

Facility: Point Beach	Scenario No.: <u> 1 </u>	Op-Test No.: <u> 2012301 </u>	
Examiners: _____	Operators: _____	_____	
_____	_____	_____	
_____	_____	_____	
<p>Initial Conditions: <u>Line 111, Point Beach to Sheboygan Energy Center, is OOS for tower repairs, 'B' MFRV controlling channel is shifted to YELLOW, 1W-3B CRDM shroud fan is OOS for bearing failure, Solar Magnetic Disturbance alert for GIC readings greater than 30 amps, IRPI in alternate for 1Y-06-21 breaker replacement, 1PI-2198 Condenser B Pressure Indicator is OOS for I&C calibration.</u></p> <p>_____</p> <p>_____</p>			
<p>Turnover <u>Unit 1 is stable at 75% power. Following turnover, per OI-72, Containment Air Recirculation System, start Unit 1 'A' Accident Recirc and Cooling Fans then secure 'B' Accident Recirc and Cooling fans in preparation for upcoming vibration data collection.</u></p> <p>_____</p> <p>_____</p>			
Event No.	Malf. No.	Event Type*	Event Description
1		N-BOP N-SRO	Shift Containment Accident Recirc and Cooling Fans
2		I-RO I-SRO TS-SRO	1LT-427 Pressurizer Level (White) fails low
3		C-RO C-SRO TS-SRO	Loop 'B' RTD Bypass Line leak (20 gpm)
4		R-RO N-BOP N-SRO	1W-3A CRDM Shroud Fan trips/Rapid Power Reduction
5		M-All	Small Break LOCA (500 gpm)
6		C-RO	1P-15A Safety Injection Pump trips 1P-15B Safety Injection Pump fails to auto start
7		C-BOP	Main Feed Isolation Valves fail to AUTO close
8		C-BOP	RCS Hot Leg Containment Isolation failures with 1 gpm leak
* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor			

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SITE: POINT BEACH

SEG # PBN LOI NRC

SEG TITLE: 2012 ILT NRC SCENARIO #1

REV. # 0

PROGRAM: INITIAL LICENSE TRAINING

#: PBN LOI TPD

COURSE: N/A

#: N/A

TOTAL TIME: 2.0 HOURS

Additional signatures may be added as desired.

Developed by:	Andrew Zommers _____ Instructor	7/27/12 _____
Reviewed by:	Joey Trudeau _____ Instructor (Simulator Scenario Development Checklist.)	_____
Validated by:	Andrew Zommers _____ Validation Lead Instructor (Simulator Scenario Validation Checklist.)	_____
Approved by:	Randy Amundson _____ Training Supervision	_____
Approved by:	Tom Larson _____ Training Program Owner	_____

Retention: Life of Plant
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Point Beach Nuclear Plant
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Guide Requirements

Goal of Training: To have the crew successfully perform/respond to shifting Accident Fans, Pressurizer Level Transmitter failure, RCS leak, Rapid Power Reduction, SBLOCA, failure of RCS Hot Leg CI valves with leak and a failure of a Safety Injection Pump. Embedded within these events is the expectation to properly utilize Technical Specifications.

Learning Objectives: None

Prerequisites:

1. Simulator available
2. Students enrolled in Initial License Program

Training Resources:

1. Floor Instructor as Shift Manager/Shift Technical Advisor
2. Simulator Booth Operator
3. Communicator
4. Evaluators

References:

1. OI-72, Containment Air Recirculation System
2. AOP-1D, Chemical and Volume Control System Malfunction
3. AOP-1A, Reactor Coolant Leak Unit 1
4. 0-SOP-IC-001 – White, Routine Maintenance Procedure Removal of Safeguards or Protection Sensor from Service - White Channels
5. 0-SOP-IC-002, Technical Specifications LCO-Instrument Cross Reference
6. AOP-17A Rapid Power Reduction Unit 1
7. AOP-24 Response to Instrument Malfunction
8. AOP-21 PPCS Malfunction
9. EOP-0 Reactor Trip or Safety Injection Unit 1
10. EOP-1 Loss of Reactor or Secondary Coolant Unit 1
11. EOP-1.2 Small Break LOCA Unit 1
12. ARB 1C04 1C 2-9, Containment or Aux Bldg Vent System Air Flow Low
13. Alarm Response Books
14. Technical Specifications and Technical Specifications Bases

Commitments: None

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Evaluation Method:

Simulator performance will be evaluated IAW NUREG 1021.

