1	ES-401, REV 9	EV 9	Ļ	T1G1 PWR EXAMINATION OUTLINE	FORM ES-401-2
	KA	NAME / SAFETY FUNCTION:	R	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
-			RO SF	SRO	
0	007EK1.04	Reactor Trip - Stabilization - Recovery / 1	3.6 3.9	3.9 🖌 🗆 🗂 🗂 🗂 🗍	Decrease in reactor power following reactor trip (prompt drop and subsequent decay)
@	008AK2.02	Pressurizer Vapor Space Accident / 3	2.7 2.7	2.7 3 5 3 3 4 4 4 4 4 4 4 4 4 4 4 4 4 4 4 4	Sensors and detectors
\bigcirc	009EA1.12	Small Break LOCA / 3	4.2 4.2	4.2	RPS
(\mathcal{F})	011EK2.02	Large Break LOCA / 3	2.6 2.7	2.7 3 2 3 3 3 4 4 4 4 4 4 4 4 4 4 4 4 4 4 4	Pumps
\bigcirc	015AA1.02	RCP Malfunctions / 4	2.8 2.7	2.7 0 0 0 0 0 0 0 0 0 0 0	RCP oil reservoir level and alarm indicators
C	022AA2.04	Loss of Rx Coolant Makeup / 2	2.9 3.8	3.8	How long PZR level can be maintained within limits
	025AA2.03	Loss of RHR System / 4	3.6 3.6	3.8	Increasing reactor building sump level
\sim	026AA1.01	Loss of Component Cooling Water / 8	3.1 3.1	3.1	CCW temperature indications
\bigcirc	027AK2.03	Pressurizer Pressure Control System Malfunction / 3	2.6 2.8	2.8	Controllers and positioners
	038EA2.10	Steam Gen. Tube Rupture / 3	3.1 3.3	3.3 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0	Flowpath for charging and letdown flows
	054AK3.04	Loss of Main Feedwater / 4	4.4	4.6 Image: Constraint of the second	Actions contained in EOPs for loss of MFW

	ES-401, REV 9	EV 9	T10	T1G1 PWR EXAMINATION OUTLINE	FORM ES-401-2
	KA	NAME / SAFETY FUNCTION:	Я	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
			RO SRO		
(2)	055EG2.2.37	Station Blackout / 6	3.6 4.6		Ability to determine operability and/or availability of safety related equipment
(m)	056AK1.03	Loss of Off-site Power / 6	3.1 3.4		Definition of subcooling: use of steam tables to determine it
(+)	057AG2.4.6	Loss of Vital AC Inst. Bus / 6	3.7 4.7		Knowledge symptom based EOP mitigation strategies.
	058AK1.01	Loss of DC Power / 6	2.8 3.1		Battery charger equipment and instrumentation
(9)	062AK3.01	Loss of Nuclear Svc Water / 4	3.2 3.5		The conditions that will initiate the automatic opening and closing of the SWS isolation valves to the nuclear service water coolers
Ð	065AK3.04	Loss of Instrument Air / 8	3 3.2		Cross-over to backup air supplies
	BE04EG2.4.3	Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4	2.7 4.1		Knowledge of events related to system operations/status that must be reported to internal orginizations or outside agencies.

ES-401, REV 9	REV 9	T1	T1G2 PWR EXAMINATION OUTLINE	FORM ES-401-2
KA	NAME / SAFETY FUNCTION:	R	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO SRO	0	
024AA1.02	Emergency Boration / 1	3.7 3.5		Boric acid pump
059AA1.03	Accidental Liquid RadWaste Rel. / 9	3 2.9		Flow rate controller
060AK3.03	Accidental Gaseous Radwaste Rel. / 9	3.8 4.2		Actions contained in EOP for accidental gaseous-waste release
074EA2.06	Inad. Core Cooling / 4	4 4.6		Changes in PZR level due to PZR steam bubble transfer to the RCS during inadequate core cooling
BA01AK2.2		3.5 3.5		Facility s heat removal systems, including primary coolant, emergency coolant, the decay heat removal systems, and relations between the proper operation of these systems to the operation of the facility.
BA04AK3.2	2 Turbine Trip / 4	3.4 3.6		Normal, abnormal and emergency operating procedures associated with (Turbine Trip).
BA06AG2.4.3	1.3 Shutdown Outside Control Room / 8	3.7 3.9		Ability to identify post-accident instrumentation.
BE08EG2.2	.39	3.9 4.5		Knowledge of less than one hour technical specification action statements for systems.
E19E14EK1.3	EOP Enclosures	3.2 3.2		Annunciators and conditions indicating signals, and remedial actions associated with the (EOP Enclosures).

