



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

February 7, 2013

Mr. David A. Heacock
President and Chief Nuclear Officer
Virginia Electric and Power Company
Innsbrook Technical Center
5000 Dominion Boulevard
Glen Allen, VA 23060-6711

SUBJECT: SURRY POWER STATION, UNIT 1 - CORRECTION LETTER TO LICENSE AMENDMENT NO. 278 REGARDING ADOPTION OF TECHNICAL SPECIFICATION TASK FORCE (TSTF) 510, REVISION 2, STEAM GENERATOR PROGRAM INSPECTION FREQUENCIES AND TUBE SAMPLE SELECTION (TAC NO. ME9199)

Dear Mr. Heacock:

On January 28, 2013, the U.S. Nuclear Regulatory Commission issued Amendment No. 278 to Renewed Facility Operating License No. DPR-32 and Amendment No. 278 to Renewed Facility Operating License No. DPR-37 for Surry Power Station, Unit Nos. 1 and 2, respectively. The amendments implemented changes to the Technical Specifications (TSs) in response to your application dated July 31, 2012, as supplemented by letter dated November 6, 2012.

These amendments revised Limiting Condition for Operation (LCO) 3.1.H, "Steam Generator (SG) Tube Integrity," TS 6.4.Q, "Steam Generator (SG) Program," and TS 6.6.A.3, "Steam Generator Tube Inspection Report," and included TS Bases changes that summarized and clarified the purpose of the TS in accordance with TS Task Force Traveler (TSTF) 510, "Revision to Steam Generator Program Inspection Frequencies and Tube Sample Selection."

We recently recognized that the license page for Unit 1 did not include the amendment number in Item 3.B, Page 3. A corrected page is enclosed.

Please feel free to contact me at 301-415-1438 if you have any questions.

Sincerely,

Handwritten signature of Karen Cotton in black ink.

Karen Cotton, Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-280 and 50-281

Enclosure: As stated

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3. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations: 10 CFR Part 20, Section 30.34 of 10 CFR Part 30, Section 40.41 of 10 CFR Part 40, Sections 50.54 and 50.59 of 10 CFR Part 50, and Section 70.32 of 10 CFR Part 70; and is subject to all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified below:

A. Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 2587 megawatts (thermal).

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 278 are hereby incorporated in the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. Reports

The licensee shall make certain reports in accordance with the requirements of the Technical Specifications.

D. Records

The licensee shall keep facility operating records in accordance with the requirements of the Technical Specifications.

E. Deleted by Amendment 65

F. Deleted by Amendment 71

G. Deleted by Amendment 227

H. Deleted by Amendment 227

I. Fire Protection

The licensee shall implement and maintain in effect the provisions of the approved fire protection program as described in the Updated Final Safety Analysis Report and as approved in the SER dated September 19, 1979, (and Supplements dated May 29, 1980, October 9, 1980, December 18, 1980, February 13, 1981, December 4, 1981, April 27, 1982, November 18, 1982, January 17, 1984, February 25, 1988, and

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/RA/

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