Operating Experience:

N/A

Related PRA Information:

Initiating Event with Core Damage Frequency:

Small LOCA 1.2E-07

Important Components:

Reactor Protection 0.3%

ESFAS 0.2%

Important Operator Actions with Task Number:

None

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QUANTITATIVE ATTRIBUTES (Use this form for Evaluations only.)

Malfunctions:

Before EOP Entry:

1. 1LT-427 Pressurizer Level Instrument transmitter fails low
2. Loop 'B' RTD Bypass Line leak
3. 1W-3A CRDM Shroud Fan Trips/Rapid downpower

After EOP Entry:

1. 1P-15A SI Pump seized shaft
2. 1P-15B SI Pump failure to auto start
3. MFIV's fail to close
4. RCS Hot Leg Containment Isolation failures with 1 gpm leak

Abnormal Events:

1. 1LT-427 Pressurizer Level Instrument transmitter fails low
2. 20 gpm RCS leak on 'B' RTD loop manifold
3. Rapid Power Reduction

Major Transients:

1. Small Break LOCA

Critical Tasks:

1. **E-0 – O: Close Containment Isolation valves such that at least one valve is closed on each critical penetration before the end of the scenario.**
2. **E-0 – I: Manually start at least one SI pump prior to transition out of EOP-0.**

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SCENARIO OVERVIEW:

INITIAL CONDITIONS:

1. This scenario can be run from the following Specific IC set:
 - IC-162, created from IC-03
2. The following equipment is OOS:
 - Line-111 Point Beach to Sheboygan Energy Center is OOS with disconnects open
 - 'B' Main Feedwater Regulating Valve controlling channels shifted to YELLOW
 - 1W-3B, 'B' CRDM Shroud Fan (need OOS tag on Control Board)
 - 1W-3A 'A' CRDM Shroud Fan is Guarded Equipment
 - Red Barrier Tape set up around 1(2) X01 Transformers per AOP-31(Document on White Board behind SM desk)
 - 1PI-2198 Condenser B Pressure Indicator is OOS for I&C calibration

SEQUENCE OF EVENTS:

Event 1: Shift Accident Fans

- Crew starts 'A' Accident Recirc and Cooling Fans and secures 'B' Accident Recirc and Cooling Fans per OI-72

Event 2: 1LT-427, PZR Level Channel Fails Low

- RO recognizes failure of 1LT-427, PZR Level Channel.
- Crew enters AOP-1D and determines 1LT-427 has failed.
- SRO refers to LCO 3.3.1, LCO 3.3.3, and if applicable LCO 3.4.9.
- Crew should swap PZR controlling channel per ARB and restore Letdown and Charging to normal per AOP.

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Event 3: Loop 'B' RTD Bypass Line Leak at 20 GPM RCS Leak

- Loop 'B' RTD bypass line develops a 20 GPM leak
- SRO enters AOP-1A, Reactor Coolant Leak.
- Crew determines that neither SI nor Rx Trip is required and RO raises charging speed to maintain Pressurizer Level.
- Crew enters diagnostic phase of AOP-1A and begins to determine leak location. Crew will possibly isolate Letdown and Charging as part of the diagnostic steps.

Event 4: 1W-3A CRDM Shroud Fan Trips/Rapid Power Reduction

- RO responds to annunciator 1C04 1C 2-9, Containment Vent System Air Flow Low, using ARB
- ARB directs rapid power reduction using AOP-17A, Rapid Power Reduction
- Crew coordinates to lower unit power at 1% per minute with an RCS leak

Event 5: Small Break LOCA (500 gpm)

- Crew recognizes RCS leak has gotten worse
- SRO refers back to continuous action steps and ensures reactor trip and manual SI/CI
- SRO enters EOP network

Event 6: 1P-15A SI Pump Shaft Seizure and 1P-15B SI Pump Fails to Auto Start

- RO recognizes SI pump trip and failure to start
- SRO concurs with RO starting 1P-15B

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Event 7: MFIV's fail to Auto close

- BOP recognizes MFIV failure to close
- SRO concurs with BOP closing MFIV's manually

Event 8: Failure of RCS Hot Leg sample CI valves to shut with 1 gpm leak

- BOP recognizes CI valves fail to close
- SRO concurs with BOP closing CI valves manually

