

than the licensee, that person shall describe, in accordance with 10 CFR 2.714(a)(2), the nature of the person's interest and the manner in which the interest is affected by this Order. A request for hearing shall not stay the immediate effectiveness of this order.

If a hearing is requested by the licensee or other persons who have an interest affected by this Order, the Commission will issue an Order designating the time and place of any such hearing.

If a hearing is held concerning this Order, the issue to be considered at the hearing shall be whether, on the basis of the information set forth in Sections II and III of this Order, the licensee should comply with the conditions set forth in Section IV of this Order.

This request for information was approved by OMB under clearance number 3150-0065 which expires June 30, 1983. Comments on burden and duplication may be directed to the Office of Management and Budget, Reports Management, Room 3208, New Executive Office Building, Washington, D.C.

This Order is effective upon issuance.

Dated at Bethesda, Maryland this 10th day of July, 1981.

For the Nuclear Regulatory Commission.

Darrell G. Eisenhut,

Director, Division of Licensing, Office of Nuclear Reactor Regulation.

[FR Doc. 81-21273 Filed 7-20-81; 8:45 am]

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Regulatory Guides; Notice of Issuance and Availability and Withdrawals

The Nuclear Regulatory Commission has issued a revision to a guide in its Regulatory Guide Series. This series has been developed to describe and make available to the public methods acceptable to the NRC staff of implementing specific parts of the Commission's regulations and, in some cases, to delineate techniques used by the staff in evaluating specific problems or postulated accidents and to provide guidance to applicants concerning certain of the information needed by the staff in its review of applications for permits and licenses.

Regulatory Guide 1.136, Revision 2, "Materials, Construction, and Testing of Concrete Containments (Articles CC-1000-2000, and -4000 through -6000 of the 'Code for Concrete Reactor Vessels and Containments')," describes bases acceptable to the NRC staff for implementing portions of the Commission's regulations, with regard to the materials, construction, and testing of concrete containments.

This revision was developed to endorse, with some exceptions, Articles CC-1000, -2000, and -4000 through -6000 of Section III, Division 2, "Code for Concrete Reactor Vessels and Containments," of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code, also known as American Concrete Institute Standard 359-80, and to include any guidance necessary as a result of public comment and additional staff review.

With the issuance of this revision, the regulatory positions of six regulatory guides are considered to be covered by one or more of the following national standards:

—ACI 359 (ASME Section III, Division 2), "Code for Concrete Reactor Vessels and Containments," endorsed by Regulatory Guide 1.136.

—ACI 349, "Code Requirements for Nuclear Safety-Related Concrete Structures," endorsed by Regulatory Guide 1.142, "Safety-Related Concrete Structures for Nuclear Power Plants (Other than Reactor Vessels and Containments)."

—ANSI N45.2.5, "Supplementary Quality Assurance Requirements for Installation, Inspection, and Testing of Structural Concrete, Structural Steel, Soils, and Foundations During the Construction Phase of Nuclear Power Plants," endorsed by Regulatory Guide 1.94, "Quality Assurance Requirements for Installation, Inspection, and Testing of Structural Concrete and Structural Steel During the Construction Phase of Nuclear Power Plants."

Therefore, the six regulatory guides identified below have been withdrawn. Withdrawal of these guides does not alter any prior or existing licensing commitments based on their use.

1.10 "Mechanical (Cadmeld) Splices in Reinforcing Bars of Category I Concrete Structures," Revision 1, January 1973,

1.15 "Testing of Reinforcing Bars for Category I Concrete Structures," Revision 1, December 1972,

1.18 Structural Acceptance Test for Concrete Primary Reactor Containments," Revision 1, December 1972,

1.19 "Nondestructive Examination of Primary Containment Liner Welds," Revision 1, August 1972 (Safety Guide 19),

1.55 "Concrete Placement in Category I Structures," June 1973, and

1.103 "Post-tensioned Prestressing Systems for Concrete Reactor Vessels and Containments," Revision 1, October 1976.

Guides may be withdrawn when they are superseded by the Commission's

regulations, when equivalent recommendations have been incorporated in applicable approved codes and standards, or when changes in methods and techniques or in the need for specific guidance have made them obsolete.

Comments and suggestions in connection with (1) items for inclusion in guides currently being developed or (2) improvements in all published guides are encouraged at any time. Comments should be sent to the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. Attention: Docketing and Service Branch.

Regulatory guides are available for inspection at the Commission's Public Document Room, 1717 H Street NW., Washington, D.C. Copies of active guides may be purchased at the current Government Printing Office price. A subscription service for future guides in specific divisions is available through the Government Printing Office. Information on the subscription service and current prices may be obtained by writing to the U.S. Nuclear Regulatory Commission, Washington, D.C. 20555. Attention: Publication Sales Manager.

(5 U.S.C. 552(a))Q02

Dated at Rockville, Maryland this 14th day of July 1981.

For the Nuclear Regulatory Commission.

Robert B. Monogue,

Director, Office of Nuclear Regulatory Research.

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[Docket No. 50-327]

Tennessee Valley Authority; Issuance of Amendment to Facility Operating License No. DPR-77

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 8 to Facility Operating License No. DPR-77, issued to Tennessee Valley Authority (licensee) for the Sequoyah Nuclear Plant, Unit 1 (the facility) located in Hamilton County, Tennessee. This amendment updates technical specifications 3.7.5 and 4.7.5 to reflect design changes resulting from the implementation of the new Essential Raw Cooling Water (ERCW) pumping station. This amendment also increases the ultimate heat sink temperature from 81 degrees F to 83 degrees F.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the