ES-4	ES-401, REV 9	T20	T2G1 PWR EXAMINATION OUTLINE	FORM ES-401-2
¥	NAME / SAFETY FUNCTION:	R	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO SRO		
003A3.05	.05 Reactor Coolant Pump	2.7 2.6		RCP lube oil and bearing lift pumps
004K1.02	02 Chemical and Volume Control	3.5 3.8		PZR and RCS temperature and pressure relationships
005A4.03	.03 Residual Heat Removal	2.8 2.7		RHR temperature, PZR heaters and flow and nitrogen
005K2.01	01 Residual Heat Removal	3.0 3.2		RHR pumps
006K4.05	05 Emergency Core Cooling	4.3 4.4		Autostart of HPI/LPI/SIP.
007A1.01	01 Pressurizer Relief/Quench Tank	2.9 3.1		Maintaining quench tank water level within limits
007A2.06 (공식)	06 Pressurizer Relief/Quench Tank	2.6 2.8		Bubble formation in PZR
008K4.01	01 Component Cooling Water	3.1 3.3		Automatic start of standby pump
010A1.08	08 Pressurizer Pressure Control	3.2 3.3		Spray nozzle DT
012K6.04	04 Reactor Protection	3.3 3.6		Bypass-block circuits
2013K5.02	D2 Engineered Safety Features Actuation	2.9 3.3		Safety system logic and reliability

	ES-401, REV 9	EV 9	Τ2	T2G1 PWR EXAMINATION OUTLINE	FORM ES-401-2
	KA	NAME / SAFETY FUNCTION:	≌	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G TO	TOPIC:
			RO SF	SRO	
39	022K2.02	Containment Cooling	2.5 2.4		Chillers
Ð	022K3.01	Containment Cooling	2.9 3.2		Containment equipment subject to damage by high or low temperature, humidity and pressure
(\mathcal{F})	026K1.02	Containment Spray	4.1 4.1		Cooling water
(Z)	039A2.03	Main and Reheat Steam	3.4 3.7		Indications and alarms for main steam and area radiation monitors (during SGTR)
Ð	039K5.05	Main and Reheat Stearn	2.7 3.1		Bases for RCS cooldown limits
(Z)	059A1.03	Main Feedwater	2.7 2.9		Power level restrictions for operation of MFVV pumps and valves.
(JS)	059A3.06	Main Feedwater	3.2 3.3		Feedwater isolation
Ð	061K2.03	Auxiliary/Emergency Feedwater	4.0 3.8		AFW diesel driven pump
(F)	062K3.02	AC Electrical Distribution	4.1 4.4		U
(fg)	063A4.02	DC Electrical Distribution	2.8 2.9		Battery voltage indicator
E.	(40)063K3.01	DC Electrical Distribution	3.7 4.1		U

Page 2 of 3

	ES-401, REV 9	EV 9	-	[2G1 F	T2G1 PWR EXAMINATION OUTLINE	FORM ES-401-2
	KA	NAME / SAFETY FUNCTION:	R		K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
			RO S	SRO		
() ()	064K6.08	Emergency Diesel Generator	3.2 3	3.3		Fuel oil storage tanks
B	073G2.1.25	Process Radiation Monitoring	3.9 4	4.2		Ability to interpret reference materials such as graphs, monographs and tables which contain performance data.
(\mathbf{G})	073G2.4.45	Process Radiation Monitoring	4.1 4	4.3		Ability to prioritize and interpret the significance of each annunciator or alarm.
(\mathfrak{B})	076G2.1.7	Service Water	44		4.4 4.7 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0	Ability to evaluate plant performance and make operational judgments based on operating characteristics, reactor behavior and instrument interpretation.
B	078A4.01	Instrument Air	3.1 3.1			Pressure gauges
(\mathcal{G})	55 103A2 03	Containment	3.5 3.	3.8		Phase A and B isolation

ES-401, REV 9	REV 9	T2(T2G2 PWR EXAMINATION OUTLINE	FORM ES-401-2
KA	NAME / SAFETY FUNCTION:	R	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO SRO		
002K4.07	Reactor Coolant	3.1 3.5		Contraction and expansion during heatup and cooldown
016K3.07	Non-nuclear Instrumentation	3.6 3.7		ECCS
028K2.01	Hydrogen Recombiner and Purge Control	2.5 2.8		Hydrogen recombiners
035A1.01	Steam Generator	3.6 3.8		S/G wide and narrow range level during startup, shutdown and normal operations
045A4.01	Main Turbine Generator	3.1 2.9		Turbine valve indicators (throttle, governor, control, stop, intercept), alarms and annunciators
055G2.1.20	Condenser Air Removal	4.6 4.6		Ability to execute procedure steps.
071K5.04	Waste Gas Disposal	2.5 3.1		Relationship of hydrogen/oxygen concentrations to flammability
072A3.01	Area Radiation Monitoring	2.9 3.1		Changes in ventilation alignment
(6 4) 079K1.01	Station Air	3.0 3.1		IAS
086A2.04	Fire Protection	3.3 3.9		Failure to actuate the FPS when required, resulting in fire damage