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SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>INITIAL CONDITIONS: Standard IC-162 from IC-03</p> <p>Unit 1</p> <ul style="list-style-type: none"> • Mode: 1 • Burnup: 16000 MWD/MTU • Power: 73.9% • Boron: 125 ppm (EOL) • Temperature: NOT • Pressure: NOP • Xenon: Equilibrium • Rods: Bank D @ 187 steps • Generator: ≈475 MWe <p>Unit 2</p> <ul style="list-style-type: none"> • Mode: 1 • Burnup: 6500 MWD/MTU • Power: 26.9% • Boron: 1170 ppm (MOL) • Temperature: NOT • Pressure: NOP • Xenon: Equilibrium • Rods: Bank D @ 143steps • Generator: ≈ 175 MWe 		

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	SIMULATOR SET UP (perform set up per the “Simulator Setup Checklist”, including entering action items per the “Simulator Input Summary.”)		<ul style="list-style-type: none"> • Line 111 and 1W-3B (OOS tag) • Line 111 disconnects magnets open • Guarded equipment tag for 1W-3A • Red Barrier Tape listed behind SM desk for AOP-31, around 1(2) X01 Transformers • Extra AO name tags • Marked-up copy of AOP-31 • 1PI-2198 Condenser B PI is OOS for I&C calibration
	Simulator Pre-brief:		
	COMPLETE TURNOVER: Review applicable current Unit Status Review relevant At-Power Risk status Review current LCOs not met and Action Requirements Verify crew performs walk down of control boards and reviews turnover checklists.		

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	<p><u>Event 1: Shift Containment Fans per OI-72</u></p> <p>End of evolution: Proceed to next event at Lead Examiner discretion.</p>	<p>SRO</p> <p>BOP</p>	<p>Crew will brief the shifting of containment fans prior to assuming the watch.</p> <p>BOP will start 1W-1A and secure 1W-1B Containment Fans per OI-72.</p>

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	<p><u>EVENT 2: 1LT-427, PZR Level Transmitter Fails Low.</u></p> <p><u>NOTE:</u> AOP-1D actions are on this page, see next page for AOP-24 actions.</p> <p>LCO 3.4.9 addressed by the SRO if TS value reached.</p> <p>If asked by the SRO, Shift Manager directs defeating the failed PZR level channel per the ARB.</p>	<p>RO</p> <p>RO/BOP</p> <p>SRO</p> <p>RO</p> <p>SRO</p> <p>RO</p> <p>Crew</p> <p>BOP</p> <p>Crew</p>	<p>Acknowledges PZR High/Lo Level alarm and informs Crew. RO recognizes loss of Letdown and recommends going to minimum charging.</p> <p>RO/BOP address the ARB's for alarms associated with the PZR Level failure</p> <p>SRO recognizes entry conditions for AOP-1D, CVCS Malfunction and AOP-24 Instrument Failure.</p> <p>Verifies no RCS leak in progress.</p> <p>Addresses notes prior to step 2 and determines 1LT-427 failure section of AOP should be performed. Continues to step 48.</p> <p>RO recognizes if PZR Level exceeds TS value of 52% and informs the SRO. (May occur depending on timing of actions, though not likely when starting at 75% power)</p> <p>Crew should swap to different controlling PZR level per ARB 1C04 1C 1-3 Pressurizer Level High or Low</p> <p>Place White PZR level to defeat</p> <p>Restore Letdown and Charging to normal per AOP-1D along with PZR Heaters.</p>

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	<p>1LT-427, PZR Level Transmitter Fails Low. (cont'd)</p> <p>SM should inform SRO that the 4th license will reference AOP-21.</p> <p>NOTE: IF asked for references, the Shift Technical Advisor will provide the following logic diagrams as referenced in 0-SOP-IC-001-WHITE page 19. Sheets #175, #180 and #181 and 0-SOP-IC-002 in its entirety.</p> <p>Per Lead Examiner, once Letdown is restored with charging in manual or auto, proceed to next event.</p>	<p>SRO</p> <p>RO</p> <p>SRO</p> <p>Crew</p> <p>SRO</p>	<p>Crew may enter AOP-24 Response to Instrument Malfunction prior to swapping controlling channels.</p> <p>Identifies 1LT-427 PZR Level as failed instrument.</p> <p>Checks that instrument is controlling channel for PZR Level Program.</p> <p>Ensures that Charging is in manual and minimum.</p> <p>Return affected parameter to desired value – ensures that PZR level is returning to program value as well as resetting pressurizer heaters.</p> <p>References step regarding performance of AOP-21.</p> <p>Addresses caution regarding currently tripped bistables – this does not apply as no other bistables are tripped.</p> <p>Directs removal of failed instrument IAW 0-SOP-IC-001 WHITE.</p> <p>Restore Charging to Automatic</p> <p>LCO 3.3.1 and LCO 3.3.3 addressed by the SRO.</p> <ul style="list-style-type: none"> • 3.3.1-1 item 8 PZR water level high • 3.3.3-1 item 14 PZR level is <u>MET</u> <p>TSAC 3.3.1.A and 3.3.1.K are entered.</p>

