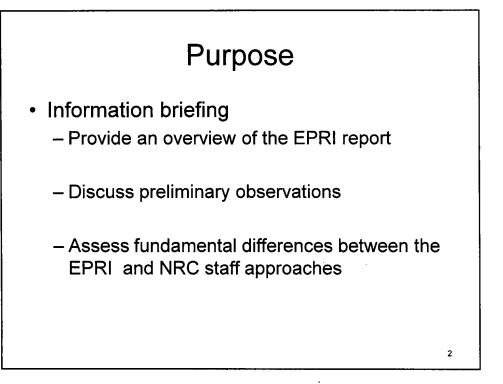
Initial Assessment of EPRI Technical Report "Investigation of Strategies for Mitigating Radiological Releases in Severe Accidents"

Japan Lessons Learned Steering Committee October 9, 2012



EPRI Study

• Purpose

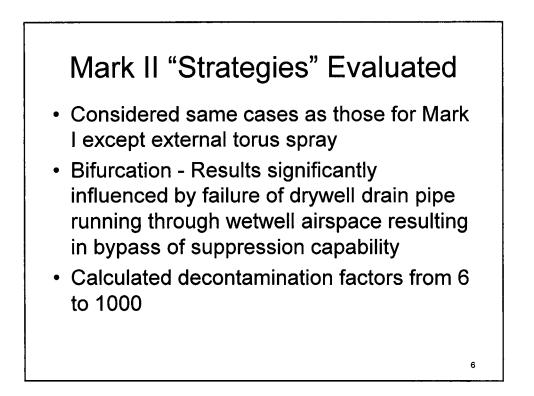
° .

- Investigate strategies to reduce fission product releases and land contamination
- Scope
 - Mark I and II containments
- Station blackout sequences
 4 hour RCIC
- MAAP
- Metric Cs release / integral decontamination factor

Mark I "Strategies" Evaluated

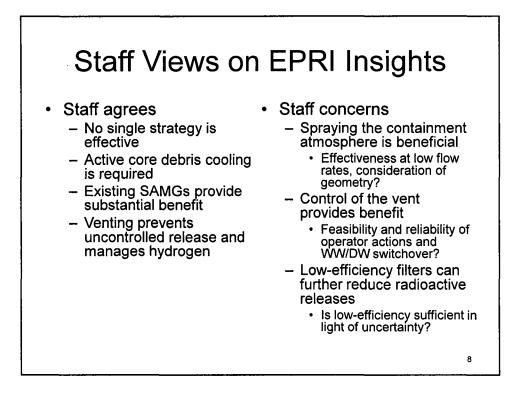
- Base case
 - No venting or core debris cooling
- Containment venting
 - Reliable hardened vent
 - External torus spray with reliable hardened vent
- · Core debris cooling
 - Containment flooding
 - Drywell sprays
- · Containment venting and core debris cooling
 - Containment flooding with reliable hardened vent
 - Drywell spray with reliable hardened vent
 - Containment flooding with controlled reliable hardened vent
 - Spray and controlled reliable hardened vent
- Calculated decontamination factors from 8 to 3594

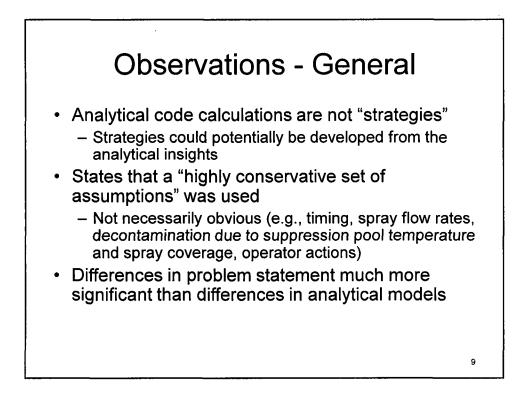
		ent Progressic	
In-Vessel Phenomenon (Minor variations in timing for sequences)	Time (hr)		
Reactor Trip	0.0		
RCIC Lost due to Loss of dc Power	4.0		
Core Uncovered	5 .2	Options	Time (hr)
Onset of Core Damage	6.1	Initiate Torus External Spray	5.0
SRV Seize Open 6.1		Initiate Drywell Flooding	6.0
Core Material to Lower Plenum 8.8		Secure DW Flood due to hi DW level	52.1-52.2
Reactor Vessel Breach	12.0	Initiate Drywell Sprays	5.0
		Secure DW Sprays due to hi DW level	49.7-58.3
	T	Cycle Wetwell Vent	11.9-17.9
Containment Failure Modes	Time (hr)	Close Wetwell Vent due to hi SP level	16.8-17.9
Liner Melt-Through	12.2-12.3	Cycle Drywell Vent	17.9-72.0
Wetwell Venting	11.9-12.1		
Drywell Venting	17.9-67.0		
Drywell Lezkage	12.0-63.7		
Drywell Overpressurization	12.5		