ES-401, REV 9	REV 9	F	T3 PWR EXAMINATION OUTLINE	FORM ES-401-2
KA	NAME / SAFETY FUNCTION:	R	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO SRO	0	
G2.1.13	Conduct of operations	2.5 3.2		Knowledge of facility requirements for controlling vital / controlled access.
G2.1.31	Conduct of operations	4.6 4.3		Ability to locate control room switches, controls and indications and to determine that they are correctly reflecting the desired plant lineup.
G2.1.38	Conduct of operations	3.7 3.8		Knowledge of the stations requirements for verbal communication when implamenting procedures
G22.15	Equipment Control	3.9 4.3		Ability to determine the expected plant configuration using design and configuration control documentaion
G2.223	Equipment Control	3.1 4.6		Ability to track Technical Specification limiting conditions for operations.
62.2.44	Equipment Control	4.2 4.4		Ability to interpret control room indications to verify the status and operation of a system, and understand how operator actions and directives affect plant and system conditions
G2.3.15	Radiation Control	2.9 3.1		Knowledge of radiation monitoring systems
(Z3) (Z3)	Radiation Control	3.5 3.6		Ability to comply with radiation work permit requirements during normal or abnormal conditions
(19) G2.4.19	Emergency Procedures/Plans	3.4 4.1		Knowledge of EOP layout, symbols and icons.
G2.4.40	Emergency Procedures/Plans	2.7 4.5		Knowledge of the SRO's responsibilities in emergency plan implementation.

Adherence to appropriate procedures and operation within the limitations in the facility s license and amendments.	3.6 4.4	BE04EA2.2 Inadequate Heat Transfer - Loss of Secondary Heat Sink / 4
Ability to use plant computer to evaluate system or component status.	3.9 3.8 0 0 0 0 0 0 0 0 0	6
Knowledge of local auxiliary operator tasks during emergency and the resultant operational effects	3.8 4.0 0 0 0 0 0 0 0 0 0	054AG2.4.35 Loss of Main Feedwater / 4
Ability to verify system alarm setpoints and operate controls identified in the alarm response manual.	4.2 4.0	
Cause of RCP failure	3 3.5 0 0 0 0 0 0 0 0 0	015AA2.01 RCP Malfunctions / 4
The effect of an open PORV on code safety, based on observation of plant parameters	3.4 3.6]]]]]]]]]]]]]	
	RO SRO	
TOPIC:	IR K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	KA NAME / SAFETY FUNCTION:
FORM ES-401-2	SRO T1G1 PWR EXAMINATION OUTLINE	ES-401, REV 9
SRO ONLY		

	ES-401, REV 9	6 <u>></u>	SRO T1G2 PWR EXAMINATION OUTLINE	NATION OUTLINE	SRO ONY FORM FS-401-2
	KA	NAME / SAFETY FUNCTION:	IR K1 K2 K3 K4 K5 R0 SR0	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
(28)	028AG2.4.47	028AG2.4.47 Pressurizer Level Malfunction / 2	4.2 4.2 0 0 0 0 0 0 0 0		Ability to diagnose and recognize trends in an accurate and timely manner utilizing the appropriate control room reference material.
(E)	037AA2.01	Steam Generator Tube Leak / 3	3 3.4		Unusual readings of the monitors; steps needed to verify readings
(BF)	BE03EG2.4.6	BE03EG2.4.6 Inadequate Subcooling Margin / 4	3.7 4.7]]]]]]]]]]		Knowledge symptom based EOP mitigation strategies.
(\mathcal{C})	BE13EA2.1	EOP Rules	3.4 4		Facility conditions and selection of appropriate procedures during abnormal and emergency

	ES-401. REV 9	6)	SRO T264 DWD EYAMINATION OUTLINE		SRA ONLY
					FURM ES-401-2
	KA	NAME / SAFETY FUNCTION:	IR K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	1 A2 A3 A4 G	TOPIC:
			RO SRO		
(006G2.2.42	Emergency Core Cooling	3.9 4.6 0 0 0 0 0 0 0 0 0		Ability to recognize system parameters that are entry-
(99)					level conditions for Technical Specifications
(007A2.03	Pressurizer Relief/Quench Tank	3.6 3.9		Overpressurization of the PZR
(18)					
	061A2.08	Auxiliary/Emergency Feedwater	3.4 3.8		pump failure or improper operation
Ø (
63)	Ø 063G2.1.30	DC Electrical Distribution	4.4 4.0		Ability to locate and operate components, including local controls
66	064G2.2.22	Emergency Diesel Generator	4.0 4.7 0 0 0 0 0 0 0 0 0		Knowledge of limiting conditions for operations and safety
)					mility.