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	<p>Event 3: Loop 'A' RTD Bypass Line Leak at 20 GPM</p> <p>Indications of an RCS leak would be 1RE-211/212 radiation monitors in alarm, rising containment humidity, Sump A rising, VCT level lowering and the automatic charging pump speeding up.</p> <p>SRO may address Technical Specifications at this time due to RCS leakage. If TS not addressed here, a scenario follow-up question will be needed. LCO 3.4.13 is not met due to unidentified leakage exceeding 1 GPM. (TSAC 3.4.13.A)</p> <p>Communication: If sent to locally check Charging Pump Relief valves, report reliefs are not lifting.</p> <p>Continue to next event per Lead Evaluator or determination of leak rate. Ensure charging/letdown is recovered, if it was isolated, prior to downpower.</p> <p>If requested, inform OS1 that the 4th License will perform the OI-55 calculation.</p>	<p>Crew</p> <p>SRO</p> <p>RO</p> <p>SRO</p> <p>SRO</p>	<p>Recognizes indications of leakage inside containment: Rising humidity, Sump 'A' alarm and RMS alarms.</p> <p>Enters AOP-1A, Reactor Coolant Leak.</p> <p>Check SI not required (CA): Adequate PZR Level and Subcooling.</p> <p>Check Rx Trip not required (CA): Charging Aligned to VCT.</p> <p>Check Pressurizer Level (CA): stable or trending to program, Adjusts Charging as needed to maintain PZR level and RCP Lab Seals.</p> <p>Check Pressurizer Pressure: Stable or trending to program.</p> <p>Check Rx Makeup Controls: Set for proper blend, armed and in auto.</p> <p>Request SM notify DCS and implement E-Plan.</p> <p>SRO may Address Tech Specs.</p> <p>Directs performance of steps 7 through 18. These steps include directions to isolate letdown and charging and various other leakage possibilities.</p>

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	Rapid power reduction continued	<p>SRO</p> <p>RO</p> <p>BOP</p> <p>RO</p>	<p>SRO also determines that the unit needs to be taken off line and could direct BOP to set the endpoint at 0% on the turbine.</p> <p>Requests SM to notify PSS.</p> <p>Checks Rod control in Auto.</p> <p>Selects Rate Reduction method, likely Operator Auto – 1st Stage IN per the note prior to step 4.</p> <p>(CA STEP) Commences boration, should refer to ROD 1.3 for power reduction boration and rod bank requirements</p> <p>(CA STEP) Check PZR Pressure – Stable or trending to program</p> <p>(CA STEP) Check PZR Level – Stable at or trending to program</p>

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SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>(cont): Rapid Power Reduction Unit 1</u></p> <p><u>Communication:</u> When requested, TH AO start 1P-99A and B SGFP Seal Water Pumps</p> <p>Per Lead Evaluator, As the leak is unisolable, the Lead Examiner will direct (after approximately a 5% power reduction) when the team is prepared to make the leak larger and booth operator will increase the leak to 500 GPM over 4 minutes.</p>	<p>BOP</p> <p>RO</p> <p>BOP</p> <p>SRO</p> <p>BOP</p>	<p>(CA STEP) Check SG levels – controlling in Auto</p> <p>(CA STEP) Maintain T_{avg} – Checks T_{avg} within limits</p> <p>Check MFW Seal Water Pumps Running - Contacts U1 Turbine Operator to Start 1P-99A and B.</p> <p>Determines endpoint is MODE 3.</p> <p>May transfer auxiliary loads.</p>

