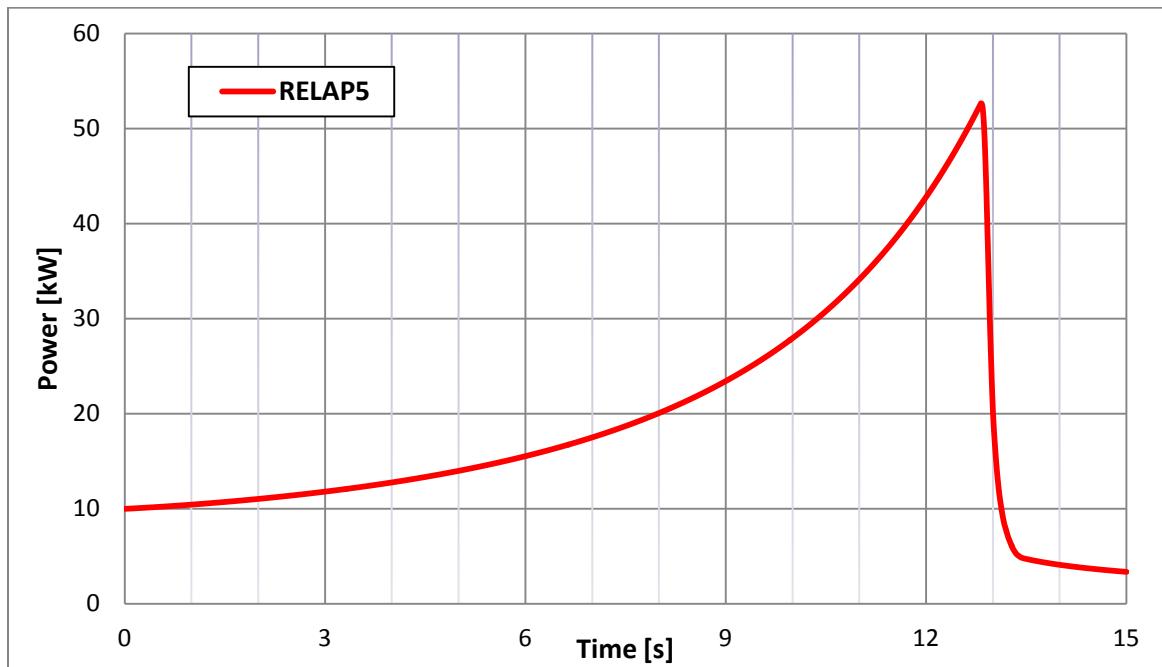


Figure 18. Power changes of fuel element for SRIA on low power level

4.4 Fast reactivity insertion accident on low power level

One of the important scenarios addressed in the supplementary document is the fast inflow of cold water into the core resulting in a sudden increase of reactivity. The negative reactivity coefficients lead to the increase of power in the fuel elements. The cold water is directed first between the first three pipes – pipes no. 4, 5, 6 on Figure 2 and after about 0.2s into the rest of the gaps.

Accident conditions

Initial conditions:

- Cooling channel Inlet water temperature $T_{in}=323.15K$, after 1s insertion of cold water $T_{2in}=283.15K$
- Mass flow $Q=8.268\text{kg/s}$ ($30\text{m}^3/\text{h}$ for density of water: 992.2kg/m^3) $T_{2in}=283.15K$.
- Pressure at the fuel channel inlet $p_{in}=1.7\text{MPa}$
- Fuel channel pressure drop $\Delta p=0.59\text{MPa}$
- Fuel element thermal power 10kW

Boundary conditions:

- Alarm signal – increase of power to 480% - in this case to 48kW
- Signal delay time $t_1=0.2s$
- Effective time of safety and control rods insertion $t_2=0.15s$
- Insertion of positive reactivity from accident $t=0s$ $\varrho=0.04\$/s$, $\varrho_{max}=1.5\$$
- Insertion of negative reactivity (from rods PK and PB) $\varrho=-5.5\$$ (-2\$ PK, -3.5\$ PB)
- Temperature reactivity coefficients: $\alpha_f=-0.23\text{f/deg}$, $\alpha_w=-1.93\text{f/deg}$

Comparison of results

Table 7. Comparison of results for FRIA on low power level

Sequence of events:	SN		RELAP5	
	Time [s]	Value	Time [s]	Value
Start of insertion of cold water to the fuel channel	1.0	-	1.0	-
Minimum ONBR	no data	about 5	1.34	4.95
Cold water In fuel pipes zone	1.59	-	1.67	-
Alarm 480% of power	2.44	-	2.47	-
Start of negative reactivity insertion	2.79	-	2.82	-
Maximum power	2.83	96.3kW	2.85	80.9kW

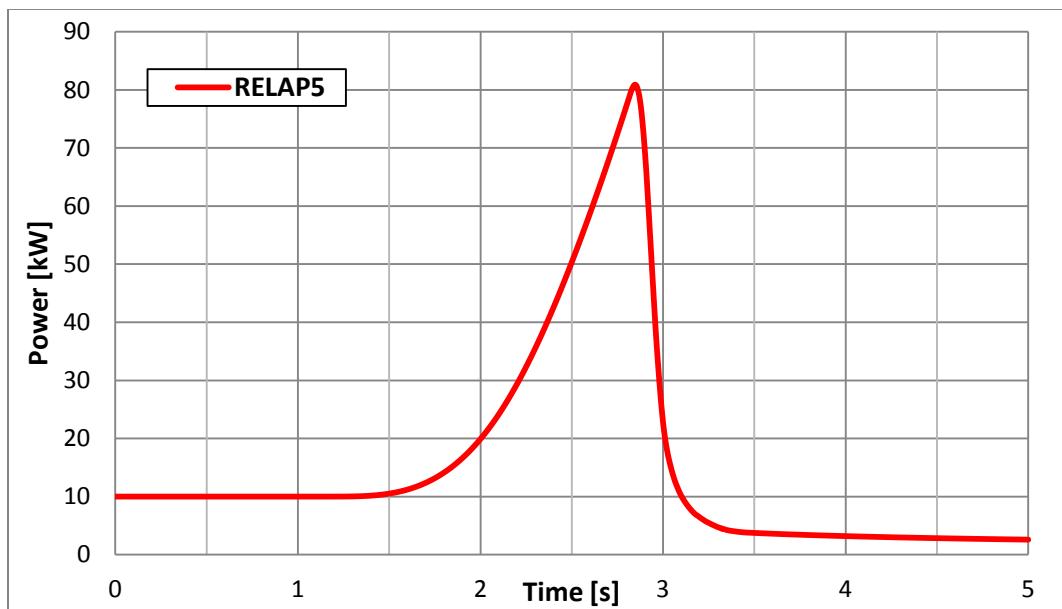


Figure 19. Power changes of fuel element for FRIA on low power level

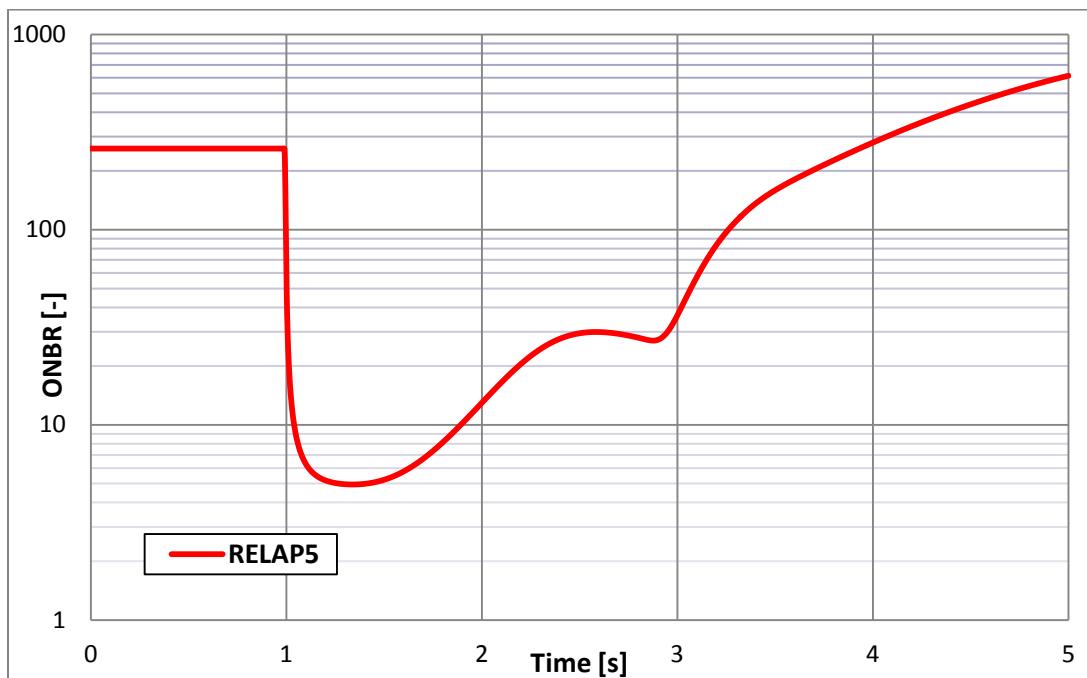


Figure 20. ONBR parameter changes

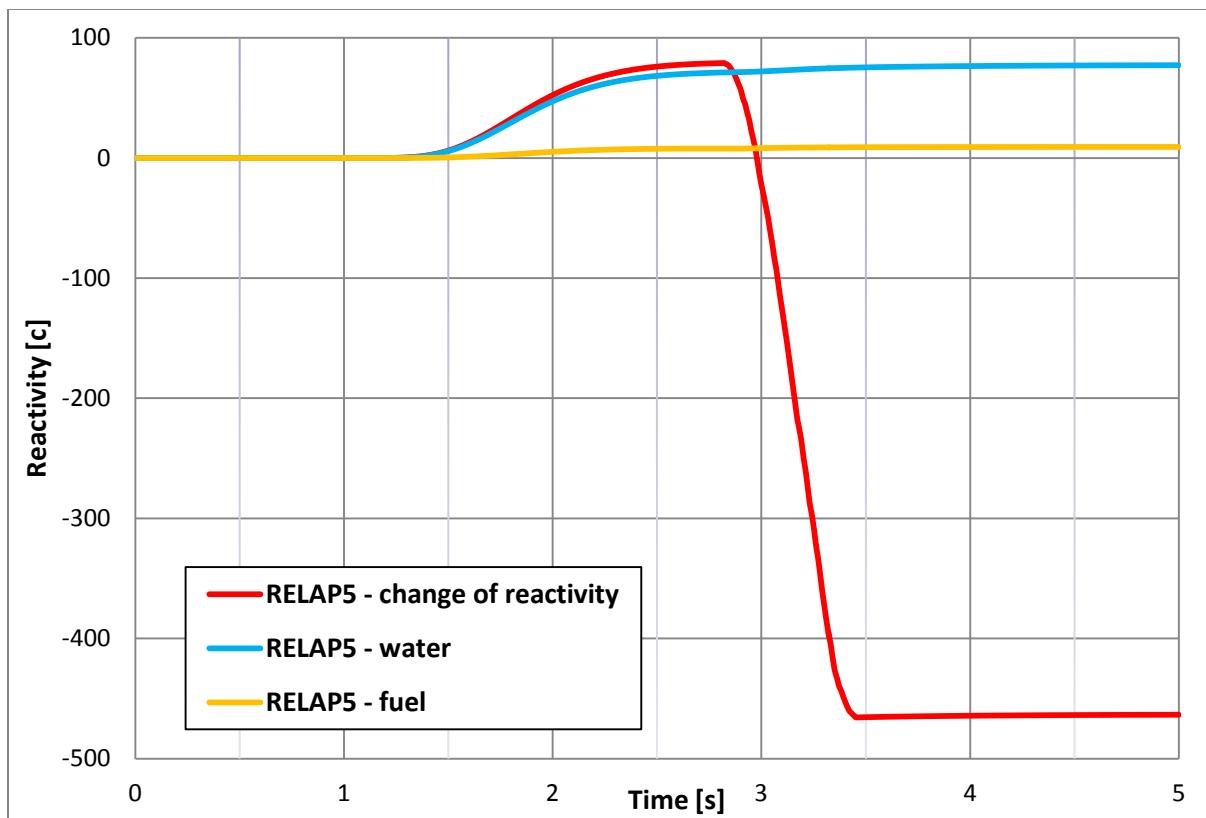


Figure 21. Reactivity changes - RELAP5 calculations: i) change of reactivity: total reactivity change, ii) water: change due to coolant temperature reactivity coefficients, iii) fuel: change due to fuel temperature reactivity coefficient

4.5 Slow reactivity insertion accident during normal operation of reactor

Accident conditions

The scenario analyzed in this transient is very similar to the slow reactivity insertion on low power. The main difference is that the reactor is in the normal operation mode, on full power, with the maximum allowed thermal power generation. In case when the operator undertakes incorrect actions or a malfunction of the reactivity regulation system occurs, it is possible that as a result, positive reactivity is inserted into the core with the maximum reactivity speed 0.04\$/s.

Initial conditions:

- Cooling channel Inlet water temperature $T_{in}=318.15K$,
- Mass flow $Q=8.252\text{kg/s}$ ($30\text{m}^3/\text{h}$ for density of water: 990.2kg/m^3)
- Pressure at the fuel channel inlet $p_{in}=1.7\text{MPa}$
- Fuel channel pressure drop $\Delta p=0.59\text{MPa}$
- Fuel element thermal power 1.8MW

Boundary conditions:

- Alarm signal – signal from increase of power to 120% - in this case to 2.16MW
- Signal delayed $t_1=0.2\text{s}$
- Effective time of safety and control rods insertion $t_2=0.15\text{s}$
- Insertion of positive reactivity from accident $t=0\text{s}$ $\varrho=0.04\$/\text{s}$, $\varrho_{\text{max}}=1.5\$$
- Insertion of negative reactivity (from rods PK and PB) $\varrho=-5.5\text{\$}$ (-2\\$ PK, -3.5\\$ PB)
- Temperature reactivity coefficients: $\alpha_f=-0.23\phi/\text{deg}$, $\alpha_w=-1.93\phi/\text{deg}$

Comparison of results

Table 8. Comparison of results for SRIA during normal operation

Sequence of events:	SN		RELAP5	
	Time [s]	Value	Time [s]	Value
Start of insertion of positive reactivity	0.0	0.04\\$/s	0.0	0.04\\$/s
Alarm signal – power 120%	5.37	-	6.44	-
Start of insertion of negative reactivity	no data	-	6.79	-
Maximum power	5.72	2.189MW	6.80	2.183MW
Maximum peak cladding temperature on ECIP of fuel pipe nr 6	5.73	424.86K	6.80	428.16K
Maximum peak cladding temperature on ICIP of fuel pipe nr 6	no data	-	6.80	440.5K
Minimum ONBR	5.73	about 1.6	6.80	1.60
Maximum peak coolant temperature on exit from fuel element	5.77	381.29K	6.86	386K