ES-401, REV 9	EV 9	SRO.	SRO T2G2 PWR EXAMINATION OUTLINE	SRO ONY FORM ES401-2
KA	NAME / SAFETY FUNCTION:	RO SRO	K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
016A2.01	Non-nuclear Instrumentation	3.0 3.1	3.0 3.1 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0	Detector failure
017G2.1.30	017G2.1.30 In-core Temperature Monitor	4.4 4.0	4.4 4.0	Ability to locate and operate components, including local controls.
93) ^{068G2.4.45}	068G2.4.45 Liquid Radwaste	4.1 4.3	4.1 4.3	Ability to prioritize and interpret the significance of each annunciator or alarm.

ES-401, REV 9	REV 9	SRO T3 PWR EXAMINATION OUTLINE	FORM ES-401-2
₹¥	NAME / SAFETY FUNCTION:	IR K1 K2 K3 K4 K5 K6 A1 A2 A3 A4 G	TOPIC:
		RO SRO	
G2.1.20	Conduct of operations	4.6 4.6 0 0 0 0 0 0 0 0 0	Ability to execute procedure steps.
G2.2.3	Equipment Control	3.8 3.9 0 0 0 0 0 0 0 0 0	(multi-unit license) Knowledge of the design, procedural and operational differences between units.
G2.2.35	Equipment Control	3.6 4.5 3 3.6 4.5 3 4.5 3 4.5 4.5 4.5 4.5 4.5 4.5 4.5 4.5 4.5 4.5	Ability to determine Technical Specification Mode of Operation
G2.3.12	Radiation Control	3.2 3.7 0 0 0 0 0 0 0 0	Knowledge of radiological safety principles pertaining to licensed operator duties
G2.3.7	Radiation Control	3.5 3.6 0 0 0 0 0 0 0	Ability to comply with radiation work permit requirements during normal or abnormal conditions
G2.4.13	Emergency Procedures/Plans	4.0 4.6	Knowledge of crew roles and responsibilities during EOP usage.
G2.4.16	Emergency Procedures/Plans	3.5 4.4	Knowledge of EOP implementation hierarchy and coordination with other support procedures or guidelines.

Tier / Group	Randomly Selected K/A	Reason for Rejection
2 / 1 Q 51	G2.1.25	This KA requires the interpretation of, for example, a graph or table. Because a document of this nature could not be found, a suitable question could not be constructed. Another KA was randomly selected from section G2.1 of the KA manual. (SMG)
2/1 Q 48	063 A4.02	Could not formulate a question that did not border on minutia due to the subject of the K/A – Battery Voltage Indicator. Randomly selected another K/A from the A4 section. (CDS)
SRO 1/1 Q 80	058 Gen. 2.1.19	Unable to write an acceptably-discriminating question at the SRO level for the K/A. Randomly selected another generic K/A from the list in NUREG 1021 ES-401.D.1.b (058 2.2.40) (TJF)
SRO T3 Q95	2.2.36	This is not a Multi Unit <u>license</u> facility. (GWA)
RO 1/1 Q18	BE04 G2.4.20	Not able to write RO level question. (GWA)
RO T3 Q74	2.4.19	Only EOP symbol would be immediate action steps. No discriminating question. (GWA)
RO 2/1 Q39	022K2.01	ANO-1 does not have containment chillers. Changed to fan cooling units. (GWA)
RO 2/1 Q46	061K2.01	ANO-1 does not have <u>diesel</u> AFW pump. (GWA)
SRO 2/1 Q89	063 Gen 2.1.30	Unable to write question at the SRO level for this KA which addresses ability to operate local controls. Replaced with Generic 2.2.42 which concerns Technical Specifications and is much more appropriate for SRO. (JWC-ANO)
RO 1/1 Q5	015 AA1.02	Unable to write question at the RO level for this KA since the question would be about nuclear trivia or a reference would need to be provided causing the question to be a direct lookup. Replaced with 015 AA1.05. (JWC-ANO)
RO 1/1 Q9	027 AK2.02	This exam bank question was originally linked to AK2.03, AK2.02 is not appropriate for this question therefore this KA was replaced. (JWC-ANO)
RO 2/2 Q 64	079 K1.01	Unit 1 no longer has a Service Air compressor. Unit 1 Service Air is supplied from Unit 2, Unit 1 simply has a Service Air header. Replaced with Instrument Air KA 078 K1.05. (JWC-ANO)
RO 1/1 Q 3	009EA1.13	Changed from RPS to ESFAS because of B&W Design.