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SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<u>Event 5: Small Break LOCA (500 GPM)</u>	Crew	Recognizes increased leak rate and uses continuous action guidance from AOP-1A to trip the reactor.
		RO	Inserts Manual Trip to trip the reactor. RO inserts manual SI and CI also and carries out Immediate Actions of EOP-0.
			Verify Reactor Trip
			Verify Turbine Trip
			Verify Safeguards Buses – At least one 4160 and one 480 V Bus energized.
			Check if SI is Actuated
			Verifies Immediate Action steps.
		SRO	Addresses Fold Out Page. -Trip RCPs if conditions are met
		BOP	Performs Attachment A. (actions listed later)
		RO	Verifies Secondary Heat Sink - Checks S/G levels - Verifies proper AFW flow for heat sink - Maintains S/G levels

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SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	Small Break LOCA (500 GPM) cont'd	RO	Verify RCP Seal Cooling - Checks CCW pump operating - Checks Lab Seal DP (CA STEP) Verify RCS Temp Control Ensure MSIVs are shut. Check PZR PORVs both shut. Check Normal spray valves both shut. Check Aux spray valve shut. Check if RCPs should remain running
		SRO	Start Monitoring CSFST's (CA STEP) Verify Sump Recirc not required Check if Secondary System intact Check if S/G Tubes intact Check if RCS is intact inside containment - NO

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	Small Break LOCA (500 GPM) cont'd	<p>SRO</p> <p>RO</p> <p>BOP</p> <p>RO</p> <p>BOP</p> <p>RO</p>	<p>Transition to EOP-1</p> <p>Check if RCP's should remain running</p> <p>Check if Secondary System is intact</p> <p>(CA STEP) Stabilize intact S/G levels</p> <p>Check Secondary System Radiation Normal</p> <p>(CA STEP) Check PORV and PORV Block Valves</p> <p>Reset SI</p> <p>Reset Containment Isolation</p> <p>Reset 1B03 and 1B04 lockouts</p> <p>Check 4160 Safeguards busses powered</p> <p>Re-establish Instrument Air to Containment</p> <p>Verify Charging Flow</p> <p>Check if SI flow should be terminated</p> <p>(CA STEP) Check if Spray should be stopped</p>

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	<p>Small Break LOCA (500 GPM) cont'd</p> <p>The 4th License will secure the running EDG's.</p> <p>Lead Evaluator can terminate the scenario once Step 16 of EOP-1 is addressed if all critical tasks are met. If the flowpath out of containment via the RCS Hot Leg Sample line is not isolated then the crew should be allowed to transition to ECA-1.2 to address the penetration leak prior to terminating the scenario.</p>	<p>BOP</p> <p>RO</p> <p>BOP</p> <p>Crew</p> <p>SRO</p>	<p>Check if RHR pumps should be stopped</p> <p>Check RCS and S/G pressures</p> <p>Check if EDG's should be stopped</p> <p>Initiate evaluation of plant status</p> <p>Check if RCS is intact outside containment - might answer NO and transition to ECA-1.2 LOCA Outside Containment</p> <p>Check RHR Pump Room High alarms clear</p> <p>Check if RCS cooldown and depressurization required</p> <p>Transition to EOP 1.2 Small Break LOCA</p>

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	<p>EOP 0 Attachment A</p> <p>Unit 2 will enter TSAC 3.7.8.F when SW-2907 and 2908 are opened.</p> <p>Communication: When contacted the PAB AO should report that SW-LW-61 and 62 radwaste SW isolation valves are both shut.</p>	BOP	<p>Verify feedwater isolation – NO, manually shut MFIV’s.</p> <p>Verify containment isolation.</p> <p>Verify AFW actuation.</p> <p>Check SI pumps running.</p> <p>Check RHR pumps running.</p> <p>Check only 1 CCW pump running.</p> <p>Verify Service Water System alignment. BOP should contact the PAB to check 2 local valves.</p> <p>Verify containment accident cooling.</p> <p>Check CR ventilation in mode 5. - CUE PURPLE LIGHT IS LIT</p> <p>Check if main steam lines can remain open.</p> <p>Verify proper SI valve alignment.</p>