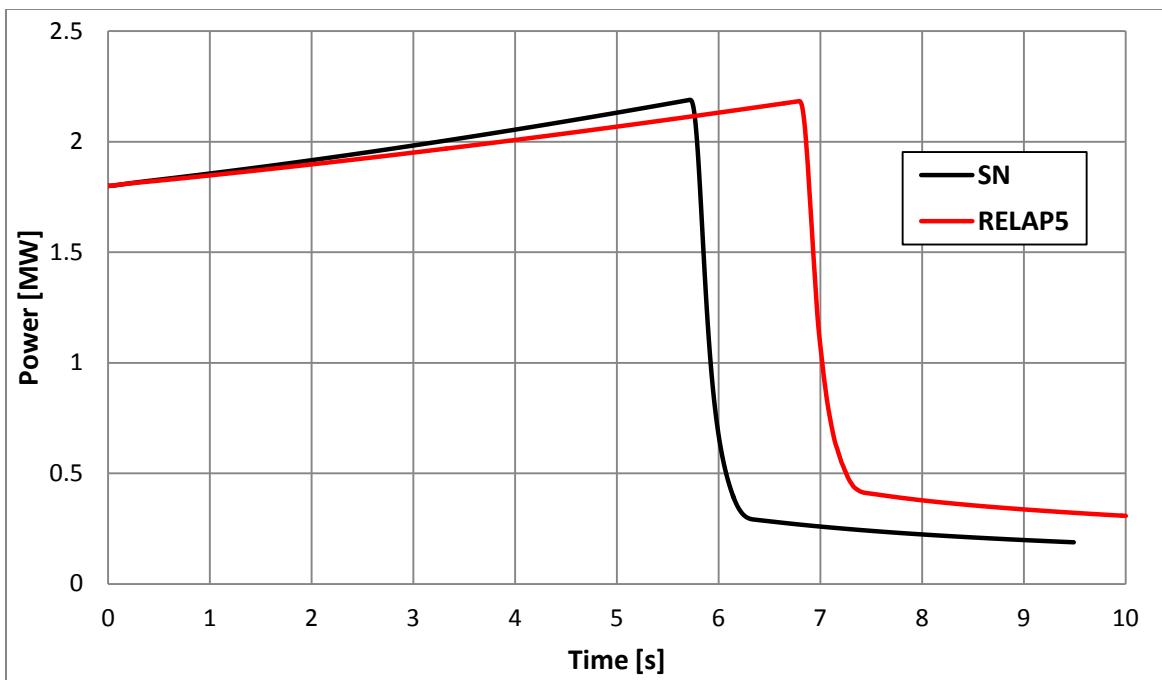


Figure 22. Power changes of fuel element for SRIA on nominal power level

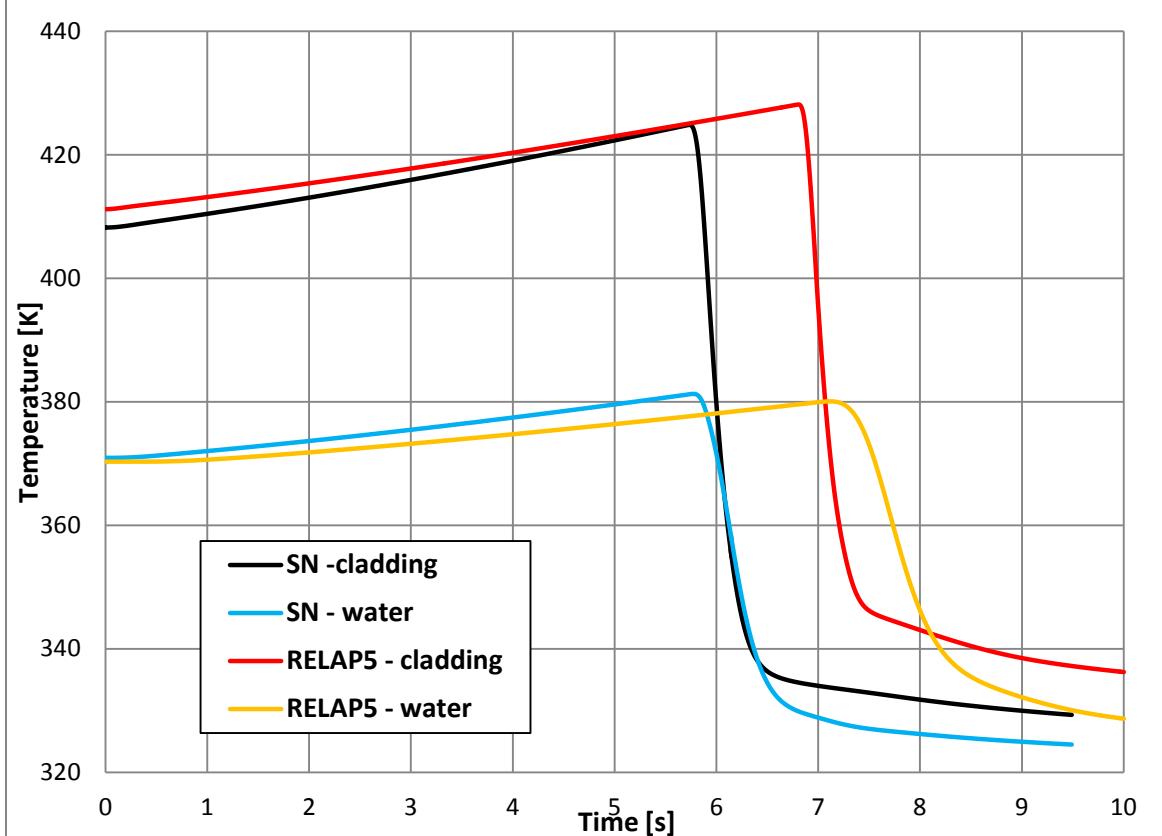


Figure 23. Cladding and water temperature changes. RELAP5 and SN codes: cladding – cladding temperature, water – fuel element outlet temperature

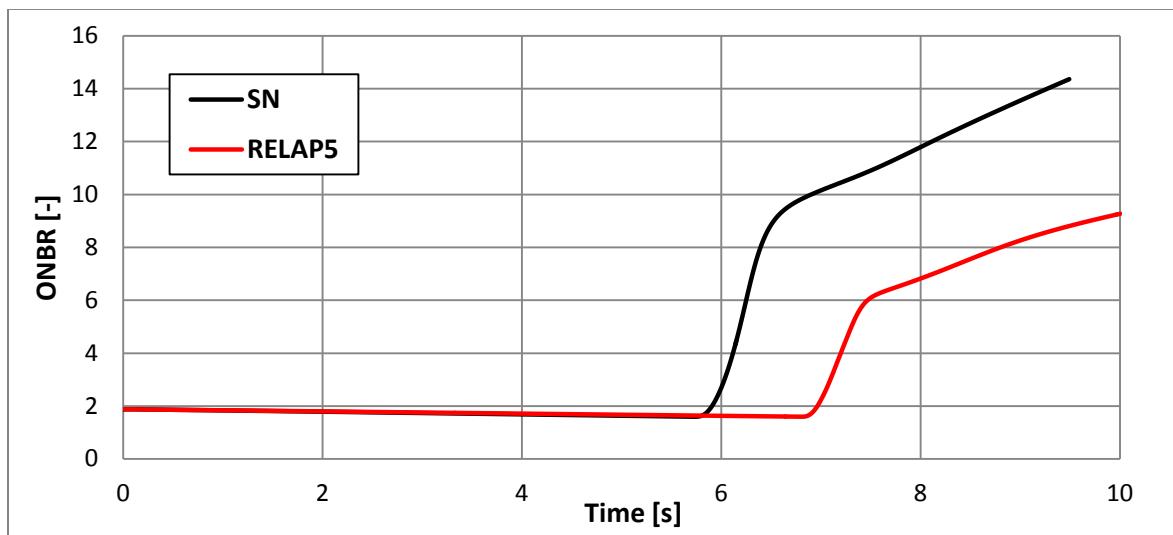


Figure 24. ONBR parameter changes

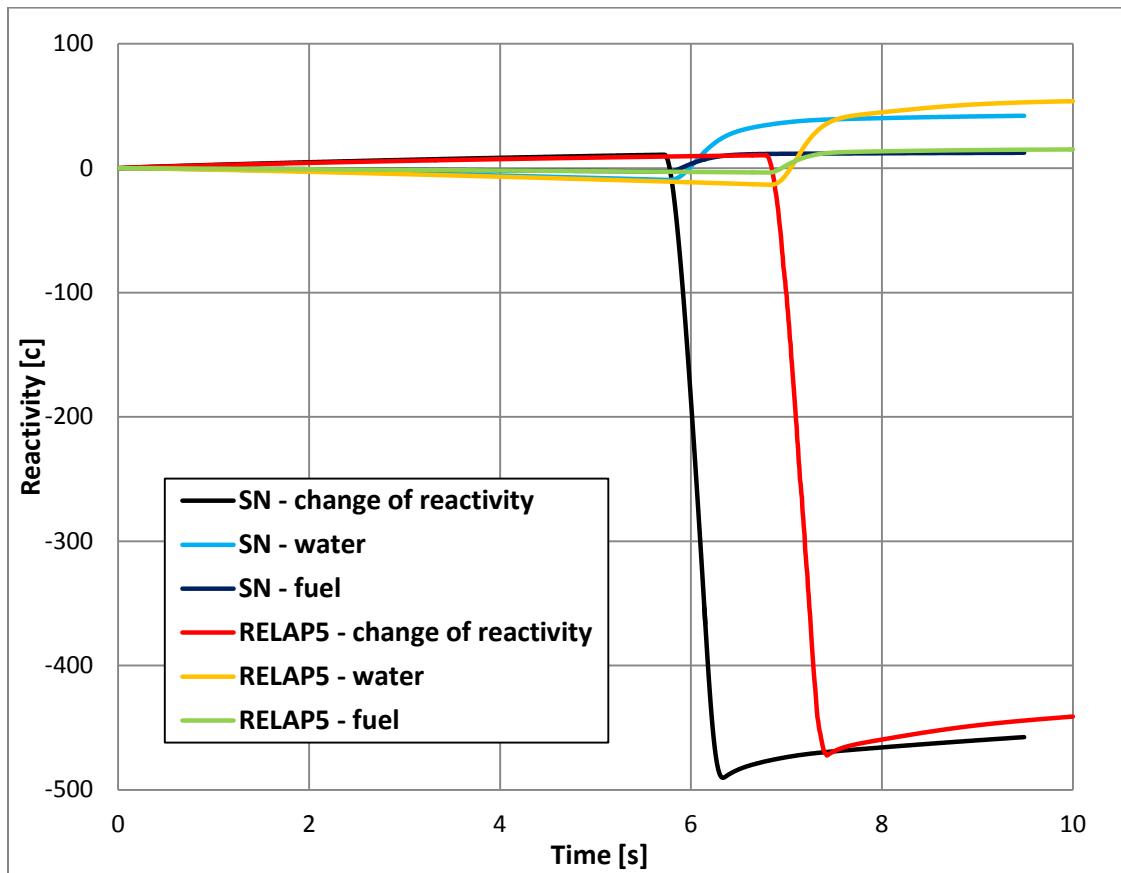


Figure 25. Reactivity changes. RELAP5 and SN codes: change of reactivity - total reactivity change, water - change due to coolant temperature reactivity coefficients, fuel - change due to fuel temperature reactivity coefficients

4.6 Fast reactivity insertion accident during reactor normal operation

Accident Conditions

A break of control rods is another scenario included in the supplementary document. The break of control rods leads to fast insertion of positive reactivity into the reactor. It is assumed that the maximum insertion of positive reactivity is equal to 1.5\$ with the top speed of insertion 4\$/s. Initial conditions:

- Cooling channel inlet water temperature $T_{in}=318.15K$,
- Mass flow $Q=8.252\text{kg/s}$ ($30\text{m}^3/\text{h}$ for density of water: 990.2kg/m^3)
- Pressure at the fuel channel inlet pin= 1.7MPa
- Fuel channel pressure drop $\Delta p=0.59\text{MPa}$
- Fuel element thermal power 1.8MW

Boundary conditions:

- Alarm signal – increase of power to 120% - in this case to 2.16MW
- Signal delay time $t_1=0.2\text{s}$
- Effective time of safety and control rods insertion $t_2=0.15\text{s}$
- Insertion of positive reactivity from accident $t=0\text{s}$ $\varrho=0.04\$/\text{s}$, $\varrho_{max}=1.5\$$
- Insertion of negative reactivity (from rods PK and PB) $\varrho=-5.5\text{\$}$ (-2\$ PK, -3.5\$ PB)
- Temperature reactivity coefficients: $\alpha_f=-0.23\text{\$/deg}$, $\alpha_w=-1.93\text{\$/deg}$

Comparison of results

Table 9. Comparison results for FRIA during normal operation

Sequence of events:	SN		RELAP5	
	Time [s]	Value	Time [s]	Value
Start of	0.0	4\$/s	0.0	4\$/s
Alarm signal 120%	0.08	-	0.07	-
Maximum thermal power	0.40	8.6MW	0.38	5.62MW
Maximum peak cladding temperature on ECIP of fuel pipe nr 6	no data	598.88K	0.40	515.52K
Maximum peak cladding temperature on ICIP of fuel pipe no. 6	no data	-	0.40	547.46K
Minimum ONBR	0.45	0.64	0.41	0.928
Start of insertion of negative reactivity	0.44	-	0.42	-
Max. peak coolant temperature on exit from fuel element	0.58	460.65K	0.55	448.39K

The main differences in the results between RELAP and the SN code are due to the different models of reactor kinetics. RELAP5 uses the point kinetic model and this model was applied to the RIA analysis. In the nodalization model applied, total reactivity was divided into many sections depending on the position in the fuel pipes or in the gap between pipes. The fuel reactivity coefficient was divided in 50 point, 10 points in each cell of the fuel pipe because the values of the divided fuel reactivity coefficients depend on the distribution of heat generation on fuel pipes and axial distribution of heat generation (i.e. shifted cosine model). The coolant reactivity coefficient was divided into 60 points, 10 points in each gap. The values of the divided coolant reactivity coefficient depend on the distribution of the mass flow between gaps and the distribution of heat (shifted cosine model). In case of the SN code a more pessimistic model was used.