RO 1/2	BA06A.G.2.4.3	Tasks in-plant more appropriate post accident for RO.
Q 25		

ES-301

Administrative Topics Outline Revision 0

Form ES-301-1

Facility: <u>ANO-1</u>		Date of Examination: 2-25-2013			
Examination Level: RO 🔀	SRO	Operating Test Number: 2013-1			
Administrative Topic (see Note)	Type Code*	Describe activity to be performed			
Conduct of Operations		A1JPM-RO-TTB: Perform Time to Boil and Time to Core Uncovery calculations.			
A1. 2.1.23 (Imp 4.3)	N/R	Ability to perform specific system and integrated plant procedures during all modes of plant operation.			
Conduct of Operations A2. 2.1.20	N/R	A1JPM-RO-SFPMU: Perform SF Pool makeup calculation.			
(Imp 4.6)		Ability to determine and interpret procedure steps.			
Equipment Control A3. 2.2.13	D/P/R	A1JPM-RO-HCRD4: Perform independent review of a prepared tag out.			
(Imp 4.1)		Knowledge of tagging and clearance procedures.			
Radiation Control A4. 2.3.7	M/R	A1JPM-NRC-ADMINRWP1: Review RWP and determine maximum stay time.			
(Imp 3.5)		Ability to comply with radiation work permit requirements during normal or abnormal conditions.			
Emergency					
Procedures/Plan	N/A	N/A			
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.					
 * Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 (1) for ROs; ≤ 4 for SROs & RO retakes) (N)ew or (M)odified from bank (≥ 1) (3) (P)revious 2 exams (≤ 1; randomly selected) (1) 					

ES-301

Administrative Topics Outline Revision 0

Form ES-301-1

Facility: <u>ANO-1</u> Examination Level: RO	SRO 🛛	Date of Examination: <u>2-25-2013</u> Operating Test Number: <u>2013-1</u>				
Administrative Topic (Note)	see Type Code*	Describe activity to be performed				
Conduct of Operation A5. 2.1.23 (Imp 4.4)	ons N/R	A1JPM-SRO-TTB: Review Time to Boil and Time to Core Uncovery calculations. Ability to perform specific system and integrated plant procedures during all modes of plant operation.				
Conduct of Operation A6. 2.1.20 (Imp 4.6)	ons N/R	A1JPM-SRO-FIRE3: Respond to a fire system annunciator using procedures and TRM. Ability to interpret and execute procedure steps.				
Equipment Contro A7. 2.2.13 (Imp 4.3)	I D/P/R	A1JPM-SRO-HCRD4: Authorize a tag out. Knowledge of tagging and clearance procedures.				
Radiation Control A8. 2.3.7 (Imp 3.6)	M/R	A1JPM-NRC-ADMINRWP1: Review RWP and determine maximum stay time. (same as RO) Ability to comply with radiation work permit requirements during normal or abnormal conditions.				
Emergency Procedures/Plan A9. 2.4.41 (Imp 4.6)	N/R	A1JPM-SRO-EAL10: Evaluate conditions and declare an EAL. Knowledge of the emergency action level thresholds and classifications.				
NOTE: All items (5 total) are required for SROs. RO applicants require only 4 items unless they are retaking only the administrative topics, when all 5 are required.						
* Type Codes & Criteria:	 * Type Codes & Criteria: (C)ontrol room, (S)imulator, or Class(R)oom (D)irect from bank (≤ 3 for ROs; ≤ 4 (1) for SROs & RO retakes) (N)ew or (M)odified from bank (≥ 1) (4) (P)revious 2 exams (≤ 1; randomly selected) (1) 					

