

Three Mile Island Unit 1 Telephone 717-948-8000 Route 441 South, P.O. Box 480 Middletown, PA 17057

February 22, 2013 TMI-13-026

10 CFR 50.73

U.S. Nuclear Regulatory Commission

Attn: Document Control Desk Washington, D.C. 20555-0001

THREE MILE ISLAND NUCLEAR STATION, UNIT 1 (TMI-1) RENEWED FACILITY OPERATING LICENSE NO. DPR-50 DOCKET NO. 50-289

SUBJECT: LICENSEE EVENT REPORT (LER) NO. 2012-002-01

"Missing Seals in Air Intake Tunnel Conduits"

This supplement updates Licensee Event Report (LER) 289/2012-002-00. The supplement revises the LER to provide the resulting safety significance based upon the assessment and evaluation of the reported condition. The corrective action to modify the design to provide the required flood protection for air intake tunnel conduits was completed. This supplement LER describes the safety significance of a past condition.

This report is submitted in accordance with 10 CFR 50.73(a)(2)(v)(B). For additional information regarding this LER contact Mike Fitzwater, Sr. Regulatory Engineer, TMI-1 Regulatory Assurance at (717) 948-8228.

There are no regulatory commitments contained in this LER.

Sincerely,

Mark Newcomer

Plant Manager, Three Mile Island, Unit 1

Exelon Generation Co., LLC

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MN/mdf

cc: TMI-1 Senior Resident Inspector

Administrator, Region I TMI-1 Project Manager

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internal seals for flood protection. When the plant was constructed in the early 1970's the conduit seals were never installed. The conduit seals are internal to the conduit fittings and not visible															
externally. The cause has been attributed to inadequate configuration management during original															
construction. The corrective action to modify the design to provide the required flood protection for AIT															
conduits was completed in October 2012. The safety significance of the past condition was evaluated.															
Had a probable maximum flood (PMF) event occurred at TMI-1, after a greater flood hazard was															
recognized in September 2011 and before the AIT conduit deficiency was identified and temporarily mitigated in August 2012, it is likely that some safe shutdown equipment would have been adversely															
affected but safe shutdown conditions would have been maintained and there would not have been any															
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The su	ubmitta	al of this	s LER	constitute	s rep	orting to	o the NF	RC in a	accorda	ance wi	th 10 CFR	50.73			

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NRC FORM 366A (10-2010) LICENSEE EVENT REPORT (LER)	U.S. NUCLEAR REGULATORY COMMISSION							
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A. EVENT DESCRIPTION

Plant Conditions before the event:

Babcock & Wilcox - Pressurized Water Reactor - 2568 MWth Core Power

Date/Time: August 10, 2012 / 12:30 hours Power Level: 100% steady state power

Mode: Power Operations

Event:

A flood protection walkdown (Post Fukushima response required by NRC 50.54(f) letter) in the air intake tunnel (AIT) was required to conduct visual inspection of 43 conduit penetrations. The sealing of all penetrations into the air intake tunnel is required to satisfy a license commitment that the plant can be maintained in a safe shutdown condition during an external flood with river flows up to the Probable Maximum Flood (PMF) (1,625,000 CFS). On 08/02/12, inspection confirmed that an internal seal fitting (Crouse Hinds model EYS) was installed at each penetration and that each conduit penetration through the wall was externally sealed. The inspection team also noted drain holes in the conduit fittings. Following that inspection, the team discussed a plan to inspect the internal seal. A second inspection was conducted on 08/10/12, and a borescope was inserted into the conduits where possible (i.e., cable configurations with respect to the holes limited the capability to view the internal configuration of all of the conduits). Six conduits were inspected and none contained the internal sealing compound material (Chico A).

Based on the apparent condition (i.e., none of the conduits had the internal seal installed in the EYS fitting), the NRC was notified of the existence of an "Event or Condition that could have prevented fulfillment of a Safety Function" as required by 10 CFR 50.72(b)(3)(v)(B) via Event Notification (EN) 48179 on 08/10/12. A subsequent Licensee Event Report (LER) 2012-002-00 was submitted on 10/09/12 with the planned intent to supplement the LER by 01/15/2013 after an evaluation on the safety significance of the event was completed. Docketed correspondence TMI-13-003 dated 01/15/13 updated the status of the evaluation and provided a revised planned date to supplement LER 2012-002-00 by 02/22/13.

B. CAUSE OF EVENT

When the plant was constructed in the early 1970's the conduit seals were never installed. The conduit seals are internal to the conduit fittings and not visible externally. The cause has been attributed to inadequate configuration management during original construction. The design drawings for the AIT conduit routing (215-162 & 215-163) show the Crouse Hinds EYS fittings, and the EYS fittings are installed. However, the internal sealing compound material, Chico A, was never installed in the EYS fittings.

C. ANALYSIS / SAFETY SIGNIFICANCE

Adequate flood protection was maintained for the emergency power systems, emergency feedwater, river water systems, Reactor Coolant Pump (RCP) seal cooling, and vital instruments. The operations and emergency response staff would have had adequate information concerning plant conditions at all times to maintain core cooling.

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The reactor would have been shutdown, and Reactor Coolant System (RCS) temperature and pressure reduced to place the decay heat removal system in service before the deficient flood protection barriers were challenged.

The impact of the leakage from the AIT conduits into the Auxiliary Building was evaluated assuming no mitigating credit was taken by the Emergency Response Organization. A technical evaluation took into account the aggregate impact of identified flood barrier deficiencies. The plant would have been shutdown on Decay Heat Removal (DHR) with the Severe Flood Mitigation System (SFMS) equipment staged and Once Through Steam Generator (OTSG) cooling available as provided in the flood abnormal operating procedure when the inleakage to the Auxiliary Building would have adversely impacted the DHR system. The evaluation concluded the Make-Up (MU) system would have subsequently been adversely impacted as well.

The plant would have been maintained stable on natural circulation using the emergency feedwater system with condensate inventory, then river water supply and a defense in depth supply using the SFMS system from the condenser pit. The RCS inventory would have been maintained. RCP seal cooling would be retained using the Intermediate Closed Cooling Water (ICCW) system. Inventory would be maintained to address nominal operating leakage by using the SFMS system which utilizes the spent fuel pool for its supply. The Core Flood (CF) system additionally would have provided a significant volume as a defense in depth inventory supply to the RCS.

As a result of the missing AIT conduit seals the DHR and MU systems would be lost due to flood water intrusion from the PMF. The emergency feedwater system, emergency diesel generators, reactor river system, SFMS, CF tanks, and RCP seal cooling are maintained. The RCS is maintained on natural circulation core cooling at reduced RCS pressure and temperature with the emergency organization staffed.

D. CORRECTIVE ACTIONS

- An industry notification was provided regarding an original construction deficiency where internal conduit seal fittings were installed but the seals were not installed.
- The flood protection design was modified for all affected AIT conduits and completed in October 2012.

E. PREVIOUS OCCURENCES

There are no previous occurrences that are applicable for this LER supplement.