

Tier / Group	Randomly Selected K/A	Reason for Rejection
<b>RO EXAM</b>		
1/1	011 EK3.03	After a significant effort, unable to develop an operationally valid discriminatory exam question due to no unique and credible relationship between starting AFW pumps and flow, ED/G, and service water pumps and the large break LOCA. Resampled and obtained 011 EK3.14.
1/1	026 AK1	There are no statements for this category. Resampled and obtained 026 AK3.02.
1/1	038 EK1.03	After a significant effort, unable to develop an operationally valid discriminatory exam question due to no unique and credible relationship between the Steam Generator tube rupture and operational implications of natural circulation. Resampled and obtained 038 EK1.01.
1/1	057 AK1	There are no statements for this category. Resampled and obtained 057 AA2.04.
1/1	058 AK2	There are no statements for this category with an IR $\geq$ 2.5. Resampled and obtained 058 AK3.02.
1/2	068 AK1	There are no statements for this category with an IR $\geq$ 2.5. Resampled and obtained 068 AK2.01.
2/1	008 K5	There are no statements for this category with an IR $\geq$ 2.5. Resampled and obtained 008 A1.01.
2/1	008 K6	There are no statements for this category with an IR $\geq$ 2.5. Resampled and obtained 008 A2.03.
2/1	012 K6.07	After a significant effort, unable to develop an operationally valid discriminatory exam question due to Core Protection Calculators not being installed at Palisades. Resampled and obtained 012 K6.06.
2/1	012 A3.03	After a significant effort, unable to develop an operationally valid discriminatory exam question due to similarities with RO question #9. Resampled and obtained 012 A3.06.
2/1	039 K4.03	Importance rating < 2.5. Resampled and obtained 039 K4.06.
2/1	039 K4.06	After a significant effort, unable to develop an operationally valid discriminatory exam question due to this topic not being safety significant at Palisades - see NRC Safety Evaluation Report: Single Failure Issue for Main Steam Isolation Valves, dated February 28, 1986. Resampled and obtained 039 K4.05.
2/1	059 K5	There are no statements for this category with an IR $\geq$ 2.5. Resampled and obtained 059 A4.03.
2/1	076 G2.4.49	After a significant effort, unable to develop an operationally valid discriminatory exam question due to no unique relationship between service water system and low power/shutdown implications in accident. Resampled and obtained G2.2.44.
2/1	078 K1.02	After a significant effort, unable to develop an operationally valid discriminatory exam question due to similarities with RO question #17. Resampled and obtained 078 K1.04.
2/2	001 K5.27	Importance rating < 2.5. Resampled and obtained 001 K5.88.
2/2	002 K6.03	After a significant effort, unable to develop an operationally valid discriminatory exam question due to no credible cause-effect relationship between a malfunction of the reactor vessel level monitoring system and the RCS. Resampled and obtained 002K6.02.
2/2	017 A3.01	After a significant effort, unable to develop an operationally valid discriminatory exam question due to similarities with RO questions #10 and 25. Resampled and obtained 017 A3.02.
2/2	017 A3.02	After a significant effort, unable to develop an operationally valid discriminatory exam question due to the feature described in K/A not existing at Palisades. Resampled and obtained 017 A4.01.
2/2	029 A4.04	After a significant effort, unable to develop an operationally valid discriminatory exam question due to the feature described in K/A not existing at Palisades. Resampled and obtained 029 A4.01.
2/2	029 A4.01	After a significant effort, unable to develop an operationally valid discriminatory exam question due to the fact that Palisades does not have a flow rate indicator in the control room for purge flow. Resampled and obtained 029 K4.03.
2/2	071 K2	There are no statements for this category with an IR $\geq$ 2.5. Resampled and obtained 071 K3.05.

**SRO EXAM**

1/1	CE/E05 G2.2.25	After a significant effort, unable to develop an operationally valid discriminatory exam question at the SRO level due to no unique and credible relationship between steam line rupture event and knowledge of the bases in technical specifications for limiting conditions for operations and safety limits. Resampled and obtained G2.4.30.
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