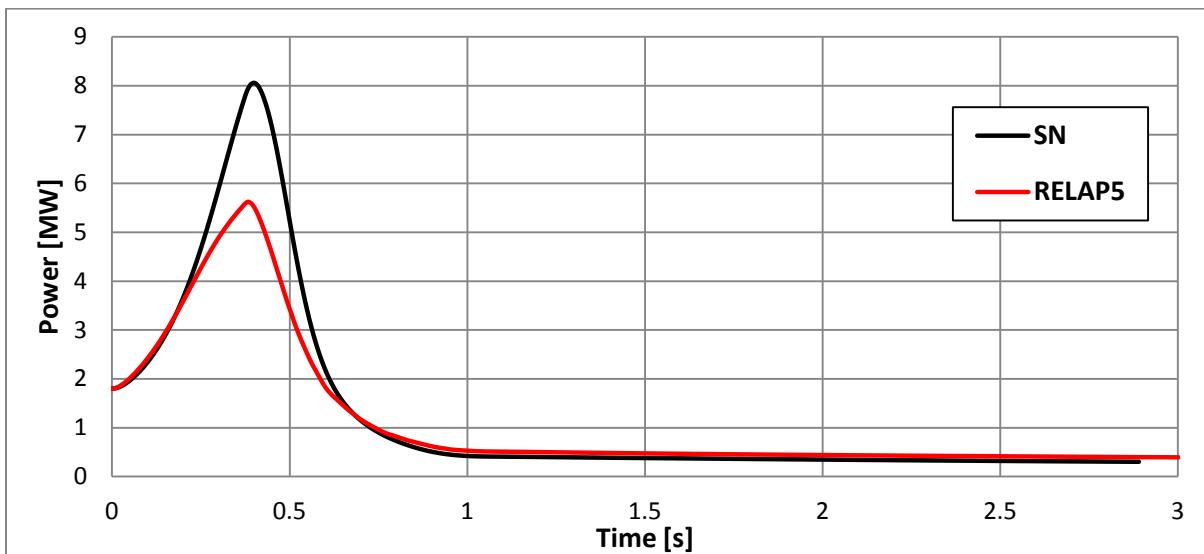


Figure 26. Power changes of fuel element for FRIA on nominal power level

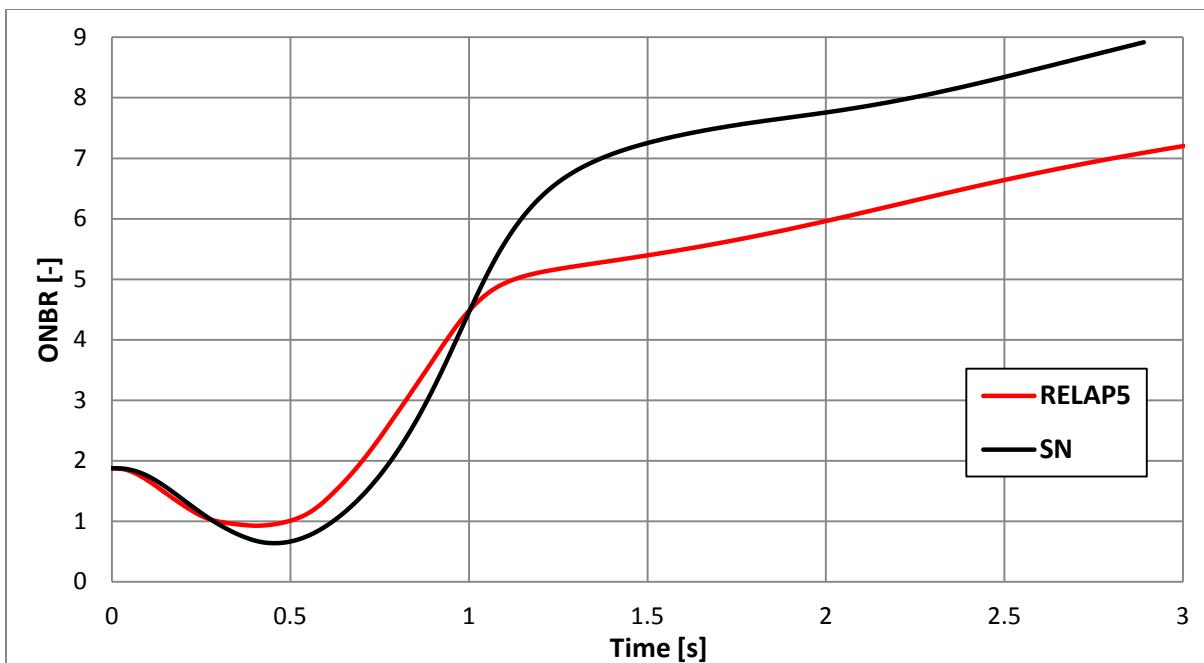


Figure 27. ONBR parameter changes

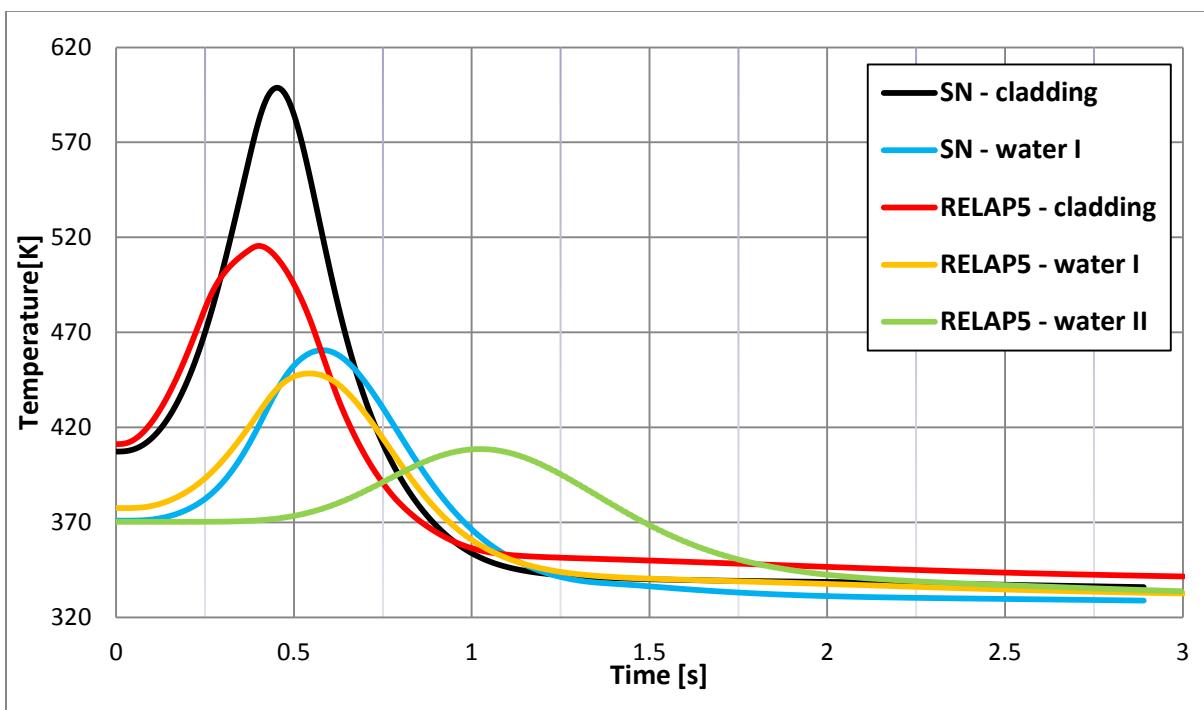


Figure 28. Cladding and water temperature changes. RELAP5 and SN codes: cladding – cladding temperature, water I – fuel element outlet temperature, water II - fuel channel outlet temperature

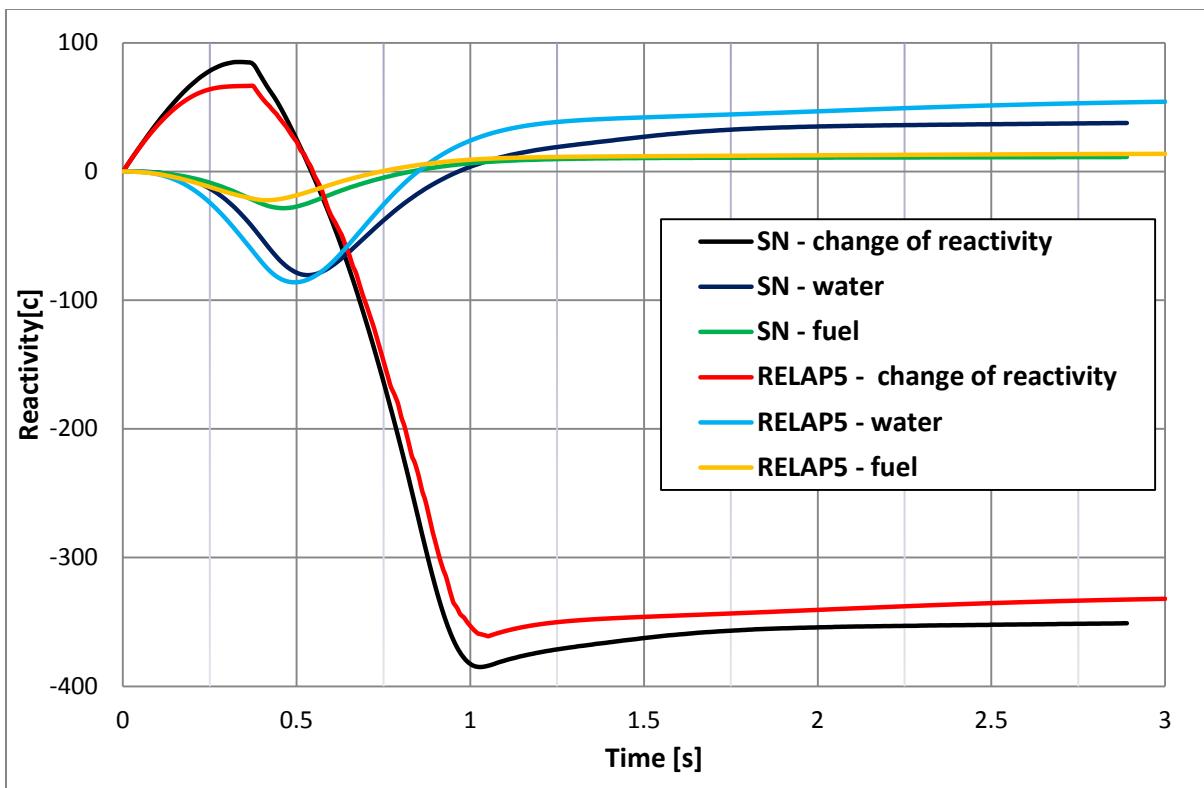


Figure 29. Reactivity changes. RELAP5 and SN codes: change of reactivity - total reactivity change, water - change due to coolant temperature reactivity coefficients, fuel - change due to fuel temperature reactivity coefficients

5 RUN STATISTICS

All analysis were performed on one computer:

Intel(R) Core(TM) i3-2125 CPU @ 3.30 GHz
RAM memory :4.00 GB
Windows 7 Professional – SP1, 64-bit version
IFC (Intel Fortran Compiler) for RELAP5

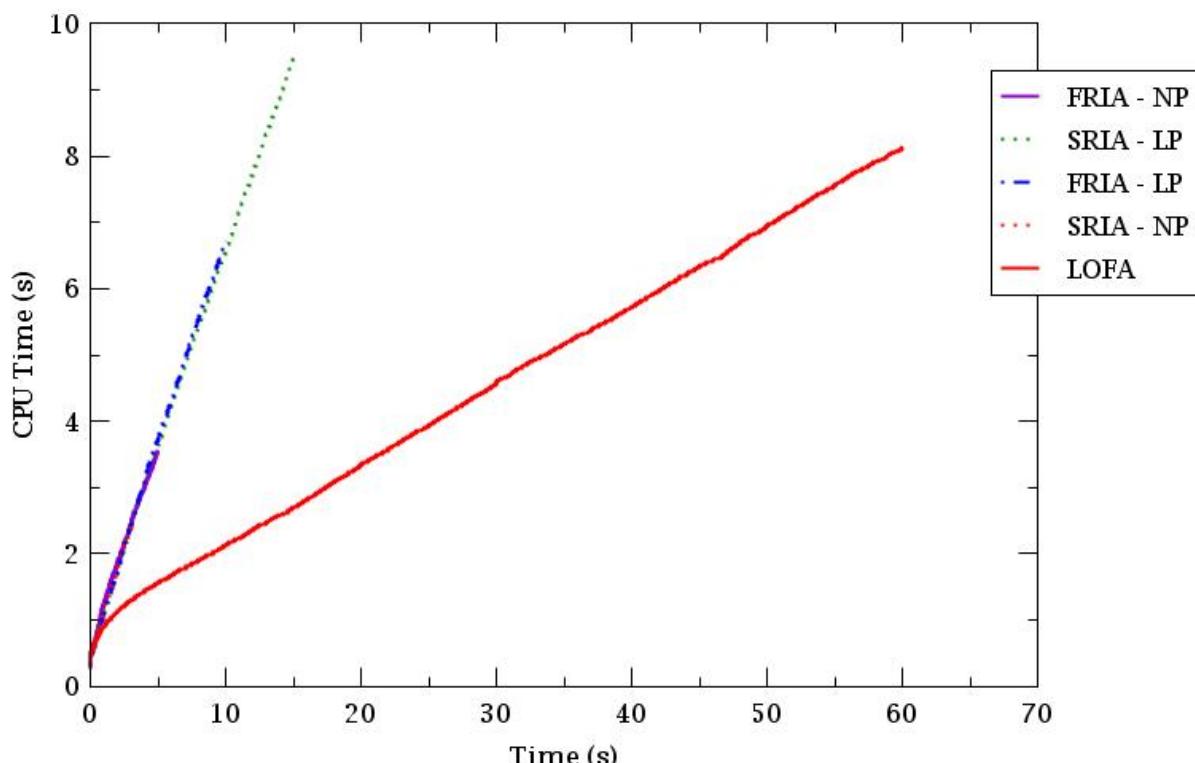


Figure 30. Calculation time for various cases

6 CONCLUSIONS

Most of the results of the simulation for the research reactor MARIA made with RELAP5 code showed that there is no big difference between the results achieved by the validated SN and the RELAP5 codes. Only in one case (RIA with fast reactivity insertion with maximum power level) there is the opportunity to exceed the lowest allowed level of the ONBR parameter. Exceeding this parameter can cause the onset of boiling of the coolant in the gap between the fuel pipes. In the analysed case, the duration of time when ONBR is lower than 1.2 is very short (about 0.35s) and should not cause any damage to the fuel and clad material. In the fast RIA scenario, the power jump during the insertion of positive reactivity was stopped by the negative reactivity coefficients (fuel and coolant) before insertion of the safety and control rods into the core could be initiated.

In other simulated scenarios, no opportunity to exceed the ONBR parameter or temperature limit of clad – 452K occurred.

In the analysed LOFA scenario, the calculations showed that enough cooling capability is provided even when only one pump is working with a lower rotation speed than the normally required two pumps.

The content of this report does not cover all possible accident conditions that should be analysed for the research reactor. Further analysis will be necessary. A comparison of the SN and RELAP5 codes showed that RELAP5 can be used in the safety analysis for the research reactor MARIA.

7 REFERENCES

1. “Aneks 2009/1 do Eksplotacyjnego Raportu Bezpieczeństwa Reaktora MARIA Badania paliwa MC”, 2009 (in Polish).
2. “Aneks 2012/1 do Eksplotacyjnego Raportu Bezpieczeństwa Reaktora MARIA Konwersja rdzenia reaktora MARIA na paliwo MC”, July 2012 (in Polish).
3. (SAR) “Eksplotacyjny Raport Bezpieczeństwa Reaktora MARIA”, 2009 (in Polish).

APPENDIX A – TECHNICAL INFORMATION ON THE RESEARCH REACTOR MARIA

The research reactor ‘Maria’ was designed by the Institute for Nuclear Studies in Poland and is operated by the National Centre for Nuclear Research that was created on the basis of two older Institutes. The reactor started to work in 1974 and is under operation till now except for the period of 1986 to 1991 when it was undergoing modernization.

It is a pool type reactor with closed fuel channels and closed fuel channel primary cooling circuit, primary pool cooling circuit and secondary cooling circuit. The fuel channel cooling circuit includes also a pressurizer.

At present the reactor is used for the production of isotopes and for performing experiments. The total thermal power is 30 MW. More data that might be of interest is presented below.

Thermo-hydraulic parameters	During a typical cycle of work: Thermal power of a fuel channel: 15-19 MW(th) for 19-23 fuel channels, Inlet temperature to fuel channels: 303-323 K, Temperature increase in the core: 20-30 deg, Hot leg pressure (in outlet collector): about 0.9-1 MPa, Cold leg pressure (in inlet collector): about 1.7-1.8 MPa, Minimum flow in fuel channels 30 m ³ /h for channels with maximum thermal power 1.8 MW, Minimum flow is 15 m ³ /h for channels with 0.9 MW (in peripheral core position).
Pressurizer	Filled with water and helium gas under high pressure At pressure about 1.7 MPa maintains sufficient water inventory in the circuit.
Safety signals causing reactor scram	1) Decrease of mass flow in the fuel channel in the channel cooling circuit. Each fuel channel is equipped with temperature and mass flow measurements (it is important for calculating the thermal power of the element), a small value of the mass flow in the channel will generate a “low mass flow” signal i.e. a warning signal when the flow drops to 90% of nominal mass flow and an alarm when the flow drops to 80% of nominal mass flow. 2) Kinetics control system. In case of sudden power jump signal is generated through detectors and other systems. It’s warning signal when is 110% of nominal reactor power, and alarm signal in case of 120% of nominal power. Additional safety signal for fast reactivity jump it is

	alarm signal from increase of period of the reactor.
Reactivity coefficients	<p>Fuel reactivity coefficient (negative) $-0.23\text{¢}/\text{deg}$ to $-0.25\text{¢}/\text{deg}$</p> <p>Coolant reactivity coefficient in the fuel channel (negative) $-1.93\text{¢}/\text{deg}$ to $-2.09\text{¢}/\text{deg}$</p> <p>Coolant reactivity coefficient of the pool plus moderator reactivity coefficient (positive) $+1.9\text{¢}/\text{deg}$</p> <p>Void reactivity coefficient (negative) $-9.37 \text{ ¢}/\% \text{ of void}$ to $-9.71\text{¢}/\% \text{ of void}$</p> <p>Values of coefficients depend on the beginning or end of the fuel cycle and number of molybdenum channels in the core (none, one or two molybdenum channels).</p>
Control system	The reactor power is controlled through changes of position of the control rods and automatic or manual changes of position of the UAR. The UAR is an automatic regulation system, that through the change of position of one control rod in the reactor core controls the power level and maintains, if it's necessary, the criticality of the reactor.
Shutdown system	<p>The shutdown system is provided through safety rods (insertion reactivity from $-7.5\\$ to $-11\\$ to the core) and control rods (the weights depend on the position of the rods in the core, it can be pessimistically assumed that the injection is $-2.5\\$). Safety rods and control rods work in accordance to the fail-safe rule. In case of loss of the power supply the safety rods will be automatically released from electromagnets into the reactor core.</p> <p>It's also possible to remove two fuel elements from the core. This is done by moving two fuel type elements (REP). This action is slower than the movement of safety rods. Sometimes it's impossible to remove the REP because of temperature stress until several hours after reactor shutdown. The reactivity of two REPs is about $4.2 +/- 0.5\\$.</p> <p>The shutdown system does not depend on the boron injection system.</p>
Emergency core cooling system	<p>In case of low water level in the pressurizer, an additional pump (1u6) that is connected to the pressurizer is automatically switched on and pumps water from the tank system (water volume $20-30\text{m}^3$) into the pressurizer. The solution is good only in case of a small leakage from the channel cooling circuit.</p> <p>The gas discharge system from the pressurizer. In case of very low water level in the pressurizer a discharge of helium</p>

	into the tank system through a system of valves will take place. This will result in a sudden depressurization of the channel cooling circuit. The safety valves 1z100A/B. In case of depressurization the valves automatically open and connect the channel cooling circuit with pool of the reactor (due to the pressure difference between two circuits).
Fuelling	Fuelling is performed when necessary depending on the reactivity supply, however always before a new work cycle. Refuelling, minimum 48 hours after a previous cycle, includes also a possibility of adding fuel elements removed after earlier cycles.
Cooling system	Fuel cooling channels are cooled by two from four pumps. After shut-down is done by two from the four mentioned pumps but with rotation speed changed from 3000r/s to 1500r/s. Hot water from the core passes through the heat exchangers system (four from six) transferring heat to the secondary circuit. The pool is individually cooled by three from four pumps and in shutdown mode - by one from two shutdown pumps. There is one secondary cooling circuit for the fuel cooling circuit and the pool cooling circuit. It pumps water to the outer cooling tower with ventilators. During normal operation two from three pumps and after shutdown one from three shutdown pumps are in operation. During loss of external power after a rapid shutdown, in order to remove decay heat, about 4 hours of cooling with post-shutdown pumps is necessary. The post-shutdown pumps are powered by accumulator batteries or diesel generators.
Moderator	Beryllium blocks are used as a moderator. They are placed between fuel channels.
Reflector	Graphite blocks are used as a reflector. They placed around beryllium blocks and fuel channels.
Operation	The reactor works in cycles lasting from five to ten days with nominal power usually between 16 and 23 MW(th). In some cases the reactor is used for training of students and then it operates for several hours with the power level about 10 kW(th).
Applications	Production of radioactive materials for medicine – mainly

molybdenum production in special molybdenum channels.
Training of students.
Research activities.