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	<p>EOP 0 Attachment A (cont'd)</p> <p>Communication: When contacted, wait 2 minutes and have the AO report CWPH temperature at 65 °F and stable.</p> <p>Communication: When contacted, wait 2 minutes and have the PAB AO report SFP temperature at 75 °F and stable with level at 63 feet and stable.</p>	BOP	<p>(CA STEP) Verify containment spray not required.</p> <p>Verify SI flow.</p> <p>Stop any boration via the blender in progress</p> <p>(CA STEP) Check CSR ventilation operating. - CUE W-13A1 RED LIGHT IS LIT.</p> <p>(CA STEP) Check Computer Room ventilation operating.</p> <p>(CA STEP) Ensure Aux Building filter/exhaust fans operating.</p> <p>(CA STEP) Check AFW Area ventilation operating.</p> <p>(CA STEP) Energize façade Freeze protection at the discretion of Operations Shift Management.</p> <p>(CA STEP) Check CWPH temperature.</p> <p>(CA STEP) Periodically check the status of SFP cooling.</p>

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SCENARIO TIME-LINE:			
SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>Event 6:</u> 1P-15A SI Pump trip and 1P-15B SI Pump fails to AUTO start</p> <p><u>Communications:</u> If contacted, PAB AO reports seized shaft on 1P-15A SI Pump.</p> <p><u>Communications:</u> If contacted TH AO reports breaker for 1P-15A is tripped on overcurrent.</p> <p><u>Communications:</u> If contacted TH AO reports breaker for 1P-15B is normal and state current position.</p> <p>CRITICAL TASK: E-0 – I: Manually start at least one SI pump prior to transition out of EOP-0.</p>	BOP	RO notices 1P-15A pump trip and failure of 1P-15B to start. After immediate actions 1P-15B can be manually started or during EOP-0 Attachment A guidance will be to start 1P-15B manually.
	<p><u>Event 7:</u> MFIV's fail to AUTO close</p>	BOP	During performance of EOP-0 Attachment A step A1, the BOP should recognize the MFIV's did not close as required and take manual action to shut these valves.
	<p><u>Event 8:</u> RCS Hot Leg Sample Line fails to isolate with 1 gpm leak</p> <p>CRITICAL TASK: E-0 – O: Close Containment Isolation valves such that at least one valve is closed on each critical penetration before the end of the scenario.</p>	BOP	During performance of EOP-0 Attachment A step A2, the BOP should recognize the RCS Hot Leg Sample Line did not isolate as required. Procedurally the BOP should initiate a manual CI and take 1SC-966C to closed and inform the SRO.

Retention: Life of Plant
 Retain in: Training Program File
 Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:			
SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>End the scenario by placing the simulator in freeze</p> <p>Inform Examinees they are to remain at their stations and cannot discuss the scenario. They are to wait for any follow up questions the examiners may have.</p>		<p>Crew:</p> <ul style="list-style-type: none"> • No debrief or critique due to this being an evaluated scenario.

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

Scenario preloads

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	SWD	LOA	LOA1SWD001	OPEN			SWYD Line 111 breaker
Preload	SWD	LOA	LOA1SWD002	OPEN			SWYD Line 111 disconnects
Preload	CNM	BKR	BRK1CNM018	6-Fail Cntl Fuse			1W-3B Shroud Fan Breaker

Retention: Life of Plant
 Retain in: Training Program File
 Form retained in accordance with record retention schedule identified in NP 1.3.1.

Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

SIMULATOR INPUT SUMMARY

EVENT 1: Shift Accident Fans per OI-72

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description

Expected Communications

None

LOAs used

None

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 2: 1LT-427 Pressurizer Level fails LOW

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	RCS	XMT	XMT1RCS008A	0	1	20 sec ramp	1LT-427 PZR Level Transmitter

Expected Communications

None

LOAs used

None

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 3: 20 GPM RCS leak 'B' Loop RTD manifold

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	RCS	MAL	MAL1RCS003F	20	3	120 ramp	Loop 'B' RTD Manifold Leak

Expected Communications

PAB AO: If sent to locally check Charging Pump Relief valves, report reliefs are not lifting.

LOAs used

None.

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 4: 1W-3B Shroud Fan trip

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	CNM	BKR	BRK1CNM017	1-Trip	5		1W-3A Shroud Fan Breaker

Expected Communications

If contacted, the TH AO reports breaker for 1W-3A is tripped and cannot be reset if asked to do so.

LOAs used

None

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 5: Small Break LOCA (500 gpm)

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	RCS	MAL	MAL1RCS003F	500		240 ramp	Loop 'B' RTD Manifold Leak Change value of the RCS Leak to 500 GPM

Expected Communications

None

LOAs used

None

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 6: 1P-15A SI Pump trip and 1P-15B SI Pump fails to AUTO start

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	SIS	PMP	PMP1SIS001	2-Shaft Seizure			1P-15A SI Pump Shaft Seizure
Preload	SIS	BKR	BKR1SIS002	4-Fail to Auto Close			1P-15B SI Pump fail to auto start

Expected Communications:

If contacted, PAB AO reports seized shaft on 1P-15A SI Pump.