ES-	-301 Control Room/In-Plant Systems Or	utline	Form ES-301-2								
	Facility: Arkansas Nuclear One – Unit 1 Date of Examination: 2-25-2013 Exam Level: RO SRO-I SRO-U SRO-U Operating Test No.: 2013-1										
Cont	Control Room Systems [@] (8 for RO); (7 for SRO-I); (2 or 3 for SRO-U, including 1 ESF)										
	System / JPM Title	Type Code*	Safety Function								
a.	A1JPM-RO-EOP26 Perform Reactor Trip Immediate Actions 024 AK3.01 (RO 4.1/SRO 4.4) RO/SRO-U/SRO-I	A/D/EN/P/S	1 Reactivity Control								
b.	A1JPM-RO-LTOP2 Establish LTOP Protection during Cooldown of the RCS 006 A4.11 (RO 4.2/SRO 4.3) RO/SRO-U/SRO-I	D/L/S	3 Reactor Pressure Control								
C.	A1JPM-RO-MUP06 Perform RCS Delithiation 004 A1.11 (RO 3.0/SRO 3.0) RO/SRO-I	A/D/S	2 Reactor Coolan System Inventory Control								
d.	A1JPM-RO-AOP26 Respond to loss of Load Center B6 062 A2.01 (RO 3.4/SRO 3.9) RO	D/EN/S	6 Electrical								
e.	A1JPM-RO-HYD05 Place Hydrogen Recombiner M55A in Operation 028 A4.01 (RO 4.0/SRO 4.0) RO/SRO-I	A/N/L/S	5 Containment Integrity								
	A1JPM-RO-RPS01 Perform placing a RPS channel in Manual bypass 012 A4.03 (RO 3.6/SRO 3.6) RO/SROU/SRO-I	D/S	7 Instrumentatior								

n			-	
g.	A1JPM-RO-EOP21 Respond to React both SG levels remain < 410") 035 K4.01 (RO 3.6/SRO 3.8) RO/SRO-I	or Trip (Check	A/D/EN/L/S	4 Heat Removal From Reactor Core (Primary)
h.	A1JPM-RO-ICW01 Contingency Action Two ICW pumps 008 A2.01 (RO 3.3/SRO 3.6) RO/SRO-I	ns for Loss of	D/S	8 Plant Service Systems
In-Pla	ant Systems [@] (3 for RO); (3 for SRO-I); (3 or $\frac{1}{2}$	2 for SRO-U)	I	Lau anno 1997 - 1997 - 1997 - 1997 - 1997 - 1997 - 1997 - 1997 - 1997 - 1997 - 1997 - 1997 - 1997 - 1997 - 1997
i.	A1JPM-RO-EFW05 Actions for CST O to Long Term Operation of EFW 061 A1.04 (RO 3.9/SRO 3.9) RO/SRO-U/SRO-I	overheating due	L/N/E/R	4 Heat Removal From Reactor Core (Secondary)
j.	A1JPM-RO-EDG13 DG1 Engine start a 064 A1.01 (RO 3.0/SRO 3.1) RO/SRO-U/SRO-I	at slow idle	A/D	6 Electrical
k.	A1JPM-RO-GRW01 Commence Waste 071 A4.26(RO 3.1/SRO 3.9) RO/SRO-I	e Gas Release	D/P/R	9 Reactivity Release
@	All RO and SRO-I control room (and in-plant) s functions; all 5 SRO-U systems must serve dif overlap those tested in the control room.	systems must be differ ferent safety functions	ent and serve diffe ; in-plant systems	erent safety and functions may
	* Type Codes	Criteria fo	or RO / SRO-I / SR	0-U
(C)ontr (D)irec (E)mer (EN)gir (L)ow-F (N)ew (nate path rol room et from bank rgency or abnormal in-plant neered safety feature Power / Shutdown or (M)odified from bank including 1(A) ous 2 exams	≤ 9 ≥ 1 (3 ≥ 1 ≥ 2 ≤ 3	(4) / ≥ 1(4) / ≥ 1 (2 2 (2) / ≥ 2(2) / ≥ 1 (2	4) 1) control room system) 2) 1) 1) (randomly selected)

ES-301

Transient and Event Checklist

Form ES-301-5

Facility:	Facility: ANO Date of Exam: 2/25/2013 Operating Test No.: 2013-1													
A	E					Scenarios								
P P			1			2			3		Т		M	
		CRE	W POS	ITION	CRE	W POS	TION	CRE	W POS	ITION			l N	
I C A N	T T Y	S R O	A T C	B O P	S R O	A T C	B O P	S R O	A T C	B O P	A	-	I M U M(*)	1
T	P E											R		U
	RX		ļ			3					1	1	1	0
RO	NOR			1							1	1	1	1
R1, R3 R6, R8	1/C			2, 6		5, 6, 7					5	4	4	2
	MAJ			5, 8		6, 8					4	2	2	1
	TS	<u> </u>						<u> </u>			0	0	2	2
	RX		3						<u> </u>		1	1	1	0
RO	NOR				ļ					1	1	1	1	1
R2, R5	1/C		4, 7		ļ					4, 5, 9	5	4	4	2
	MAJ		5, 8							6, 7	4	2	2	1
	TS	<u> </u>									0	0	2	2
	RX	<u> </u>							3		1	1	1	0
RO	NOR						1, 2	<u> </u>	8		3	1	1	1
R4, R7	1/C						4, 9		4, 6, 9		5	4	4	2
	MAJ TS						6, 8		6, 7		4	2	2	1
							······································				0	0	2	2
	RX		3				***				1	1	1	0
SRO-I	NOR I/C		4, 7		1, 2, 3 4, 5, 6,						3	4	1	1
S1, S2	MAJ				7, 9							·		
	TS		5, 8		6, 8						4	2	2	1
	RX				5, 7, 9						3	0	2	2
	NOR	3					1, 2				0 3	1	1	0
SRO-U U1, U4	I/C	1, 2, 4, 6, 7					4, 9			*** *****	7	4	4	2
	MAJ	5,8					6, 8				4	2	2	1
	TS	1, 2, 7, 8									4	0	2	2
	RX										0	1	1	0
	NOR	3			1, 2, 3			1, 3			6	1	1	1
SRO-U U2, U3	1/C	1, 2, 4, 6, 7			4, 5, 6, 7, 9			4, 6, 9			13	4	4	2
	MAJ	5, 8			6, 8			6, 7			6	2	2	1
	TS	1, 2, 7, 8		ľ	5, 7, 9						7	0	2	2