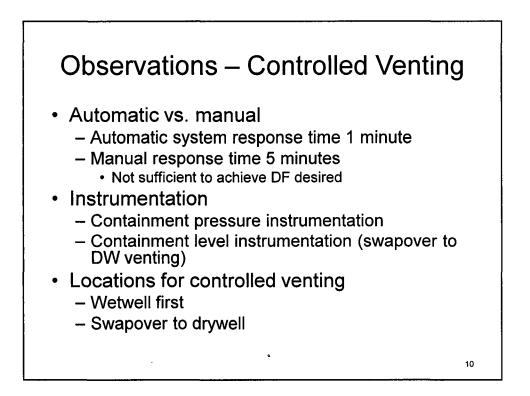


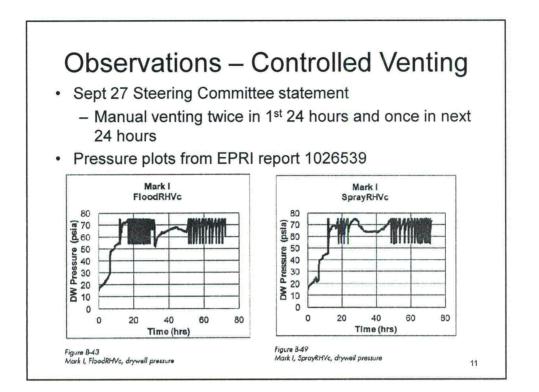
Sensitivity Analysis

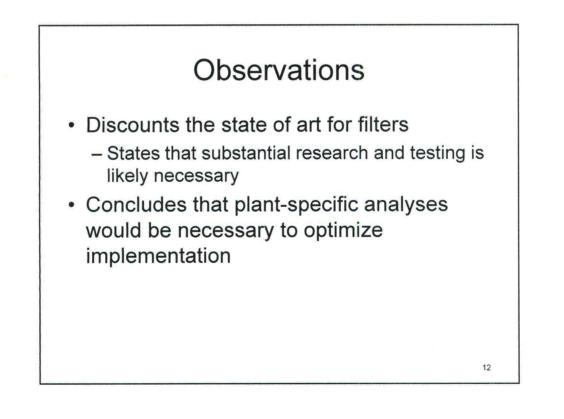
- Core damage timing
- Reactor vessel pressure
- Early containment venting for hydrogen control
- Early containment venting for pressure control
- · Spray water droplet diameter
- Spray aerosol removal efficiency
- · Spray flow rate
- · Core debris flow to suppression pool
- In-vessel recovery
- Reliable hardened vent sizing











Most Important Slide

"Two Acceptable Approaches -- Fundamentally Different Results"

Industry

NRC Staff

- Purpose Minimize land contamination due to potential releases
- Belief High confidence in Mark I & II containments, plant systems, and analytical codes
- Maximize use of existing systems
- More active management of containment and other systems
 following a severe accident
- Purpose Evaluate Mark I and II containment vulnerabilities to improve defense-in-depth
- Belief Greater level of uncertainty in Mark I and II containments, plant systems, and analytical codes
- Provide additional tools for response
 - Minimize need for operator actions through incorporation of some "passive" aspects

13

Result of Differences in Approaches Decontamination Factors

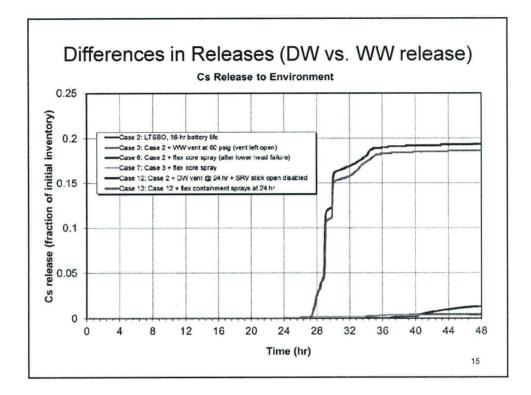
Industry

Evaluate integrated ability of •

- plant to limit radionuclide release
- Inverse of entire radionuclide release fraction to environment
 DF=(1/(1-.5-.32-.08-.07-
 - DF=(1/(1-.5-.32-.06 .0288))=893
- Accounts for release, transport, deposition, and hold up
 - Reactor core and vessel
 - Reactor coolant system
 - Drywell and wetwell
 - Suppression Pool
 - Reactor Building
- Consistent with risk-informed regulation

NRC Staff

- Evaluate design to improve containment defense-in-depth
- Inverse of radionuclide release
 fraction exiting filter
 - DF=(1/(1-.99888))=893
- Accounts for filtering mechanisms only
- Consistent with addressing weaknesses in design
- Consistent with approach in other countries (assumes severe accident with release will occur and need to mitigate it)



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