Safety analysis Safety Analysis Report [1] updated in 2009.
 Update of the report is planned till 03.2015.

APPENDIX B – INPUT FILE.

**RESEARCH REACTOR ‘MARIA’
STEADY STATE CALCULATIONS INPUT
DATE: 26.10.2012**

```

=MARIA
*m: SNAP:Symbolic Nuclear Analysis Package, Version 2.2.0,
September 04, 2012
*m: PLUGIN:RELAP Version 4.3.0
*m: CODE:RELAP5 Version 3.3
*m: DATE:10/26/12
***** Model Options *****
***** type state *****
100 new transnt
* iunits ounts
102 si si
*tend minstep maxstep copt pfreq majed rsrtf
201 10.0 1.0e-8 0.5 3 5 100 1000
20500000 9999
***** Control Blocks *****
***** name type scale ival iflag limit *****
20500010 "dP_15-65" sum 1.0 5.882538e5 0 0
* a0 scale name param
20500011 0.0 1.0 p 15010000
20500012 -1.0 p 65010000
* name type scale ival iflag limit
20500020 "dT_10-70" sum 1.0 0.38535583 0 0
* a0 scale name param
20500021 0.0 -1.0 tempf 10010000
20500022 1.0 tempf 70060000
* name type scale ival iflag limit
20500030 "Ent_70" mult 1.0 1.743369e6 0 0
* input param input param
20500031 hvmix 70060000 mflowj 74000000
* name type scale ival iflag limit
20500040 "Power" sum 1.0 8973.1709 0 0
* a0 scale name param
20500041 0.0 -1.0 cntrlvar 5
20500042 1.0 cntrlvar 3
* name type scale ival iflag limit
20500050 "Ent_10" mult 1.0 1.734396e6 0 0
* input param input param
20500051 hvmix 10010000 mflowj 6000000
* name type scale ival iflag limit
20500100 "unnamed" sum 1.0 323.40231 0 0
* a0 scale name param
20500101 0.0 1.0 httemp 6000113
* name type scale ival iflag limit
20500110 "unnamed" sum 1.0 323.46774 0 0
* a0 scale name param
20500111 0.0 1.0 httemp 6000213
* name type scale ival iflag limit
20500120 "unnamed" sum 1.0 323.52133 0 0
* a0 scale name param
20500121 0.0 1.0 httemp 6000313
* name type scale ival iflag limit
20500130 "unnamed" sum 1.0 323.55933 0 0
* a0 scale name param
20500131 0.0 1.0 httemp 6000413
* name type scale ival iflag limit
20500140 "unnamed" sum 1.0 323.57919 0 0
* a0 scale name param
20500141 0.0 1.0 httemp 6000513
* name type scale ival iflag limit
20500150 "unnamed" sum 1.0 323.57993 0 0
* a0 scale name param
20500151 0.0 1.0 httemp 6000613
* name type scale ival iflag limit
20500160 "unnamed" sum 1.0 323.56198 0 0
* a0 scale name param
20500161 0.0 1.0 httemp 6000713
* name type scale ival iflag limit
20500170 "unnamed" sum 1.0 323.5274 0 0
* a0 scale name param
20500171 0.0 1.0 httemp 6000813
* name type scale ival iflag limit
20500180 "unnamed" sum 1.0 323.47955 0 0
* a0 scale name param
20500181 0.0 1.0 httemp 6000913
* name type scale ival iflag limit
20500190 "unnamed" sum 1.0 323.45679 0 0
* a0 scale name param

```

```

20500191 0.0 1.0 httemp 6001013
* name type scale ival iflag limit
20500200 "unnamed" sum 1.0 323.40326 0 0
* a0 scale name param
20500201 0.0 1.0 httemp 6000105
* name type scale ival iflag limit
20500210 "unnamed" sum 1.0 323.46854 0 0
* a0 scale name param
20500211 0.0 1.0 httemp 6000205
* name type scale ival iflag limit
20500220 "unnamed" sum 1.0 323.5217 0 0
* a0 scale name param
20500221 0.0 1.0 httemp 6000305
* name type scale ival iflag limit
20500230 "unnamed" sum 1.0 323.55899 0 0
* a0 scale name param
20500231 0.0 1.0 httemp 6000405
* name type scale ival iflag limit
20500240 "unnamed" sum 1.0 323.57797 0 0
* a0 scale name param
20500241 0.0 1.0 httemp 6000505
* name type scale ival iflag limit
20500250 "unnamed" sum 1.0 323.57767 0 0
* a0 scale name param
20500251 0.0 1.0 httemp 6000605
* name type scale ival iflag limit
20500260 "unnamed" sum 1.0 323.55865 0 0
* a0 scale name param
20500261 0.0 1.0 httemp 6000705
* name type scale ival iflag limit
20500270 "unnamed" sum 1.0 323.52307 0 0
* a0 scale name param
20500271 0.0 1.0 httemp 6000805
* name type scale ival iflag limit
20500280 "unnamed" sum 1.0 323.47437 0 0
* a0 scale name param
20500281 0.0 1.0 httemp 6000905
* name type scale ival iflag limit
20500290 "unnamed" sum 1.0 323.45117 0 0
* a0 scale name param
20500291 0.0 1.0 httemp 6001005
* name type scale ival iflag limit
20500300 "unnamed" sum 1.0 323.42294 0 0
* a0 scale name param
20500301 0.0 1.0 httemp 5500113
* name type scale ival iflag limit
20500310 "unnamed" sum 1.0 323.49878 0 0
* a0 scale name param
20500311 0.0 1.0 httemp 5500213
* name type scale ival iflag limit
20500320 "unnamed" sum 1.0 323.56104 0 0
* a0 scale name param
20500321 0.0 1.0 httemp 5500313
* name type scale ival iflag limit
20500330 "unnamed" sum 1.0 323.60513 0 0
* a0 scale name param
20500331 0.0 1.0 httemp 5500413
* name type scale ival iflag limit
20500340 "unnamed" sum 1.0 323.62796 0 0
* a0 scale name param
20500341 0.0 1.0 httemp 5500513
* name type scale ival iflag limit
20500350 "unnamed" sum 1.0 323.62811 0 0
* a0 scale name param
20500351 0.0 1.0 httemp 5500613
* name type scale ival iflag limit
20500360 "unnamed" sum 1.0 323.60602 0 0
* a0 scale name param
20500361 0.0 1.0 httemp 5500713
* name type scale ival iflag limit
20500370 "unnamed" sum 1.0 323.5639 0 0
* a0 scale name param
20500371 0.0 1.0 httemp 5500813
* name type scale ival iflag limit
20500380 "unnamed" sum 1.0 323.50562 0 0
* a0 scale name param
20500381 0.0 1.0 httemp 5500913
* name type scale ival iflag limit

```

```

20500390 "unnamed" sum 1.0 323.47556 0 0
*      a0 scale name param
20500391 0.0 1.0 httemp 5501013
*      name type scale ival iflag limit
20500400 "unnamed" sum 1.0 323.43329 0 0
*      a0 scale name param
20500401 0.0 1.0 httemp 5500105
*      name type scale ival iflag limit
20500410 "unnamed" sum 1.0 323.51294 0 0
*      a0 scale name param
20500411 0.0 1.0 httemp 5500205
*      name type scale ival iflag limit
20500420 "unnamed" sum 1.0 323.57779 0 0
*      a0 scale name param
20500421 0.0 1.0 httemp 5500305
*      name type scale ival iflag limit
20500430 "unnamed" sum 1.0 323.62311 0 0
*      a0 scale name param
20500431 0.0 1.0 httemp 5500405
*      name type scale ival iflag limit
20500440 "unnamed" sum 1.0 323.64563 0 0
*      a0 scale name param
20500441 0.0 1.0 httemp 5500505
*      name type scale ival iflag limit
20500450 "unnamed" sum 1.0 323.64407 0 0
*      a0 scale name param
20500451 0.0 1.0 httemp 5500605
*      name type scale ival iflag limit
20500460 "unnamed" sum 1.0 323.61902 0 0
*      a0 scale name param
20500461 0.0 1.0 httemp 5500705
*      name type scale ival iflag limit
20500470 "unnamed" sum 1.0 323.57294 0 0
*      a0 scale name param
20500471 0.0 1.0 httemp 5500805
*      name type scale ival iflag limit
20500480 "unnamed" sum 1.0 323.51001 0 0
*      a0 scale name param
20500481 0.0 1.0 httemp 5500905
*      name type scale ival iflag limit
20500490 "unnamed" sum 1.0 323.47787 0 0
*      a0 scale name param
20500491 0.0 1.0 httemp 5501005
*      name type scale ival iflag limit
20500500 "unnamed" sum 1.0 323.45538 0 0
*      a0 scale name param
20500501 0.0 1.0 httemp 5000113
*      name type scale ival iflag limit
20500510 "unnamed" sum 1.0 323.52072 0 0
*      a0 scale name param
20500511 0.0 1.0 httemp 5000213
*      name type scale ival iflag limit
20500520 "unnamed" sum 1.0 323.56552 0 0
*      a0 scale name param
20500521 0.0 1.0 httemp 5000313
*      name type scale ival iflag limit
20500530 "unnamed" sum 1.0 323.58615 0 0
*      a0 scale name param
20500531 0.0 1.0 httemp 5000413
*      name type scale ival iflag limit
20500540 "unnamed" sum 1.0 323.58093 0 0
*      a0 scale name param
20500541 0.0 1.0 httemp 5000513
*      name type scale ival iflag limit
20500550 "unnamed" sum 1.0 323.55032 0 0
*      a0 scale name param
20500551 0.0 1.0 httemp 5000613
*      name type scale ival iflag limit
20500560 "unnamed" sum 1.0 323.49677 0 0
*      a0 scale name param
20500561 0.0 1.0 httemp 5000713
*      name type scale ival iflag limit
20500570 "unnamed" sum 1.0 323.42453 0 0
*      a0 scale name param
20500571 0.0 1.0 httemp 5000813
*      name type scale ival iflag limit
20500580 "unnamed" sum 1.0 323.33942 0 0
*      a0 scale name param

20500581 0.0 1.0 httemp 5000913
*      name type scale ival iflag limit
20500590 "unnamed" sum 1.0 323.29184 0 0
*      a0 scale name param
20500591 0.0 1.0 httemp 5001013
*      name type scale ival iflag limit
20500600 "unnamed" sum 1.0 323.4418 0 0
*      a0 scale name param
20500601 0.0 1.0 httemp 5000105
*      name type scale ival iflag limit
20500610 "unnamed" sum 1.0 323.50943 0 0
*      a0 scale name param
20500611 0.0 1.0 httemp 5000205
*      name type scale ival iflag limit
20500620 "unnamed" sum 1.0 323.55914 0 0
*      a0 scale name param
20500621 0.0 1.0 httemp 5000305
*      name type scale ival iflag limit
20500630 "unnamed" sum 1.0 323.58707 0 0
*      a0 scale name param
20500631 0.0 1.0 httemp 5000405
*      name type scale ival iflag limit
20500640 "unnamed" sum 1.0 323.59106 0 0
*      a0 scale name param
20500641 0.0 1.0 httemp 5000505
*      name type scale ival iflag limit
20500650 "unnamed" sum 1.0 323.57092 0 0
*      a0 scale name param
20500651 0.0 1.0 httemp 5000605
*      name type scale ival iflag limit
20500660 "unnamed" sum 1.0 323.52841 0 0
*      a0 scale name param
20500661 0.0 1.0 httemp 5000705
*      name type scale ival iflag limit
20500670 "unnamed" sum 1.0 323.46704 0 0
*      a0 scale name param
20500671 0.0 1.0 httemp 5000805
*      name type scale ival iflag limit
20500680 "unnamed" sum 1.0 323.39188 0 0
*      a0 scale name param
20500681 0.0 1.0 httemp 5000905
*      name type scale ival iflag limit
20500690 "unnamed" sum 1.0 323.3504 0 0
*      a0 scale name param
20500691 0.0 1.0 httemp 5001005
*      name type scale ival iflag limit
20500700 "unnamed" sum 1.0 323.52029 0 0
*      a0 scale name param
20500701 0.0 1.0 httemp 3000113
*      name type scale ival iflag limit
20500710 "unnamed" sum 1.0 323.58707 0 0
*      a0 scale name param
20500711 0.0 1.0 httemp 3000213
*      name type scale ival iflag limit
20500720 "unnamed" sum 1.0 323.62198 0 0
*      a0 scale name param
20500721 0.0 1.0 httemp 3000313
*      name type scale ival iflag limit
20500730 "unnamed" sum 1.0 323.62195 0 0
*      a0 scale name param
20500731 0.0 1.0 httemp 3000413
*      name type scale ival iflag limit
20500740 "unnamed" sum 1.0 323.58676 0 0
*      a0 scale name param
20500741 0.0 1.0 httemp 3000513
*      name type scale ival iflag limit
20500750 "unnamed" sum 1.0 323.51904 0 0
*      a0 scale name param
20500751 0.0 1.0 httemp 3000613
*      name type scale ival iflag limit
20500760 "unnamed" sum 1.0 323.42395 0 0
*      a0 scale name param
20500761 0.0 1.0 httemp 3000713
*      name type scale ival iflag limit
20500770 "unnamed" sum 1.0 323.30902 0 0
*      a0 scale name param
20500771 0.0 1.0 httemp 3000813
*      name type scale ival iflag limit

```

```

20500780 "unnamed" sum 1.0 323.18326 0 0
*      a0 scale name param
20500781 0.0 1.0 httemp 3000913
*      name type scale ival iflag limit
20500790 "unnamed" sum 1.0 323.11334 0 0
*      a0 scale name param
20500791 0.0 1.0 httemp 3001013
*      name type scale ival iflag limit
20500800 "unnamed" sum 1.0 323.5889 0 0
*      a0 scale name param
20500801 0.0 1.0 httemp 3000205
*      name type scale ival iflag limit
20500810 "unnamed" sum 1.0 323.52145 0 0
*      a0 scale name param
20500811 0.0 1.0 httemp 3000105
*      name type scale ival iflag limit
20500820 "unnamed" sum 1.0 323.62451 0 0
*      a0 scale name param
20500821 0.0 1.0 httemp 3000305
*      name type scale ival iflag limit
20500830 "unnamed" sum 1.0 323.62512 0 0
*      a0 scale name param
20500831 0.0 1.0 httemp 3000405
*      name type scale ival iflag limit
20500840 "unnamed" sum 1.0 323.59042 0 0
*      a0 scale name param
20500841 0.0 1.0 httemp 3000505
*      name type scale ival iflag limit
20500850 "unnamed" sum 1.0 323.52289 0 0
*      a0 scale name param
20500851 0.0 1.0 httemp 3000605
*      name type scale ival iflag limit
20500860 "unnamed" sum 1.0 323.42776 0 0
*      a0 scale name param
20500861 0.0 1.0 httemp 3000705
*      name type scale ival iflag limit
20500870 "unnamed" sum 1.0 323.31241 0 0
*      a0 scale name param
20500871 0.0 1.0 httemp 3000805
*      name type scale ival iflag limit
20500880 "unnamed" sum 1.0 323.18585 0 0
*      a0 scale name param
20500881 0.0 1.0 httemp 3000905
*      name type scale ival iflag limit
20500890 "unnamed" sum 1.0 323.11505 0 0
*      a0 scale name param
20500891 0.0 1.0 httemp 3001005
*      name type scale ival iflag limit
20500900 "unnamed" sum 1.0 323.50284 0 0
*      a0 scale name param
20500901 0.0 1.0 httemp 2500113
*      name type scale ival iflag limit
20500910 "unnamed" sum 1.0 323.58917 0 0
*      a0 scale name param
20500911 0.0 1.0 httemp 2500213
*      name type scale ival iflag limit
20500920 "unnamed" sum 1.0 323.64075 0 0
*      a0 scale name param
20500921 0.0 1.0 httemp 2500313
*      name type scale ival iflag limit
20500930 "unnamed" sum 1.0 323.65314 0 0
*      a0 scale name param
20500931 0.0 1.0 httemp 2500413
*      name type scale ival iflag limit
20500940 "unnamed" sum 1.0 323.62518 0 0
*      a0 scale name param
20500941 0.0 1.0 httemp 2500513
*      name type scale ival iflag limit
20500950 "unnamed" sum 1.0 323.55893 0 0
*      a0 scale name param
20500951 0.0 1.0 httemp 2500613
*      name type scale ival iflag limit
20500960 "unnamed" sum 1.0 323.45953 0 0
*      a0 scale name param
20500961 0.0 1.0 httemp 2500713
*      name type scale ival iflag limit
20500970 "unnamed" sum 1.0 323.33487 0 0
*      a0 scale name param

20500971 0.0 1.0 httemp 2500813
*      name type scale ival iflag limit
20500980 "unnamed" sum 1.0 323.19482 0 0
*      a0 scale name param
20500981 0.0 1.0 httemp 2500913
*      name type scale ival iflag limit
20500990 "unnamed" sum 1.0 323.11862 0 0
*      a0 scale name param
20500991 0.0 1.0 httemp 2501013
*      name type scale ival iflag limit
20501000 "unnamed" sum 1.0 323.51813 0 0
*      a0 scale name param
20501001 0.0 1.0 httemp 2500105
*      name type scale ival iflag limit
20501010 "unnamed" sum 1.0 323.60406 0 0
*      a0 scale name param
20501011 0.0 1.0 httemp 2500205
*      name type scale ival iflag limit
20501020 "unnamed" sum 1.0 323.65451 0 0
*      a0 scale name param
20501021 0.0 1.0 httemp 2500305
*      name type scale ival iflag limit
20501030 "unnamed" sum 1.0 323.66528 0 0
*      a0 scale name param
20501031 0.0 1.0 httemp 2500405
*      name type scale ival iflag limit
20501040 "unnamed" sum 1.0 323.63525 0 0
*      a0 scale name param
20501041 0.0 1.0 httemp 2500505
*      name type scale ival iflag limit
20501050 "unnamed" sum 1.0 323.56671 0 0
*      a0 scale name param
20501051 0.0 1.0 httemp 2500605
*      name type scale ival iflag limit
20501060 "unnamed" sum 1.0 323.465 0 0
*      a0 scale name param
20501061 0.0 1.0 httemp 2500705
*      name type scale ival iflag limit
20501070 "unnamed" sum 1.0 323.3382 0 0
*      a0 scale name param
20501071 0.0 1.0 httemp 2500805
*      name type scale ival iflag limit
20501080 "unnamed" sum 1.0 323.19641 0 0
*      a0 scale name param
20501081 0.0 1.0 httemp 2500905
*      name type scale ival iflag limit
20501090 "unnamed" sum 1.0 323.11926 0 0
*      a0 scale name param
20501091 0.0 1.0 httemp 2501005
*      name type scale ival iflag limit
20501100 "unnamed" sum 1.0 323.6972 0 0
*      a0 scale name param
20501101 0.0 1.0 httemp 2500409
*      name type scale ival iflag limit
20501110 "unnamed" powerr 1.0 0.52376056 0 0
*      input param power
20501111 cntrlvar 114 -0.23
*      name type scale ival iflag limit
20501120 "unnamed" powerr 1.0 25.589294 0 0
*      input param power
20501121 htrnr 2500400 0.35
*      name type scale ival iflag limit
20501130 "T_ONBR" sum 1.0 478.85593 0 0
*      a0 scale name param
20501131 0.0 1.0 sattemp 25040000
20501132 0.182 cntrlvar 115
*      name type scale ival iflag limit
20501140 "unnamed" sum 1.0 16.640322 0 0
*      a0 scale name param
20501141 0.0 1.0e-5 p 25040000
*      name type scale ival iflag limit
20501150 "unnamed" mult 1.0 13.402663 0 0
*      input param input param
20501151 cntrlvar 111 cntrlvar 112
*      name type scale ival iflag limit
20501160 "unnamed" sum 1.0 0.60251117 0 0
*      a0 scale name param
20501161 0.0 1.0 httemp 2500401