Instructions:

- 1. Check the applicant level and enter the operating test number and Form ES-D-1 event numbers for each event type; TS are not applicable for RO applicants. ROs must serve in both the "at-the-controls (ATC)" and "balance-of-plant (BOP)" positions; Instant SROs must serve in both the SRO and the ATC positions, including at least two instrument or component (I/C) malfunctions and one major transient, in the ATC position. If an Instant SRO additionally serves in the BOP position, one I/C malfunction can be credited toward the two I/C malfunctions required for the ATC position.
- Reactivity manipulations may be conducted under normal or *controlled* abnormal conditions (refer to Section D.5.d) but must be significant per Section C.2.a of Appendix D. (*) Reactivity and normal evolutions may be replaced with additional instrument or component malfunctions on a 1-for-1 basis.
- 3. Whenever practical, both instrument and component malfunctions should be included; only those that require verifiable actions that provide insight to the applicant's competence count toward the minimum requirements specified for the applicant's license level in the right-hand columns.

Scenario Outline

Facility:	<u>ANO-1</u>	Sce	enario No.: <u>1</u>	Op-Test No.: 2013-1
Examine	ers:		Operators:	
- 100% - EFIC - C-28 - CV-3 - #2 E	onditions: % Power. C failed and will r B IA Compresso 3806 SW to #1 E DG will not Auto vide picture of RS	or is out of se DG is failed -Start.	ervice for overhaul.	
- C-28 - Curr Natu	shift – normal we BBIA Compresso ently in a Severe ral Emergencies d to Drain RB Su	or is out of see Thundersto have been ump to 40%.	completed.	bur. All required actions of 1203.025, g of the month and sampling is not
Event No.	Malf. No.	Event Type*		Event Description
1	CV-4400	N-(BOP) C-(SRO) TS	with failure of CV-4400 to	eactor Building Sump per 1104.014 o close. .3, Reactor Building Isolation Valves)
2	Lightning strike DI-DG1S K01A1	N/A	Lightning strike #1 EDG auto-start #1 EDG Auto Start alarm	
	CV-3806	C-(BOP) C-(SRO) TS	#1 EDG SW valve fails to #1 EDG shutdown	open.
3	N/A	R-(ATC) N-(SRO)	Dispatcher directs a powe minutes,	er reduction to 700MW in the next 15
4	Lightning strike	I-(ATC)	Lighting strike	
	ED451	I-(SRO)	Loss of NNI-Y power sup	ply
5	ED451 ED183	I-(SRO) M-(ALL)	Loss of NNI-Y power sup Loss of Offsite Power/De	
5				graded Power t
	ED183 DG176	M-(ALL) C-(BOP) C-(SRO) CT C-(ATC) C-(ATC) C-(SRO)	Loss of Offsite Power/De	graded Power t
6	ED183 DG176 DI_DG2_VR-LW	M-(ALL) C-(BOP) C-(SRO) CT C-(ATC)	Loss of Offsite Power/De #2 EDG will not auto start #2 EDG voltage low (<41	graded Power t

