



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

March 26, 2013

Mr. David A. Heacock
President and Chief Nuclear Officer
Dominion Nuclear
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: MILLSTONE POWER STATION UNIT NO. 2 - ISSUANCE OF AMENDMENT
RE: REVISE SURVEILLANCE REQUIREMENT 4.4.3.2, REACTOR COOLANT
SYSTEM RELIEF VALVES (TAC NO. MF0389)

Dear Mr. Heacock:

The Commission has issued the enclosed Amendment No. 314 to Renewed Facility Operating License No. DPR-65 for the Millstone Power Station, Unit No. 2, in response to your application dated December 17, 2012, as supplemented by letter dated January 31, 2013.

The amendment would revise Technical Specification (TS) Surveillance Requirement 4.4.3.2 to remove the requirement to perform the quarterly surveillance for a pressurizer power-operated relief valve block valve that is being maintained closed in accordance with TS 3.4.3 Action a. The proposed change is consistent with the requirements of the "Standard Technical Specification - Combustion Engineering Plants" (NUREG-1432, Revision 4).

A copy of the related Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

A handwritten signature in black ink that reads "James Kim".

James Kim, Project Manager
Plant Licensing Branch 1-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-336

Enclosures:

1. Amendment No. 314 to DPR-65
2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DOMINION NUCLEAR CONNECTICUT, INC.

DOCKET NO. 50-336

MILLSTONE POWER STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 314
Renewed License No. DPR-65

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the applicant dated December 17, 2012, as supplemented by letter dated January 31, 2013, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-65 is hereby amended to read as follows:

- (2) Technical Specifications

- The Technical Specifications contained in Appendix A, as revised through Amendment No. 314, are hereby incorporated in the renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance, and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Sean C. Meighan, Acting Chief
Plant Licensing Branch I-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment: Changes to the License
and Technical Specifications

Date of Issuance: March 26, 2013

ATTACHMENT TO LICENSE AMENDMENT NO. 314

RENEWED FACILITY OPERATING LICENSE NO. DPR-65

DOCKET NO. 50-336

Replace the following page of the Renewed Facility Operating License with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove
Page 3

Insert
Page 3

Replace the following pages of the Appendix A Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove
3/4 4-3a

Insert
3/4 4-3a

Connecticut, in accordance with the procedures and limitations set forth in this renewed operating license;

- (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter 1: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady-state reactor core power levels not in excess of 2700 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 314, are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

Renewed License No. DPR-65
Amendment No.314

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS

4.4.3.1 In addition to the requirements of Specification 4.0.5, each PORV shall be demonstrated OPERABLE:

- a. Once per 31 days by performance of a CHANNEL FUNCTIONAL TEST, excluding valve operation, and
- b. Once per 18 months by performance of a CHANNEL CALIBRATION.
- c. Once per 18 months by operating the PORV through one complete cycle of full travel at conditions representative of MODES 3 or 4.

4.4.3.2 Each block valve shall be demonstrated OPERABLE once per 92 days by operating the valve through one complete cycle of full travel. This demonstration is not required if a PORV block valve is closed in accordance with the ACTIONS of Specification 3.4.3. |



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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 314

TO RENEWED FACILITY OPERATING LICENSE NO. DPR-65

DOMINION NUCLEAR CONNECTICUT, INC.

MILLSTONE POWER STATION, UNIT NO. 2

DOCKET NO. 50-336

1.0 INTRODUCTION

By letter dated April 13, 2012, December 17, 2012 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML12362A012), as supplemented by letter dated January 31, 2013 (ML13045A482), Dominion Nuclear Connecticut, Inc. (the licensee) requested changes to the Technical Specifications (TSs) for Millstone Power Station Unit 2 (MPS2).

The proposed changes would revise TS 3.4.3 to expand the range of conditions under which quarterly testing of block valves for the pressurizer power operated relief valves (PORVs) would not be required.

The supplemental letter dated January 31, 2013, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the Nuclear Regulatory Commission (NRC) staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on January 22, 2013, (78 FR 4472).