```

20501162	-1.0	tempf	10010000	20200231	0.3	0.9		
*	name type scale	ival	iflag limit	20200232	0.35	0.975		
20501170	"unnamed"	sum	1.0 155.84138 0 0	20200233	0.36	0.983		
*	a0	scale	name param	20200234	0.37	0.99		
20501171	0.0	1.0	cntrivar 113	20200235	0.38	0.999		
20501172	-1.0	tempf	10010000	20200236	0.39	1.0		
*	name type scale	ival	iflag limit	*n: PB				
20501180	"unnamed"	powerr	1.0 1.6597203 0 0	*	type	trip	factor1	factor2
*	input	param	power	20200300	reac-t	2	1.0	-3.5
20501181	cntrivar	116	-1.0	*	Time	Reactivity		
*	name type scale	ival	iflag limit	20200301	0.0	0.0		
20501190	"ONBR"	mult	1.0 258.65311 0 0	20200302	0.01	0.0		
*	input	param	input param	20200303	0.02	0.0		
20501191	cntrivar	117	cntrivar 118	20200304	0.03	0.0		
20600000	expanded			20200305	0.04	0.0		
*****	Variable Trips			20200306	0.05	0.0		
*****	n: Pressure_signal			20200307	0.06	1.0e-3		
*	var	param r	var param acon l timeo	20200308	0.07	2.0e-3		
20600010	p	10010000	le null 0 9.0 l -1.0	20200309	0.08	3.0e-3		
*n: Scram_delay				20200310	0.09	4.0e-3		
*	var param r	var param acon l timeo		20200311	0.1	5.0e-3		
20600020	time	0 gt	timeof 1001 0.35 l -1.0	20200312	0.11	6.0e-3		
*n: A120 Power_signal				20200313	0.12	7.0e-3		
*	var param r	var param acon l timeo		20200314	0.13	8.0e-3		
20600030	rktpow	0 ge null	0 2.16e6 l -1.0	20200315	0.14	9.0e-3		
*n: LOOS OF COOLANT FLOW				20200316	0.15	0.01		
*	var param r	var param acon l timeo		20200317	0.16	0.013		
20600040	time	0 ge null	0 160.0 l -1.0	20200318	0.17	0.016		
*n: RIA_FAST				20200319	0.18	0.019		
*	var param r	var param acon l timeo		20200320	0.19	0.022		
20600050	time	0 ge null	0 160.0 l -1.0	20200321	0.2	0.025		
*n: RIA_SLOW				20200322	0.21	0.03		
*	var param r	var param acon l timeo		20200323	0.22	0.04		
20600060	time	0 ge null	0 160.0 l -1.0	20200324	0.23	0.05		
*****	Logical Trips			20200325	0.24	0.06		
*****	n: SCRAM			20200326	0.25	0.07		
*	trip1	oper	trip2 l timeo	20200327	0.26	0.085		
20610010	3	or	1 l -1.0	20200328	0.27	0.1		
*****	General Tables			20200329	0.28	0.118		
*****	n: ENVI			20200330	0.29	0.135		
*	type	trip		20200331	0.3	0.157		
20200100	temp	0		20200332	0.35	0.3		
*	Time	Temperature		20200333	0.36	0.32		
20200101	0.0	300.0		20200334	0.37	0.345		
*n: PK + PAR				20200335	0.38	0.38		
*	type	trip	factor1 factor2	20200336	0.39	0.41		
20200200	reac-t	2	1.0 -2.0	20200337	0.4	0.45		
*	Time	Reactivity		20200338	0.41	0.49		
20200201	0.0	0.0		20200339	0.42	0.515		
20200202	0.01	5.0e-3		20200340	0.43	0.55		
20200203	0.02	0.01		20200341	0.44	0.59		
20200204	0.03	0.022		20200342	0.45	0.62		
20200205	0.04	0.035		20200343	0.46	0.66		
20200206	0.05	0.05		20200344	0.47	0.7		
20200207	0.06	0.07		20200345	0.48	0.735		
20200208	0.07	0.09		20200346	0.49	0.77		
20200209	0.08	0.11		20200347	0.5	0.8		
20200210	0.09	0.145		20200348	0.51	0.825		
20200211	0.1	0.165		20200349	0.52	0.86		
20200212	0.11	0.2		20200350	0.53	0.89		
20200213	0.12	0.23		20200351	0.54	0.905		
20200214	0.13	0.275		20200352	0.55	0.925		
20200215	0.14	0.31		20200353	0.56	0.935		
20200216	0.15	0.35		20200354	0.57	0.95		
20200217	0.16	0.4		20200355	0.58	0.963		
20200218	0.17	0.45		20200356	0.59	0.975		
20200219	0.18	0.49		20200357	0.6	0.985		
20200220	0.19	0.525		20200358	0.63	1.0		
20200221	0.2	0.565		*n: Reactivity_Slow				
20200222	0.21	0.6		*	type	trip		
20200223	0.22	0.64		20200400	reac-t	6		
20200224	0.23	0.675		*	Time	Reactivity		
20200225	0.24	0.72		20200401	-1.0	0.0		
20200226	0.25	0.75		20200402	0.0	0.0		
20200227	0.26	0.79		20200403	1.0	0.04		
20200228	0.27	0.825		20200404	2.0	0.08		
20200229	0.28	0.85		20200405	3.0	0.12		
20200230	0.29	0.88		20200406	4.0	0.16		

20200407	5.0	0.2	20200529	0.27	1.08
20200408	6.0	0.24	20200530	0.28	1.12
20200409	7.0	0.28	20200531	0.29	1.16
20200410	8.0	0.32	20200532	0.3	1.2
20200411	9.0	0.36	20200533	0.31	1.24
20200412	10.0	0.4	20200534	0.32	1.28
20200413	11.0	0.44	20200535	0.33	1.32
20200414	12.0	0.48	20200536	0.34	1.36
20200415	13.0	0.52	20200537	0.35	1.4
20200416	14.0	0.56	20200538	0.36	1.44
20200417	15.0	0.6	20200539	0.37	1.48
20200418	16.0	0.64	20200540	0.375	1.5
20200419	17.0	0.68	*n: Power_for_SS		
20200420	18.0	0.72	* type trip		
20200421	19.0	0.76	20200600	power	0
20200422	20.0	0.8	* Time Power		
20200423	21.0	0.84	20200601	-1.0	1.8e6
20200424	22.0	0.88	20200602	0.0	1.8e6
20200425	23.0	0.92	20200603	1.0	1.8e6
20200426	24.0	0.96	20200604	1.0e6	1.8e6
20200427	25.0	1.0	***** Materials *****		
20200428	26.0	1.04	*****n:		
20200429	27.0	1.08	Fuel		
20200430	28.0	1.12	* type tflag vflag		
20200431	29.0	1.16	20100100	tbl/fctn	1 1
20200432	30.0	1.2	* thcond		
20200433	31.0	1.24	20100101	59.0	
20200434	32.0	1.28	* temp capacity		
20200435	33.0	1.32	20100151	273.0	0.35837293
20200436	34.0	1.36	20100152	303.0	0.35837293
20200437	35.0	1.4	20100153	313.0	0.37012693
20200438	36.0	1.44	20100154	323.0	0.38188093
20200439	37.0	1.48	20100155	333.0	0.39363493
20200440	38.0	1.52	20100156	343.0	0.40538893
20200441	39.0	1.56	20100157	353.0	0.41714293
20200442	40.0	1.6	20100158	363.0	0.42889693
20200443	41.0	1.64	20100159	373.0	0.44065093
20200444	42.0	1.68	20100160	383.0	0.45240493
20200445	43.0	1.72	20100161	393.0	0.46415893
20200446	44.0	1.76	20100162	403.0	0.47591293
20200447	45.0	1.8	20100163	413.0	0.48766693
20200448	46.0	1.84	20100164	423.0	0.49942093
20200449	47.0	1.88	20100165	433.0	0.51117493
20200450	48.0	1.92	20100166	443.0	0.52292893
20200451	49.0	1.96	20100167	453.0	0.53468293
20200452	50.0	2.0	20100168	463.0	0.54643693
*n: Reactivity_Fast			20100169	473.0	0.55819093
* type trip			20100170	523.0	0.61696093
20200500	reac-t	5	20100171	573.0	0.67573093
* Time Reactivity			20100172	623.0	0.73450093
20200501	-1.0	0.0	20100173	673.0	0.79327093
20200502	0.0	0.0	20100174	723.0	0.85204093
20200503	0.01	0.04	20100175	773.0	0.91081093
20200504	0.02	0.08	20100176	823.0	0.96958093
20200505	0.03	0.12	20100177	873.0	1.0283509
20200506	0.04	0.16	20100178	923.0	1.0871209
20200507	0.05	0.2	20100179	973.0	1.1458909
20200508	0.06	0.24	20100180	2000.0	1.1458909
*n: AGN cladding			*n: AGN cladding		
* type tflag vflag			20100200	tbl/fctn	1 1
* thcond			20100201	130.0	
20200511	0.09	0.36	* heat Capacity		
20200512	0.1	0.4	20100251	2.3496e6	
20200513	0.11	0.44	*n: Aluminium		
20200514	0.12	0.48	* type tflag vflag		
20200515	0.13	0.52	20100300	tbl/fctn	1 1
20200516	0.14	0.56	* temp thcond		
20200517	0.15	0.6	20100301	273.0	237.0
20200518	0.16	0.64	20100302	300.0	237.0
20200519	0.17	0.68	20100303	350.0	240.0
20200520	0.18	0.72	20100304	400.0	240.0
20200521	0.19	0.76	20100305	500.0	236.0
20200522	0.2	0.8	20100306	600.0	231.0
20200523	0.21	0.84	20100307	800.0	218.0
20200524	0.22	0.88	20100308	1200.0	218.0
20200525	0.23	0.92	* temp capacity		
20200526	0.24	0.96	20100351	273.0	2.4219e6
20200527	0.25	1.0			
20200528	0.26	1.04			

20100352 298.0 2.4219e6
 20100353 350.0 2.51262e6
 20100354 400.0 2.57985e6
 20100355 500.0 2.68596e6
 20100356 600.0 2.7918e6
 20100357 1000.0 2.7918e6
 *n: SAW
 * type tflag vflag
 20100400 tbl/fctn 1 1
 * temp thcond
 20100401 273.0 170.0
 20100402 900.0 170.0
 * temp capacity
 20100451 273.0 2.4192e6
 20100452 900.0 2.4192e6
 **** Hyd. Components
 ***** name type
 0050000 "unnamed" tmdpvol
 * area length vol
 0050101 20.0 20.0 0.0
 * az-angle inc-angle dz
 0050102 0.0 90.0 20.0
 * x-rough x-hd flags
 0050103 0.0 0.0 0
 * cword
 0050200 3
 * srch press temp
 0050201 0.0 1.4e6 323.15
 0050202 0.99 1.4e6 323.15
 0050203 1.0 1.4e6 323.15
 * name type
 0060000 "unnamed" tmdpjun
 * from to area jefvcahs
 0060101 5010002 10010001 2.294e-3 0
 * control
 0060200 1
 * srch mfl mfv unused
 0060201 0.0 7.41 0.0 0.0
 * name type
 0100000 "unnamed" pipe
 * ncells
 0100001 6
 * x-area void
 0100101 2.294e-3 6
 * x-length void
 0100301 0.3 1
 0100302 0.5 4
 0100303 0.4 5
 0100304 0.2 6
 * volume void
 0100401 0.0 6
 * azim-angle void
 0100501 0.0 6
 * vert-angle void
 0100601 -90.0 6
 * x-wall xhd void
 0100801 1.0e-5 0.027 6
 * x-flags void
 0101001 0 6
 * ebt press water-ie steam-ie void none id
 0101201 000 1.698744e6 2.085118e5 2.596097e6 0.0 0.0 1
 0101202 000 1.700844e6 2.085396e5 2.596122e6 0.0 0.0 2
 0101203 000 1.703468e6 2.085674e5 2.596154e6 0.0 0.0 3
 0101204 000 1.706092e6 2.085951e5 2.596185e6 0.0 0.0 4
 0101205 000 1.708454e6 2.086174e5 2.596214e6 0.0 0.0 5
 0101206 000 1.710028e6 2.086285e5 2.596232e6 0.0 0.0 6
 * jefvcahs jun num
 0101101 00000000 5
 * vl vv unused junid
 0101301 3.6371021 3.6371021 0.0 1
 0101302 3.63711 3.63711 0.0 2
 0101303 3.6371169 3.6371169 0.0 3
 0101304 3.6371241 3.6371241 0.0 4
 0101305 3.6371293 3.6371293 0.0 5
 * hd corr gas slope junid
 0101401 0.027 0.0 1.0 1.0 5
 * name type
 0150000 "unnamed" branch
 * njuns
 0150001 4
 * area length vol
 0150101 2.294e-3 0.1 0.0
 * az-angle inc-angle dz
 0150102 0.0 -90.0 -0.1
 * x-rough x-hd flags
 0150103 0.0 0.0 0
 * ebt press water-ie steam-ie void
 0150200 000 1.710953e6 2.086334e5 2.596243e6
 0.0
 * from to area
 0151101 15010001 10060002 2.294e-3
 * fwd. loss rev. loss efvcahs
 0151102 0.0 0.0 0
 * hyd diam ccfl wght ccfl gas ccfl slope
 0151110 0.027 0.0 1.0 1.0
 * vl vv unused
 0151201 -3.6371312 -3.6371312 0.0
 * from to area
 0152101 15010002 20010001 5.24e-4
 * fwd. loss rev. loss efvcahs
 0152102 0.4 0.4 0
 * vl vv unused
 0152201 6.1487789 6.1487789 0.0
 * from to area
 0153101 15010002 25010001 4.54e-4
 * fwd. loss rev. loss efvcahs
 0153102 0.4 0.4 0
 * vl vv unused
 0153201 6.1307321 6.1307321 0.0
 * from to area
 0154101 15010002 30010001 3.83e-4
 * fwd. loss rev. loss efvcahs
 0154102 0.4 0.4 0
 * vl vv unused
 0154201 6.1051373 6.1051373 0.0
 * name type
 0200000 "unnamed" pipe
 * ncells
 0200001 12
 * x-area void
 0200101 5.24e-4 12
 * x-length void
 0200301 0.05 1
 0200302 0.1 11
 0200303 0.05 12
 * volume void
 0200401 0.0 12
 * azim-angle void
 0200501 0.0 12
 * vert-angle void
 0200601 -90.0 12
 * x-wall xhd void
 0200801 1.0e-5 4.827e-3 12
 * x-flags void
 0201001 0 12
 * ebt press temp none none none id
 0201201 003 1.689443e6 323.045 0.0 0.0 0.0 1
 * ebt press water-ie steam-ie void none id
 0201202 000 1.682478e6 2.086602e5 2.595899e6 0.0 0.0 2
 * ebt press water-ie steam-ie void none id
 0201203 000 1.673191e6 2.086929e5 2.595784e6 0.0 0.0 3
 * ebt press water-ie steam-ie void none id
 0201204 000 1.663904e6 2.087463e5 2.595668e6 0.0 0.0 4
 * ebt press water-ie steam-ie void none id
 0201205 000 1.654618e6 2.088172e5 2.595551e6 0.0 0.0 5
 * ebt press water-ie steam-ie void none id
 0201206 000 1.645331e6 2.089011e5 2.595432e6 0.0 0.0 6
 * ebt press water-ie steam-ie void none id
 0201207 000 1.636045e6 2.089922e5 2.595312e6 0.0 0.0 7
 * ebt press water-ie steam-ie void none id
 0201208 000 1.626759e6 2.090844e5 2.595191e6 0.0 0.0 8
 * ebt press water-ie steam-ie void none id
 0201209 000 1.617473e6 2.091712e5 2.595068e6 0.0 0.0 9
 * ebt press water-ie steam-ie void none id
 0201210 000 1.608187e6 2.092468e5 2.594944e6 0.0 0.0 10
 * ebt press water-ie steam-ie void none id