Scenario Outline

Form ES-D-1

Facility:	ANO-1	Sc	enario No.: <u>2</u>	Op-Test No.: <u>2013-1</u>			
Examine	ers:	****	Operators:				
 Initial Conditions: 12 EFPD BOL Reactor Power ~2% Ready to place "A" MFW pump in service per step 17.16 of 1102.002 Plant Startup procedure. Turnover: Reactor Power is ~2% Ready to place the "A" MFW pump is service and secure P-75 AFW pump. Commence raising power when "A" MFW pump is in service. 							
Event No.	Malf. No.	Event Type*		Event Description			
1	N/A	N-(BOP) N-(SRO)	Place "A" MFW Pump in s pump.	ervice and secure P-75 Aux FW			
2	N/A	N-(BOP) N-(SRO) CT	Reset anticipatory trip (AF	RTS) for the "A" MFW pump.			
3	N/A	R-(ATC) N-(SRO)	Commence power escala	tion to 7% power.			
4	CV6604	C-(BOP) C-(SRO)	Gland Seal regulator fails	closed.			
5	TR049	I-(ATC) I-(SRO) TS	Controlling Pressurizer lev	vel instrument fails low			
6	ED452	C-(ATC) C-(SRO) M-(ALL)	Loss of ICS power requirin Actuation.	ng Manual Reactor Trip EFW			
7	CV2648 CV2626	C-(ATC) C-(SRO) TS	Failed EFW control valves	s cause overcooling			
8	RC005	M-(ALL) CT	LOCA resulting in ESAS a (Trip all RCPs within 2 mir				
9	ES660	C-(BOP) C-(SRO) TS	ES Standby HPI pump fai	ls to auto-start			
* (1	N)ormal, (R)ead	ctivity, (I)nstr	ument, (C)omponent, (M)a	jor			

Scenario Outline

Facility: <u>ANO-1</u>	Scenario No.: <u>3</u>	Op-Test No.: 2013-1
Examiners:	Operators:	
Initial Conditions:		
- 100% Power.		
- EFIC failed and will r	ot auto-actuate.	
- P-4A Service Water I	Pump out of service for motor replaceme	ent.
- RPS is failed.		

- Rx Trip pushbutton on C03 is failed

Turnover:

- P-4A Service Water Pump out of service for motor replacement.
- Add N2 to "A" CFT to ~585# per 1104.001 per step 11.0.

Event Event Event Malf. No. Type* Description No. N-(BOP) Add N2 to "A" CFT to raise pressure to ~585# N/A 1 N-(SRO) FW086 N/A P-8A heater drain pump winding failure and trip 2&3 R-(ATC) N/A **Power Reduction** N-(SRO) I-(ATC) TR580 I-(BOP) Controlling Header Pressure fails low 4 I-(SRO) CO_P14B Operating EH pump trips CO^{P14A} 5 C-(BOP) Turbine trip >43% DI_ICC0020 C-(ATC) RP246,7,8,9 RPS is failed C-(SRO) TS 6 C-(ATC) DI ICC0020 C03 manual trip pushbutton failed TS M-(ALL) Manual Reactor Trip СТ M-(ALL) RC002 ~150 GPM tube leak in the "B" SG TS СТ 7 & 8 N-(ATC) N/A Plant cooldown and depressuization 9 CV061 C-(ALL) **Operating HPI Pump trips** * (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

Scenario Outline

Form ES-D-1

Facility:	ANO-1	Sc	enario No.: <u>4 R2</u>	Op-Test No.: 2013-1
Examine	ers:		Operators:	
- 100 - #2 E Turnove - Nati Cou /Sev - #2 E	DG Tagged out er: onal Weather sen nties until 8:00 p rere Thunderstor	for repairs. rvice has iss m today. AC m Warning,	P 1203.025, Natural Emerge all required actions have been	Warning for Pope and Conway ncies, Section 3, Tornado Watch
Event No.	Malf. No.	Event Type*	c.	Event Description
1	N/A	N (ALL)	Perform 1104.036 Supplem	ent 11 for EDG #1.
2	TR568	I (ATC)	"A" OTSG operating level tr	ansmitter fails high.
3	DI_RX7460S DI_RX7460SP	I (SRO) TS	RB atmosphere radioactivity determination.	y monitor fails low. TS 3.4.15
4	N/A	R (ATC) C (SRO) TS	#1 EDG inoperability (comn comply with Technical Spec	non mode) requires plant shutdown to sifications.
5	RX150	I (BOP) CT	Turbine EHC fails to respon control required).	d in auto mode for ICS (manual
6	CV2692	C (ATC) CT	One MSIV for "B" OTSG dri per EOP 1202.001.	fts shut-requires manual reactor trip
7	MS131	M (ALL)	"A" OTSG steam leak in cor to 1202.003 overcooling if n	ntainment (MSLI activates). Transfer lecessary.
8	ES264 CV2214	I (ATC) C (BOP) TS	actuate automatically and a that should shut. Operator v	ig but "B" train RBIC will fail to n ICW isolation valve will fail open vill be required to manually initiate "B" dundant ICW valve to ensure ICW
*	N)ormal, (R)eac	tivity (l)nstr	ument, (C)omponent, (M)ajo	