2.0 REGULATORY EVALUATION

The regulation at Title 10, *Code of Federal Regulations* (10 CFR), Section 50.36(c)(3) states that, "Surveillance requirements (SRs) are requirements related to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation (LCOs) will be met." The MPS2 PORV block valves are the subject of LCOs and SRs in TS 3/4.4.3, "Reactor Coolant System Relief Valves." The NRC staff also considered relevant information in the MPS2, Unit 2, Updated Final Safety Analysis Report (UFSAR), TSs, and NUREG-1432, Revision 4, Standard Technical Specifications - Combustion Engineering Plants.

3.0 TECHNICAL EVALUATION

The PORVs are described in Section 4.3.7, "Valves" of the MPS2, UFSAR. The failure position of the PORV is closed. The pressurizer is equipped with two PORVs, each of which has a block

valve designed to be closed in the event that the PORV is determined to be inoperable. The pressurizer PORVs are not required to open in order to prevent the overpressurization of the Reactor Coolant System (RCS). The two pressurizer safety valves act to limit the system pressure to a maximum of 110 percent of design. Therefore, a loss of pressurizer PORV automatic control (or if the associated block valve is closed) and the subsequent failure of the PORVs to open will result in higher reactor coolant pressures, but will not cause any overpressurization problems.

Currently, SR 4.4.3.2 requires that the PORV block valves be subjected to an operability test, every 92 days, by operating the valve through one complete cycle of full travel unless the block valve has been closed and power removed to meet TS 3.4.3 actions b or c. The licensee's proposal to change SR 4.4.3.2 would expand the range of conditions under which quarterly testing of block valves for the PORVs would not be required. These conditions include any situation where a block valve is closed due to any action in TS 3.4.3, regardless if the power is removed from the block valve. This change aligns MPS2's surveillance requirement with the equivalent surveillance requirement of the standard technical specifications for combustion engineering plants, NUREG-1432.

The TS 3.4.3 and SR 4.4.3.2 are applicable in Modes 1, 2, and 3. The Final Safety Analysis Report states that the PORVs provide protection for low temperature overpressure (LTOP). The LTOP protection is required in Modes 4, 5, and 6 and since the change to the SR does not affect the PORV function in LTOP protection during Modes 4, 5, and 6, the change is acceptable with regard to LTOP.

As noted in the licensee's December 17, 2012, application, the PORV block valve has been closed due to PORV seat leakage, in accordance with TS 3.4.3 action a, since December 11, 2012. The motor-operated block valves are normally open. The block valves are closed when needed to protect against excessive PORV seat leakage and stuck open PORVs. The block valves are located between the pressurizer and the PORV. The removal of the surveillance does not affect the function of the PORVs and it prevents the PORV leakage from becoming worse.

The licensee stated that the leakage is believed to be seat leakage that exits the valve and pressurizer as steam through a tail pipe piece and is captured as steam and condensed in the quench tank. The quench tank is inside of containment and the leakage from the PORV is part of the reactor coolant system identified leakage. The quench tank pathway allows for the clean-up of gaseous and effluent leakage. The cause of the leakage will be determined when the valve is examined during the next refueling outage scheduled for spring 2014. The cause will be documented in the Millstone Power Station Corrective Action Program along with any further corrective actions and any applicability to the other PORV. With the block valve closed, the described leakage does not occur and the RCS boundary remains intact. Therefore, the current configuration is acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Connecticut State official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in amounts, and no significant change in the types of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (78 FR 4472).

Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. Woodyatt

Date: March 26, 2013

March 26, 2013

Mr. David A. Heacock
President and Chief Nuclear Officer
Dominion Nuclear
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

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Sincerely,

/ra/

James Kim, Project Manager
Plant Licensing Branch 1-1
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Accession No.: **ML13057A525**

*See memo dated February 25, 2013

Office	LPL1-1/PM	LPL1-1/LA	SRXB/BC	STSB/BC	OGC	LPL1-1/BC(A)
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Date	2/28/13	2/28/13	2/25/13	3/1/13	3/12/13	3/26/13

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