0201211 000 1.598902e6 2.093061e5 2.594818e6 0.0 0.0 11
 * ebt press water-ie steam-ie void none id
 0201212 000 1.591937e6 2.093155e5 2.594723e6 0.0 0.0 12
 * area jun
 0200201 5.24e-4 11
 * jefvcahs jun num
 0201101 00000000 11
 * vl vv unused junid
 0201301 6.1488385 6.1488385 0.0 1
 0201302 6.1488714 6.1488714 0.0 2
 0201303 6.1489172 6.1489172 0.0 3
 0201304 6.1489778 6.1489778 0.0 4
 0201305 6.1490493 6.1490493 0.0 5
 0201306 6.1491299 6.1491299 0.0 6
 0201307 6.1492157 6.1492157 0.0 7
 0201308 6.149302 6.149302 0.0 8
 0201309 6.1493845 6.1493845 0.0 9
 0201310 6.1494598 6.1494598 0.0 10
 0201311 6.1495237 6.1495237 0.0 11
 * hd corr gas slope junid
 0201401 4.827e-3 0.0 1.0 1.0 11
 * name type
 0250000 "unnamed" pipe
 * ncells
 0250001 12
 * x-area volid
 0250101 4.54e-4 12
 * x-length volid
 0250301 0.05 1
 0250302 0.1 11
 0250303 0.05 12
 * volume volid
 0250401 0.0 12
 * azim-angle volid
 0250501 0.0 12
 * vert-angle volid
 0250601 -90.0 12
 * x-wall xhd volid
 0250801 1.0e-5 4.802e-3 12
 * x-flags volid
 0251001 0 12
 * ebt press water-ie steam-ie void none id
 0251201 000 1.689598e6 2.086388e5 2.595986e6 0.0 0.0 1
 0251202 000 1.682626e6 2.086712e5 2.595901e6 0.0 0.0 2
 0251203 000 1.673329e6 2.087246e5 2.595786e6 0.0 0.0 3
 0251204 000 1.664032e6 2.088165e5 2.595676e6 0.0 0.0 4
 0251205 000 1.654736e6 2.0894e5 2.595552e6 0.0 0.0 5
 0251206 000 1.645439e6 2.090854e5 2.595434e6 0.0 0.0 6
 0251207 000 1.636143e6 2.092415e5 2.595314e6 0.0 0.0 7
 0251208 000 1.626848e6 2.093963e5 2.595192e6 0.0 0.0 8
 0251209 000 1.617552e6 2.095376e5 2.595069e6 0.0 0.0 9
 0251210 000 1.608257e6 2.096547e5 2.594945e6 0.0 0.0 10
 0251211 000 1.598962e6 2.097386e5 2.594819e6 0.0 0.0 11
 0251212 000 1.591991e6 2.09739e5 2.594724e6 0.0 0.0 12
 * area jun
 0250201 4.54e-4 11
 * jefvcahs jun num
 0251101 00000000 11
 * vl vv unused junid
 0251301 6.1307912 6.1307912 0.0 1
 0251302 6.1308308 6.1308308 0.0 2
 0251303 6.1308908 6.1308908 0.0 3
 0251304 6.1309767 6.1309767 0.0 4
 0251305 6.1310835 6.1310835 0.0 5
 0251306 6.1312056 6.1312056 0.0 6
 0251307 6.1313348 6.1313348 0.0 7
 0251308 6.1314626 6.1314626 0.0 8
 0251309 6.1315818 6.1315818 0.0 9
 0251310 6.1316848 6.1316848 0.0 10
 0251311 6.1317654 6.1317654 0.0 11
 * hd corr gas slope junid
 0251401 4.802e-3 0.0 1.0 1.0 11
 * name type
 0300000 "unnamed" pipe
 * ncells
 0300001 12
 * x-area volid
 0300101 3.83e-4 12

* x-length volid
 0300301 0.05 1
 0300302 0.1 11
 0300303 0.05 12
 * volume volid
 0300401 0.0 12
 * azim-angle volid
 0300501 0.0 12
 * vert-angle volid
 0300601 -90.0 12
 * x-wall xhd volid
 0300801 1.0e-5 4.767e-3 12
 * x-flags volid
 0301001 0 12
 * ebt press water-ie steam-ie void none id
 0301201 000 1.689816e6 2.086581e5 2.595989e6 0.0 0.0 1
 0301202 000 1.682834e6 2.087158e5 2.595904e6 0.0 0.0 2
 0301203 000 1.673524e6 2.087883e5 2.595789e6 0.0 0.0 3
 0301204 000 1.664214e6 2.088898e5 2.595672e6 0.0 0.0 4
 0301205 000 1.654904e6 2.090142e5 2.595555e6 0.0 0.0 5
 0301206 000 1.645594e6 2.091535e5 2.595436e6 0.0 0.0 6
 0301207 000 1.636285e6 2.092983e5 2.595316e6 0.0 0.0 7
 0301208 000 1.626976e6 2.094388e5 2.595194e6 0.0 0.0 8
 0301209 000 1.617667e6 2.095653e5 2.595071e6 0.0 0.0 9
 0301210 000 1.608359e6 2.096694e5 2.594946e6 0.0 0.0 10
 0301211 000 1.599051e6 2.097441e5 2.59482e6 0.0 0.0 11
 0301212 000 1.592076e6 2.097471e5 2.594725e6 0.0 0.0 12
 * area jun
 0300201 3.83e-4 11
 * jefvcahs jun num
 0301101 00000000 11
 * vl vv unused junid
 0301301 6.1052089 6.1052089 0.0 1
 0301302 6.1052651 6.1052651 0.0 2
 0301303 6.1053376 6.1053376 0.0 3
 0301304 6.1054296 6.1054296 0.0 4
 0301305 6.1055369 6.1055369 0.0 5
 0301306 6.1056542 6.1056542 0.0 6
 0301307 6.1057749 6.1057749 0.0 7
 0301308 6.1058931 6.1058931 0.0 8
 0301309 6.1060019 6.1060019 0.0 9
 0301310 6.1060958 6.1060958 0.0 10
 0301311 6.1061702 6.1061702 0.0 11
 * hd corr gas slope junid
 0301401 4.767e-3 0.0 1.0 1.0 11
 * name type
 0350000 "unnamed" branch
 * njuns
 0350001 4
 * area length vol
 0350101 2.294e-3 0.05 0.0
 * az-angle inc-angle dz
 0350102 0.0 -90.0 -0.05
 * x-rough x-hd flags
 0350103 0.0 0.0 0
 * ebt press water-ie steam-ie void
 0350200 000 1.592632e6 2.095778e5 2.594733e6
 0.0
 * from to area
 0351101 35010002 40010001 2.294e-3
 * fwd. loss rev. loss efvcahs
 0351102 1.5 1.5 0
 * vl vv unused
 0351201 3.6376903 3.6376903 0.0
 * from to area
 0352101 35010001 20120002 5.24e-4
 * fwd. loss rev. loss efvcahs
 0352102 0.5 0.5 0
 * vl vv unused
 0352201 -6.1495485 -6.1495485 0.0
 * from to area
 0353101 35010001 25120002 4.54e-4
 * fwd. loss rev. loss efvcahs
 0353102 0.5 0.5 0
 * vl vv unused
 0353201 -6.131784 -6.131784 0.0
 * from to area
 0354101 35010001 30120002 3.83e-4

```

*      fwd. loss  rev. loss    efvcahs
0354102      0.5      0.5      0
*      vl      vv      unused
0354201 -6.1061902 -6.1061902      0.0
*      name      type
0400000 "unnamed" branch
*      njuns
0400001      0
*      area      length      vol
0400101 4.418e-3      0.05      0.0
*      az-angle      inc-angle      dz
0400102      0.0      -90.0      -0.05
*      x-rough      x-hd      flags
0400103      0.0      0.0      0
*      ebt      press      water-ie      steam-ie      void
0400200      000 1.589762e6 2.095778e5 2.594694e6
0.0
*      name      type
0450000 "unnamed" branch
*      njuns
0450001      5
*      area      length      vol
0450101 1.81e-3      0.05      0.0
*      az-angle      inc-angle      dz
0450102      0.0      90.0      0.05
*      x-rough      x-hd      flags
0450103      0.0      0.0      0
*      ebt      press      water-ie      steam-ie      void
0450200      000 1.562884e6 2.095779e5 2.594318e6
0.0
*      from      to      area
0451101 45010001 40010001 1.81e-3
*      fwd. loss  rev. loss    efvcahs
0451102      1.5      1.5      0
*      vl      vv      unused
0451201 -4.6104264 -4.6104264      0.0
*      from      to      area
0452101 45010002 50010001 3.12e-4
*      fwd. loss  rev. loss    efvcahs
0452102      0.5      0.5      0
*      vl      vv      unused
0452201 11.260211 11.260211      0.0
*      from      to      area
0453101 45010002 55010001 2.42e-4
*      fwd. loss  rev. loss    efvcahs
0453102      0.5      0.5      0
*      vl      vv      unused
0453201 11.155989 11.155989      0.0
*      from      to      area
0454101 45010002 60010001 1.71e-4
*      fwd. loss  rev. loss    efvcahs
0454102      0.5      0.5      0
*      vl      vv      unused
0454201 10.971682 10.971682      0.0
*      from      to      area
0455101 45010002 80010001 1.1e-5
*      fwd. loss  rev. loss    efvcahs
0455102      0.5      0.5      0
*      vl      vv      unused
0455201 23.26129 23.26129      0.0
*      name      type
0500000 "unnamed" pipe
*      ncells
0500001      12
*      x-area      valid
0500101 3.12e-4      12
*      x-length      valid
0500301      0.05      1
0500302      0.1      11
0500303      0.05      12
*      volume      valid
0500401      0.0      12
*      azim-angle      valid
0500501      0.0      12
*      vert-angle      valid
0500601      90.0      12
*      x-wall      xhd      valid
0500801 1.0e-5 4.717e-3      12
*      x-flags      valid
0501001      0      12
*      ebt      press      water-ie      steam-ie      void      none      id
0501201 000 1.470377e6 2.095963e5 2.592927e6 0.0 0.0 1
0501202 000 1.444268e6 2.096746e5 2.592505e6 0.0 0.0 2
0501203 000 1.409458e6 2.097678e5 2.591922e6 0.0 0.0 3
0501204 000 1.374648e6 2.098706e5 2.591312e6 0.0 0.0 4
0501205 000 1.339838e6 2.099767e5 2.590677e6 0.0 0.0 5
0501206 000 1.305028e6 2.100796e5 2.590014e6 0.0 0.0 6
0501207 000 1.270217e6 2.101732e5 2.589321e6 0.0 0.0 7
0501208 000 1.235407e6 2.102518e5 2.588597e6 0.0 0.0 8
0501209 000 1.200597e6 2.10311e5 2.587841e6 0.0 0.0 9
0501210 000 1.165786e6 2.103481e5 2.587056e6 0.0 0.0 10
0501211 000 1.130976e6 2.103731e5 2.586223e6 0.0 0.0 11
0501212 000 1.104867e6 2.10377e5 2.585578e6 0.0 0.0 12
*      area      jun
0500201 3.12e-4      11
*      jefvcahs      jun      num
0501101 00000000      11
*      vl      vv      unused      junid
0501301 11.260675 11.260675      0.0      1
0501302 11.260897 11.260897      0.0      2
0501303 11.261178 11.261178      0.0      3
0501304 11.261471 11.261471      0.0      4
0501305 11.261768 11.261768      0.0      5
0501306 11.262062 11.262062      0.0      6
0501307 11.262344 11.262344      0.0      7
0501308 11.262608 11.262608      0.0      8
0501309 11.262847 11.262847      0.0      9
0501310 11.26306 11.26306      0.0      10
0501311 11.263257 11.263257      0.0      11
*      hd      corr      gas      slope      junid
0501401 4.717e-3      0.0      1.0      1.0      1.0      11
*      name      type
0550000 "unnamed" pipe
*      ncells
0550001      12
*      x-area      valid
0550101 2.42e-4      12
*      x-length      valid
0550301      0.05      1
0550302      0.1      11
0550303      0.05      12
*      volume      valid
0550401      0.0      12
*      azim-angle      valid
0550501      0.0      12
*      vert-angle      valid
0550601      90.0      12
*      x-wall      xhd      valid
0550801 1.0e-5 4.64e-3      12
*      x-flags      valid
0551001      0      12
*      ebt      press      water-ie      steam-ie      void      none      id
0551201 000 1.472093e6 2.09595e5 2.592954e6 0.0 0.0 1
0551202 000 1.445911e6 2.096632e5 2.592532e6 0.0 0.0 2
0551203 000 1.411005e6 2.097448e5 2.591948e6 0.0 0.0 3
0551204 000 1.376099e6 2.098365e5 2.591338e6 0.0 0.0 4
0551205 000 1.341192e6 2.099321e5 2.590702e6 0.0 0.0 5
0551206 000 1.306286e6 2.10028e5 2.590038e6 0.0 0.0 6
0551207 000 1.271379e6 2.101188e5 2.589344e6 0.0 0.0 7
0551208 000 1.236472e6 2.101997e5 2.58862e6 0.0 0.0 8
0551209 000 1.201565e6 2.102668e5 2.587862e6 0.0 0.0 9
0551210 000 1.166658e6 2.103176e5 2.587076e6 0.0 0.0 10
0551211 000 1.131751e6 2.103593e5 2.586242e6 0.0 0.0 11
0551212 000 1.10557e6 2.103754e5 2.585596e6 0.0 0.0 12
*      area      jun
0550201 2.42e-4      11
*      jefvcahs      jun      num
0551101 00000000      11
*      vl      vv      unused      junid
0551301 11.156439 11.156439      0.0      1
0551302 11.156647 11.156647      0.0      2
0551303 11.156912 11.156912      0.0      3
0551304 11.157188 11.157188      0.0      4
0551305 11.157471 11.157471      0.0      5
0551306 11.157754 11.157754      0.0      6
0551307 11.158031 11.158031      0.0      7

```

0551308	11.158295	11.158295	0.0	8		*	fwd. loss	rev. loss	efvcahs			
0551309	11.158542	11.158542	0.0	9		0652102	0.4	0.4	0			
0551310	11.15877	11.15877	0.0	10		*	vl	vv	unused			
0551311	11.158986	11.158986	0.0	11		0652201	-11.263387	-11.263387	0.0			
*	hd corr	gas	slope	junid		*	from	to	area			
0551401	4.64e-3	0.0	1.0	1.0	11	0653101	65010001	55120002	2.42e-4			
*	name type					*	fwd. loss	rev. loss	efvcahs			
0600000	"unnamed"	pipe				0653102	0.4	0.4	0			
*	ncells					*	vl	vv	unused			
0600001	12					0653201	-11.159129	-11.159129	0.0			
*	x-area	void				*	from	to	area			
0600101	1.71e-4		12			0654101	65010001	60120002	1.71e-4			
*	x-length	void				*	fwd. loss	rev. loss	efvcahs			
0600301	0.05	1				0654102	0.4	0.4	0			
0600302	0.1	11				*	vl	vv	unused			
0600303	0.05	12				0654201	-10.974606	-10.974606	0.0			
*	volume	void				*	from	to	area			
0600401	0.0	12				0655101	65010001	80120002	1.1e-5			
*	azim-angle	void				*	fwd. loss	rev. loss	efvcahs			
0600501	0.0	12				0655102	0.4	0.4	0			
*	vert-angle	void				*	vl	vv	unused			
0600601	90.0	12				0655201	-23.267387	-23.267387	0.0			
*	x-wall	xhd	void			*	name type					
0600801	1.0e-5	4.506e-3		12		0700000	"unnamed"	pipe				
*	x-flags	void				*	ncells					
0601001	0	12				0700001	6					
*	ebt	press	water-ie	steam-ie	void	*	x-area	void				
0601201	000	1.475083e6	2.095953e5	2.593001e6	0.0 0.0 1	0700101	1.81e-3		6			
0601202	000	1.448776e6	2.096526e5	2.592579e6	0.0 0.0 2	*	x-length	void				
0601203	000	1.413704e6	2.097188e5	2.591994e6	0.0 0.0 3	0700301	0.2	1				
0601204	000	1.37863e6	2.097916e5	2.591384e6	0.0 0.0 4	0700302	0.4	2				
0601205	000	1.343557e6	2.098679e5	2.590746e6	0.0 0.0 5	0700303	0.5	5				
0601206	000	1.308484e6	2.099442e5	2.590081e6	0.0 0.0 6	0700304	0.3	6				
0601207	000	1.27341e6	2.100174e5	2.589386e6	0.0 0.0 7	*	volume	void				
0601208	000	1.238336e6	2.100842e5	2.588659e6	0.0 0.0 8	0700401	0.0	6				
0601209	000	1.203262e6	2.10142e5	2.5879e6	0.0 0.0 9	*	azim-angle	void				
0601210	000	1.168188e6	2.10189e5	2.587106e6	0.0 0.0 10	0700501	0.0	6				
0601211	000	1.133113e6	2.102301e5	2.586275e6	0.0 0.0 11	*	vert-angle	void				
0601212	000	1.106807e6	2.102442e5	2.585627e6	0.0 0.0 12	0700601	90.0	6				
*	area	jun			*	x-wall	xhd	void				
0600201	1.71e-4		11		0700801	1.0e-5	0.0	6				
*	jefvcahs	jun num			*	x-flags	void					
0601101	00000000	11			0701001	0	6					
*	vl	vv	unused	junid	*	ebt	press	water-ie	steam-ie	void	none	id
0601301	10.972111	10.972111	0.0	1	0701201	000	1.120749e6	2.103169e5	2.585973e6	0.0	0.0	1
0601302	10.972301	10.972301	0.0	2	0701202	000	1.116802e6	2.102979e5	2.585876e6	0.0	0.0	2
0601303	10.972545	10.972545	0.0	3	0701203	000	1.110882e6	2.102742e5	2.585728e6	0.0	0.0	3
0601304	10.972795	10.972795	0.0	4	0701204	000	1.104304e6	2.102504e5	2.585564e6	0.0	0.0	4
0601305	10.97305	10.97305	0.0	5	0701205	000	1.097726e6	2.102265e5	2.585398e6	0.0	0.0	5
0601306	10.973306	10.973306	0.0	6	0701206	000	1.092463e6	2.102121e5	2.585264e6	0.0	0.0	6
0601307	10.973557	10.973557	0.0	7	*	jefvcahs	jun num					
0601308	10.973801	10.973801	0.0	8	0701101	00000000	5					
0601309	10.974033	10.974033	0.0	9	*	vl	vv	unused	junid			
0601310	10.974254	10.974254	0.0	10	0701301	4.6117172	4.6117172	0.0				1
0601311	10.974466	10.974466	0.0	11	0701302	4.6117153	4.6117153	0.0				2
*	hd corr	gas	slope	junid	0701303	4.6117148	4.6117148	0.0				3
0601401	4.506e-3	0.0	1.0	1.0	0701304	4.6117158	4.6117158	0.0				4
*	name type				0701305	4.6117167	4.6117167	0.0				5
0650000	"unnamed"	branch			*	name type						
*	njuns				0740000	"unnamed"	sngljun					
0650001	5				*	from	to	area				
*	area length	vol			0740101	70060002	75010001	0.0				
0650101	1.81e-3	0.1	0.0		*	fwd. loss	rev. loss	efvcahs				
*	az-angle inc-angle	dz			0740102	0.0	0.0	0				
0650102	0.0	90.0	0.1		*	discharge	thermal					
*	x-rough	x-hd	flags		0740103	1.0	0.14					
0650103	0.0	0.0	0		*	flow	vl	vv	unused			
*	ebt	press	water-ie	steam-ie	0740201	0	4.6117196	4.6117196		0.0		
0650200	000	1.122699e6	2.103265e5	2.586021e6	*	name type						
0.0					0750000	"unnamed"	tmdpvol					
*	from	to	area		*	area length	vol					
0651101	65010002	70010001	1.81e-3		0750101	20.0	20.0	0.0				
*	fwd. loss	rev. loss	efvcahs		*	az-angle inc-angle	dz					
0651102	0.0	0.0	0		0750102	0.0	90.0	20.0				
*	vl	vv	unused		*	x-rough	x-hd	flags				
0651201	4.6117187	4.6117187	0.0		0750103	0.0	0.0	0				
*	from	to	area		*	cword						
0652101	65010001	50120002	3.12e-4		0750200	3						

```

*      srch    press   temp
0750201     0.0    1.101e6   300.0
*      name    type
0800000 "unnamed"    pipe
*      ncells
0800001     12
*      x-area    volid
0800101    1.1e-5     1
0800102    3.46e-4    11
0800103    1.1e-5    12
*      x-length    volid
0800301     0.05     1
0800302     0.1      11
0800303     0.05    12
*      volume    volid
0800401     0.0      12
*      azim-angle    volid
0800501     0.0      12
*      vert-angle    volid
0800601    90.0     12
*      x-wall    xhd    volid
0800801    1.0e-5    0.0     12
*      x-flags    volid
0801001     0      12
*      ebt    press water-ie steam-ie void none id
0801201 000 1.12475e6 2.096705e5 2.586072e6 0.0 0.0 1
0801202 000 1.34429e6 2.096706e5 2.59076e6 0.0 0.0 2
0801203 000 1.343289e6 2.096722e5 2.590741e6 0.0 0.0 3
0801204 000 1.342287e6 2.096755e5 2.590723e6 0.0 0.0 4
0801205 000 1.341286e6 2.096807e5 2.590704e6 0.0 0.0 5
0801206 000 1.340284e6 2.096875e5 2.590686e6 0.0 0.0 6
0801207 000 1.339282e6 2.09696e5 2.590667e6 0.0 0.0 7
0801208 000 1.338281e6 2.09706e5 2.590648e6 0.0 0.0 8
0801209 000 1.337279e6 2.09717e5 2.590629e6 0.0 0.0 9
0801210 000 1.336277e6 2.097289e5 2.59061e6 0.0 0.0 10
0801211 000 1.335276e6 2.097414e5 2.590592e6 0.0 0.0 11
0801212 000 1.020448e6 2.098705e5 2.583331e6 0.0 0.0 12
*      area    jun
0800201    3.46e-4    11
*      jefvcahs    jun num
0801101 00000000     11
*      vl    vv    unused    junid
0801301    0.73966569  0.73966569     0.0     1
0801302    0.73959684  0.73959684     0.0     2
0801303    0.73959726  0.73959726     0.0     3
0801304    0.73959786  0.73959786     0.0     4
0801305    0.73959857  0.73959857     0.0     5
0801306    0.73959947  0.73959947     0.0     6
0801307    0.73960048  0.73960048     0.0     7
0801308    0.73960161  0.73960161     0.0     8
0801309    0.7396028  0.7396028     0.0     9
0801310    0.73960406  0.73960406     0.0    10
0801311    0.73960543  0.73960543     0.0    11
***** Heat Structures *****
***** nh np geom ssif leftcoord *****
reflood
10010000 8 7 2 1 0.024 0
*      mesh    format
10010100     0      1
*      intervals    radius
10010101     6     0.026
*      material    interval
10010201     3      6
*      rpkf    interval
10010301     0.0     6
*      temp    interval
10010401    300.0    7
*      Left Boundary Condition Data
*      bound    incr    type    code    factor    node
10010501 70060000     0 101     1 0.3     1
10010502 70050000     0 101     1 0.5     2
10010503 70040000     0 101     1 0.5     3
10010504 70030000     0 101     1 0.5     4
10010505 70020000     0 101     1 0.4     5
10010506 70010000     0 101     1 0.2     6
10010507 65010000     0 101     1 0.1     7
10010508 45010000     0 101     1 0.05    8
*      Right Boundary Condition Data
*      bound    incr    type    code    factor    node
10010601 10010000     0 101     1 0.3     1
10010602 10020000     0 101     1 0.5     2
10010603 10030000     0 101     1 0.5     3
10010604 10040000     0 101     1 0.5     4
10010605 10050000     0 101     1 0.4     5
10010606 10060000     0 101     1 0.2     6
10010607 15010000     0 101     1 0.1     7
10010608 35010000     0 101     1 0.05    8
*      source    mult    dmhl    dmhr    num
10010701     0     0.0     0.0     0.0     8
*      Left Additional Boundary Condition Data
10010800     0
*      hthd    hlf    hlr    gslf    gslr    glcf    glcr    lbf    node
10010801 0.0 10.0 10.0 0.0 0.0 0.0 0.0 1.0 8
*      Right Additional Boundary Condition Data
10010900     0
*      hthd    hlf    hlr    gslf    gslr    glcf    glcr    lbf    node
10010901 0.0 10.0 10.0 0.0 0.0 0.0 0.0 1.0 8
*n: 5
*      nh np    geom    ssif    leftcoord    reflood
10025000 10 17     2 1 0.033 0
*      mesh    format
10025100     0      1
*      intervals    radius
10025101     4     0.0336
10025102     8     0.0344
10025103     4     0.035
*      material    interval
10025201     2      4
10025202     1     12
10025203     2     16
*      rpkf    interval
10025301     0.0     4
10025302     1.0     12
10025303     0.0     16
*      temp    interval
10025401    300.0    17
*      Left Boundary Condition Data
*      bound    incr    type    code    factor    node
10025501 25110000     0 101     1 0.1     1
10025502 25100000     0 101     1 0.1     2
10025503 25090000     0 101     1 0.1     3
10025504 25080000     0 101     1 0.1     4
10025505 25070000     0 101     1 0.1     5
10025506 25060000     0 101     1 0.1     6
10025507 25050000     0 101     1 0.1     7
10025508 25040000     0 101     1 0.1     8
10025509 25030000     0 101     1 0.1     9
10025510 25020000     0 101     1 0.1    10
*      Right Boundary Condition Data
*      bound    incr    type    code    factor    node
10025601 20110000     0 101     1 0.1     1
10025602 20100000     0 101     1 0.1     2
10025603 20090000     0 101     1 0.1     3
10025604 20080000     0 101     1 0.1     4
10025605 20070000     0 101     1 0.1     5
10025606 20060000     0 101     1 0.1     6
10025607 20050000     0 101     1 0.1     7
10025608 20040000     0 101     1 0.1     8
10025609 20030000     0 101     1 0.1     9
10025610 20020000     0 101     1 0.1    10
*      source    mult    dmhl    dmhr    num
10025701    1000 0.026176409 0.0 0.0 1
10025702    1000 0.036591412 0.0 0.0 2
10025703    1000 0.044041994 0.0 0.0 3
10025704    1000 0.047924554 0.0 0.0 5
10025705    1000 0.044041994 0.0 0.0 6
10025706    1000 0.036591412 0.0 0.0 7
10025707    1000 0.026176409 0.0 0.0 8
10025708    1000 0.013640753 0.0 0.0 9
10025709    1000 6.890509e-3 0.0 0.0 10
*      Left Additional Boundary Condition Data
10025800     0
*      hthd    hlf    hlr    gslf    gslr    glcf    glcr    lbf    node
10025801 9.816e-3 10.0 10.0 0.0 0.0 0.0 0.0 1.0 10
*      Right Additional Boundary Condition Data
10025900     0

```

```

*      hthd hlf hlr gslf gslr glcf glcr lbf node
10025901 0.010697 10.0 10.0 0.0 0.0 0.0 0.0 1.0 10
*      nh np geom ssif leftcoord reflood
10026000 2 7 2 1 0.033 0
*      mesh format
10026100 0 1
*      intervals radius
10026101 6 0.035
*      material interval
10026201 2 6
*      rpkf interval
10026301 0.0 6
*      temp interval
10026401 300.0 7
*      Left Boundary Condition Data
*      bound incr type code factor node
10026501 25010000 0 101 1 0.05 1
10026502 25120000 0 101 1 0.05 2
*      Right Boundary Condition Data
*      bound incr type code factor node
10026601 20010000 0 101 1 0.05 1
10026602 20120000 0 101 1 0.05 2
*      source mult dmhl dmhr num
10026701 0 0.0 0.0 0.0 2
*      Left Additional Boundary Condition Data
10026800 0
*      hthd hlf hlr gsf gsr glcf glcr lbf node
10026801 0.0 10.0 10.0 0.0 0.0 0.0 0.0 1.0 2
*      Right Additional Boundary Condition Data
10026900 0
*      hthd hlf hlr gsf gsr glcf glcr lbf node
10026901 0.0 10.0 10.0 0.0 0.0 0.0 0.0 0.0 1.0 2
*n: 4
*      nh np geom ssif leftcoord reflood
10030000 10 17 2 1 0.0285 0
*      mesh format
10030100 0 1
*      intervals radius
10030101 4 0.0291
10030102 8 0.0299
10030103 4 0.0305
*      material interval
10030201 2 4
10030202 1 12
10030203 2 16
*      rpkf interval
10030301 0.0 4
10030302 1.0 12
10030303 0.0 16
*      temp interval
10030401 300.0 17
*      Left Boundary Condition Data
*      bound incr type code factor node
10030501 30110000 0 101 1 0.1 1
10030502 30100000 0 101 1 0.1 2
10030503 30090000 0 101 1 0.1 3
10030504 30080000 0 101 1 0.1 4
10030505 30070000 0 101 1 0.1 5
10030506 30060000 0 101 1 0.1 6
10030507 30050000 0 101 1 0.1 7
10030508 30040000 0 101 1 0.1 8
10030509 30030000 0 101 1 0.1 9
10030510 30020000 0 101 1 0.1 10
*      Right Boundary Condition Data
*      bound incr type code factor node
10030601 25110000 0 101 1 0.1 1
10030602 25100000 0 101 1 0.1 2
10030603 25090000 0 101 1 0.1 3
10030604 25080000 0 101 1 0.1 4
10030605 25070000 0 101 1 0.1 5
10030606 25060000 0 101 1 0.1 6
10030607 25050000 0 101 1 0.1 7
10030608 25040000 0 101 1 0.1 8
10030609 25030000 0 101 1 0.1 9
10030610 25020000 0 101 1 0.1 10
*      source mult dmhl dmhr num
10030701 1000 0.018878743 0.0 0.0 1
10030702 1000 0.02639017 0.0 0.0 2
10030703 1000 0.03176362 0.0 0.0 3
10030704 1000 0.034563769 0.0 0.0 5
10030705 1000 0.03176362 0.0 0.0 6
10030706 1000 0.02639017 0.0 0.0 7
10030707 1000 0.018878743 0.0 0.0 8
10030708 1000 9.837876e-3 0.0 0.0 9
10030709 1000 4.969519e-3 0.0 0.0 10
*      Left Additional Boundary Condition Data
10030800 0
*      hthd hlf hlr gsf gsr glcf glcr lbf node
10030801 9.784e-3 10.0 10.0 0.0 0.0 0.0 0.0 1.0 10
*      Right Additional Boundary Condition Data
10030900 0
*      hthd hlf hlr gsf gsr glcf glcr lbf node
10030901 0.01083 10.0 10.0 0.0 0.0 0.0 0.0 1.0 10
*      nh np geom ssif leftcoord reflood
10031000 2 7 2 1 0.0285 0
*      mesh format
10031100 0 1
*      intervals radius
10031101 6 0.0305
*      material interval
10031201 2 6
*      rpkf interval
10031301 0.0 6
*      temp interval
10031401 300.0 7
*      Left Boundary Condition Data
*      bound incr type code factor node
10031501 30010000 0 101 1 0.05 1
10031502 30120000 0 101 1 0.05 2
*      Right Boundary Condition Data
*      bound incr type code factor node
10031601 25010000 0 101 1 0.05 1
10031602 25120000 0 101 1 0.05 2
*      source mult dmhl dmhr num
10031701 0 0.0 0.0 0.0 2
*      Left Additional Boundary Condition Data
10031800 0
*      hthd hlf hlr gsf gsr glcf glcr lbf node
10031801 0.0 10.0 10.0 0.0 0.0 0.0 0.0 1.0 2
*      Right Additional Boundary Condition Data
10031900 0
*      hthd hlf hlr gsf gsr glcf glcr lbf node
10031901 0.0 10.0 10.0 0.0 0.0 0.0 0.0 0.0 1.0 2
*n: 3
*      nh np geom ssif leftcoord reflood
10050000 10 17 2 1 0.024 0
*      mesh format
10050100 0 1
*      intervals radius
10050101 4 0.0246
10050102 8 0.0254
10050103 4 0.026
*      material interval
10050201 2 4
10050202 1 12
10050203 2 16
*      rpkf interval
10050301 0.0 4
10050302 1.0 12
10050303 0.0 16
*      temp interval
10050401 300.0 17
*      Left Boundary Condition Data
*      bound incr type code factor node
10050501 50020000 0 101 1 0.1 1
10050502 50030000 0 101 1 0.1 2
10050503 50040000 0 101 1 0.1 3
10050504 50050000 0 101 1 0.1 4
10050505 50060000 0 101 1 0.1 5
10050506 50070000 0 101 1 0.1 6
10050507 50080000 0 101 1 0.1 7
10050508 50090000 0 101 1 0.1 8
10050509 50100000 0 101 1 0.1 9
10050510 50110000 0 101 1 0.1 10
*      Right Boundary Condition Data
*      bound incr type code factor node

```

	bound	incr	type	code	factor	node
10050601	30110000	0	101	1	0.1	1
10050602	30100000	0	101	1	0.1	2
10050603	30090000	0	101	1	0.1	3
10050604	30080000	0	101	1	0.1	4
10050605	30070000	0	101	1	0.1	5
10050606	30060000	0	101	1	0.1	6
10050607	30050000	0	101	1	0.1	7
10050608	30040000	0	101	1	0.1	8
10050609	30030000	0	101	1	0.1	9
10050610	30020000	0	101	1	0.1	10
*	source	mult	dmhl	dmhr	num	
10050701	1000	0.014357363	0.0	0.0	1	
10050702	1000	0.020069835	0.0	0.0	2	
10050703	1000	0.024156366	0.0	0.0	3	
10050704	1000	0.026285892	0.0	0.0	5	
10050705	1000	0.024156366	0.0	0.0	6	
10050706	1000	0.020069835	0.0	0.0	7	
10050707	1000	0.014357363	0.0	0.0	8	
10050708	1000	7.481746e-3	0.0	0.0	9	
10050709	1000	3.77934e-3	0.0	0.0	10	
*	Left Additional Boundary Condition Data					
10050800	0					
*	hthd hlf hlr gslf gslr glcf glcr lbf node					
10050801	9.742e-3	10.0	10.0	0.0	0.0	0.0 1.0 10
*	Right Additional Boundary Condition Data					
10050900	0					
*	hthd hlf hlr gslf gslr glcf glcr lbf node					
10050901	0.011028	10.0	10.0	0.0	0.0	0.0 0.0 1.0 10
*	nh np geom ssif leftcoord reflood					
10051000	2	7	2	1	0.024	0
*	mesh format					
10051100	0	1				
*	intervals radius					
10051101	6	0.026				
*	material interval					
10051201	2	6				
*	rpkf interval					
10051301	0.0	6				
*	temp interval					
10051401	300.0	7				
*	Left Boundary Condition Data					
*	bound incr type code factor node					
10051501	50010000	0	101	1	0.05	1
10051502	50120000	0	101	1	0.05	2
*	Right Boundary Condition Data					
*	bound incr type code factor node					
10051601	30120000	0	101	1	0.05	1
10051602	30010000	0	101	1	0.05	2
*	source mult dmhl dmhr num					
10051701	0	0.0	0.0	0.0	0.0	2
*	Left Additional Boundary Condition Data					
10051800	0					
*	hthd hlf hlr gslf gslr glcf glcr lbf node					
10051801	0.0	10.0	10.0	0.0	0.0	0.0 0.0 1.0 2
*	Right Additional Boundary Condition Data					
10051900	0					
*	hthd hlf hlr gslf gslr glcf glcr lbf node					
10051901	0.0	10.0	10.0	0.0	0.0	0.0 0.0 1.0 2
*n: 2						
*	nh np geom ssif leftcoord reflood					
10055000	10	17	2	1	0.0195	0
*	mesh format					
10055100	0	1				
*	intervals radius					
10055101	4	0.0201				
10055102	8	0.0209				
10055103	4	0.0215				
*	material interval					
10055201	2	4				
10055202	1	12				
10055203	2	16				
*	rpkf interval					
10055301	0.0	4				
10055302	1.0	12				
10055303	0.0	16				
*	temp interval					
10055401	300.0	17				
*	Left Boundary Condition Data					
*	bound incr type code factor node					
10055501	55020000	0	101	1	0.1	1
10055502	55030000	0	101	1	0.1	2
10055503	55040000	0	101	1	0.1	3
10055504	55050000	0	101	1	0.1	4
10055505	55060000	0	101	1	0.1	5
10055506	55070000	0	101	1	0.1	6
10055507	55080000	0	101	1	0.1	7
10055508	55090000	0	101	1	0.1	8
10055509	55100000	0	101	1	0.1	9
10055510	55110000	0	101	1	0.1	10
*	Right Boundary Condition Data					
*	bound incr type code factor node					
10055601	50020000	0	101	1	0.1	1
10055602	50030000	0	101	1	0.1	2
10055603	50040000	0	101	1	0.1	3
10055604	50050000	0	101	1	0.1	4
10055605	50060000	0	101	1	0.1	5
10055606	50070000	0	101	1	0.1	6
10055607	50080000	0	101	1	0.1	7
10055608	50090000	0	101	1	0.1	8
10055609	50100000	0	101	1	0.1	9
10055610	50110000	0	101	1	0.1	10
*	source mult dmhl dmhr num					
10055701	1000	0.011184465	0.0	0.0	1	
10055702	1000	0.015634512	0.0	0.0	2	
10055703	1000	0.018817943	0.0	0.0	3	
10055704	1000	0.020476855	0.0	0.0	5	
10055705	1000	0.018817943	0.0	0.0	6	
10055706	1000	0.015634512	0.0	0.0	7	
10055707	1000	0.011184465	0.0	0.0	8	
10055708	1000	5.828322e-3	0.0	0.0	9	
10055709	1000	2.944127e-3	0.0	0.0	10	
*	Left Additional Boundary Condition Data					
10055800	0					
*	hthd hlf hlr gslf gslr glcf glcr lbf node					
10055801	0.09676	10.0	10.0	0.0	0.0	0.0 0.0 1.0 10
*	Right Additional Boundary Condition Data					
10055900	0					
*	hthd hlf hlr gslf gslr glcf glcr lbf node					
10055901	0.011343	10.0	10.0	0.0	0.0	0.0 0.0 1.0 10
*	nh np geom ssif leftcoord reflood					
10056000	2	7	2	1	0.0195	0
*	mesh format					
10056100	0	1				
*	intervals radius					
10056101	6	0.0215				
*	material interval					
10056201	2	6				
*	rpkf interval					
10056301	0.0	6				
*	temp interval					
10056401	300.0	7				
*	Left Boundary Condition Data					
*	bound incr type code factor node					
10056501	55120000	0	101	1	0.05	1
10056502	55010000	0	101	1	0.05	2
*	Right Boundary Condition Data					
*	bound incr type code factor node					
10056601	50120000	0	101	1	0.05	1
10056602	50010000	0	101	1	0.05	2
*	source mult dmhl dmhr num					
10056701	0	0.0	0.0	0.0	0.0	2
*	Left Additional Boundary Condition Data					
10056800	0					
*	hthd hlf hlr gslf gslr glcf glcr lbf node					
10056801	0.0	10.0	10.0	0.0	0.0	0.0 0.0 1.0 2
*	Right Additional Boundary Condition Data					
10056900	0					
*	hthd hlf hlr gslf gslr glcf glcr lbf node					
10056901	0.0	10.0	10.0	0.0	0.0	0.0 0.0 1.0 2
*n: 1						
*	nh np geom ssif leftcoord reflood					
10060000	10	17	2	1	0.015	0
*	mesh format					
10060100	0	1				
*	intervals radius					
10060101	4	0.0156				

```

10060102     8    0.0164
10060103     4    0.017
*      material   interval
10060201     2     4
10060202     1    12
10060203     2    16
*      rpkf   interval
10060301    0.0     4
10060302    1.0    12
10060303    0.0    16
*      temp   interval
10060401   300.0    17
* Left Boundary Condition Data
*      bound  incr  type  code   factor  node
10060501 60020000  0   101   1    0.1    1
10060502 60030000  0   101   1    0.1    2
10060503 60040000  0   101   1    0.1    3
10060504 60050000  0   101   1    0.1    4
10060505 60060000  0   101   1    0.1    5
10060506 60070000  0   101   1    0.1    6
10060507 60080000  0   101   1    0.1    7
10060508 60090000  0   101   1    0.1    8
10060509 60100000  0   101   1    0.1    9
10060510 60110000  0   101   1    0.1   10
* Right Boundary Condition Data
*      bound  incr  type  code   factor  node
10060601 55020000  0   101   1    0.1    1
10060602 55030000  0   101   1    0.1    2
10060603 55040000  0   101   1    0.1    3
10060604 55050000  0   101   1    0.1    4
10060605 55060000  0   101   1    0.1    5
10060606 55070000  0   101   1    0.1    6
10060607 55080000  0   101   1    0.1    7
10060608 55090000  0   101   1    0.1    8
10060609 55100000  0   101   1    0.1    9
10060610 55110000  0   101   1    0.1   10
*      source  mult  dmhl   dmhr   num
10060701   1000  8.72547e-3 0.0     0.0    1
10060702   1000  0.012197137 0.0     0.0    2
10060703   1000  0.014680665 0.0     0.0    3
10060704   1000  0.015974851 0.0     0.0    5
10060705   1000  0.014680665 0.0     0.0    6
10060706   1000  0.012197137 0.0     0.0    7
10060707   1000  8.72547e-3 0.0     0.0    8
10060708   1000  4.546918e-3 0.0     0.0    9
10060709   1000  2.296836e-3 0.0     0.0   10
* Left Additional Boundary Condition Data
10060800     0
*      hthd  hlf  hlr  gslf  gslr  glcf  glcr  lbf  node
10060801 9.563e-3 10.0 10.0  0.0  0.0  0.0  0.0  1.0  10
* Right Additional Boundary Condition Data
10060900     0
*      hthd  hlf  hlr  gslf  gslr  glcf  glcr  lbf  node
10060901 0.011926 10.0 10.0  0.0  0.0  0.0  0.0  0.1  10
*      nh  np   geom   ssif   leftcoord reflood
10061000 2   7    2    1    0.015  0
*      mesh   format
10061100     0   1
*      intervals   radius
10061101     6   0.017
*      material   interval
10061201     2   6
*      rpkf   interval
10061301    0.0   6
*      temp   interval
10061401   300.0    7
* Left Boundary Condition Data
*      bound  incr  type  code   factor  node
10061501 60120000  0   101   1    0.05   1
10061502 60010000  0   101   1    0.05   2
* Right Boundary Condition Data
*      bound  incr  type  code   factor  node
10061601 55120000  0   101   1    0.05   1
10061602 55010000  0   101   1    0.05   2
*      source  mult  dmhl   dmhr   num
10061701     0   0.0   0.0   0.0   0.0   2
* Left Additional Boundary Condition Data
10061800     0
*      hthd  hlf  hlr  gslf  gslr  glcf  glcr  lbf  node
10061801 0.0 10.0 10.0  0.0  0.0  0.0  0.0  0.1  2
* Right Additional Boundary Condition Data
10061900     0
*      hthd  hlf  hlr  gslf  gslr  glcf  glcr  lbf  node
10061901 0.0 10.0 10.0  0.0  0.0  0.0  0.0  0.1  2
*n: 0
*      nh  np   geom   ssif   leftcoord reflood
10080000 12   17   2    1    0.0105  0
*      mesh   format
10080100     0   1
*      intervals   radius
10080101     16   0.0125
*      material   interval
10080201     3   16
*      rpkf   interval
10080301    0.0   16
*      temp   interval
10080401   300.0    17
* Left Boundary Condition Data
*      bound  incr  type  code   factor  node
10080501 80020000  0   101   1    0.1    1
10080502 80030000  0   101   1    0.1    2
10080503 80040000  0   101   1    0.1    3
10080504 80050000  0   101   1    0.1    4
10080505 80060000  0   101   1    0.1    5
10080506 80070000  0   101   1    0.1    6
10080507 80080000  0   101   1    0.1    7
10080508 80090000  0   101   1    0.1    8
10080509 80100000  0   101   1    0.1    9
10080510 80110000  0   101   1    0.1   10
10080511 80010000  0   101   1    0.05   11
10080512 80120000  0   101   1    0.05   12
* Right Boundary Condition Data
*      bound  incr  type  code   factor  node
10080601 60020000  0   101   1    0.1    1
10080602 60030000  0   101   1    0.1    2
10080603 60040000  0   101   1    0.1    3
10080604 60050000  0   101   1    0.1    4
10080605 60060000  0   101   1    0.1    5
10080606 60070000  0   101   1    0.1    6
10080607 60080000  0   101   1    0.1    7
10080608 60090000  0   101   1    0.1    8
10080609 60100000  0   101   1    0.1    9
10080610 60110000  0   101   1    0.1   10
10080611 60010000  0   101   1    0.05   11
10080612 60120000  0   101   1    0.05   12
*      source  mult  dmhl   dmhr   num
10080701     0   0.0   0.0   0.0   0.0   12
* Left Additional Boundary Condition Data
10080800     0
*      hthd  hlf  hlr  gslf  gslr  glcf  glcr  lbf  node
10080801 9.563e-3 10.0 10.0  0.0  0.0  0.0  0.0  0.1  10
10080802   0.0 10.0 10.0  0.0  0.0  0.0  0.0  0.1  12
* Right Additional Boundary Condition Data
10080900     0
*      hthd  hlf  hlr  gslf  gslr  glcf  glcr  lbf  node
10080901 0.011926 10.0 10.0  0.0  0.0  0.0  0.0  0.1  10
10080902   0.0 10.0 10.0  0.0  0.0  0.0  0.0  0.1  12
***** Point Kinetics *****
***** type   feedback *****
30000000  point separabl
*      decay   power   react   dnf
30000001  gamma-ac   1.98e6   0.0  45.03311
*      control
30000011   2 * General Table 2 (PK + PAR)
30000012   3 * General Table 3 (PB)
30000013   4 * General Table 4 (Reactivity_Slow)
30000014   5 * General Table 5 (Reactivity_Fast)
.

```

BIBLIOGRAPHIC DATA SHEET

(See instructions on the reverse)

NUREG/IA-0422

2. TITLE AND SUBTITLE

Transient Analysis of the Research Reactor MARIA MC fuel elements using RELAP5
mod3.3

3. DATE REPORT PUBLISHED

MONTH	YEAR
March	2013

5. AUTHOR(S)

M. Dabrowski, P. Domitr, E. Staron

4. FIN OR GRANT NUMBER

6. TYPE OF REPORT

Technical

7. PERIOD COVERED (Inclusive Dates)

8. PERFORMING ORGANIZATION - NAME AND ADDRESS (If NRC, provide Division, Office or Region, U.S. Nuclear Regulatory Commission, and mailing address; if contractor, provide name and mailing address.)

Panstwowa Agencja Atomistyki

Krucza 36

00-522 Warszawa

Poland

9. SPONSORING ORGANIZATION - NAME AND ADDRESS (If NRC, type "Same as above"; if contractor, provide NRC Division, Office or Region, U.S. Nuclear Regulatory Commission, and mailing address.)

Division of Systems Analysis

Office of Nuclear Regulatory Research

U.S. Nuclear Regulatory Commission

Washington, DC 20555-0001

10. SUPPLEMENTARY NOTES

A. Calvo, NRC Project Manager

11. ABSTRACT (200 words or less)

The new fuel type in the Research Reactor 'Maria' is undergoing a test procedure. Thermal-hydraulic calculations using RELAP have been applied in order to cross-check the results obtained using a specialized thermal-hydraulic 'SN' code. Nodalization of the fuel element has been developed and calculations have been performed. LOFA and RIA cases have been analysed and compared showing good agreement between the two codes.

12. KEY WORDS/DESCRIPTORS (List words or phrases that will assist researchers in locating the report.)

Polish National Atomic Energy Agency (PAA)

Research Reactor 'Maria'

RELAP5

GTRI (Global Threat Reduction Initiative)

National Centre for Nuclear Research

Reactivity Initiated Accident (RIA)

Loss of Flow Accident (LOFA)

Loss Of Coolant Accident (LOCA)

Highly Enriched Uranium (HEU)

Low Enriched Uranium fuel (LEU)

13. AVAILABILITY STATEMENT
unlimited

14. SECURITY CLASSIFICATION

(This Page)
unclassified

(This Report)
unclassified

15. NUMBER OF PAGES

16. PRICE



Federal Recycling Program



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, DC 20555-0001

OFFICIAL BUSINESS

NUREG/IA-0422

**Transient Analysis of the Research Reactor MARIA MC
Fuel Elements Using RELAP5 Mod 3.3**

March 2013