If contacted TH AO reports breaker for 1P-15A is tripped on overcurrent.

If contacted TH AO reports breaker for 1P-15B is normal and state current position.

LOAs used:

None

Retention: Life of Plant
 Retain in: Training Program File
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 7: MFIV's fail to AUTO close

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	CFW	VLV	VLV1CFW030A	1-Fail to Auto Close			CF-3124 MFIV fails to auto close
Preload	CFW	VLV	VLV1CFW031A	1-Fail to Auto Close			CF-3125 MFIV fails to auto close

Expected Communications:

If contacted, PAB AO reports actual position of valve with no other abnormal indications.

LOAs used:

None

Retention: Life of Plant
 Retain in: Training Program File
 Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 8: RCS Hot Leg Sample Line fails to isolate with 1 gpm leak

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	NSS	VLV	VLV1NSS005D	0.5	7 (JCRFTR)		1SC-955 RCS Hot Leg fails partially open
Preload	NSS	VLV	VLV1NSS009D	1.0			1SC-966C RCS Hot Leg Sample valve fails open
Preload	NSS	FLX	FLX1NSS001	1 gpm	7 (JCRFTR)		HTX 14C Shell Tube Leak at 1 gpm
Preload			X01I121C.eq.1				Conditional for taking 1SC-966C to shut on Panel C01, will override the failed open malfunction so valve can shut manually.

Expected Communications:

None

LOAs used:

None

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

Attach the following information as necessary:

- Simulator Set-up Checklist (before and after training)
- Pre-evaluation Brief Guide (for evaluations only)
- Post-evaluation Critique (for evaluations only)
- Turnover Log

Historical Record: This SEG was developed for the 2012 NRC ILT Examination.

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

1.0 PLANT CONDITIONS:

	UNIT 1	UNIT 2
Time in core life (MWD/MTU):	16000	6500
Reactor power (%):	73.9%	26.9%
Boron concentration (ppm):	125 ppm	1170 ppm
Rod height (CBD @):	CBD @ 187	CBD @ 143
Target AFD	1.5	

2.0 TECHNICAL SPECIFICATION ACTION CONDITIONS IN EFFECT:

<u>TSAC</u>	<u>Description</u>	<u>Reason</u>
None	None	None

3.0 EQUIPMENT OUT OF SERVICE:

- Line 111, Point Beach to Sheboygan Energy Center, is Out of Service for emergent tower repairs.
- 1W-3B Shroud Fan is Out of Service for bearing failure.

4.0 PLANNED EVOLUTIONS:

After shift turnover, start 1W-1A Containment Fans and secure 1W-1B Containment Fans per OI-72.

Retention: Life of Plant

Retain in: Training Program File

Form retained in accordance with record retention schedule identified in NP 1.3.1.

Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

5.0 TURNOVER INFORMATION:

- Safety Monitor is Green.
- Line 111, Point Beach to Sheboygan Energy Center is OOS for emergent tower repairs.
- Unit 1 is at 75% power with Xenon stable per ATC for grid stability.
- AOP-31, Solar Magnetic Disturbance Alert Response, is in effect for both units due to a Kp6 SMD Alert. ATC has notified PBNP that 1X-01 DC Neutral Current (GIC) readings are 32 amps and continuous monitoring is in effect per step 5 and Attachment B. The 4th license is providing the Auxiliary Operators with control room data for Attachment B. ATC is continuing to monitor 2X-01 DC Neutral Current (GIC) readings. Red barrier tape is put up around 1(2) X01 Transformers.
- 1W-3B, Containment Control Rod Shroud Fan, is Out of Service due to bearing failure.
- 1-43/RPI, Rod Position Indication Power Transfer Switch is in Alternate (1Y-02-15) for 1Y-06-21 breaker replacement.
- SG B Feedwater Flow Control and SG B Steam Flow Control Transfer Switches have been selected to Yellow after performance of 1 ICP 02.001 BL, Reactor Protection and Engineered Safety Features Blue Channel Analog 92 Day Surveillance Test.
- Clock time is **real time** and you have the normal shift complement.

Retention: Life of Plant

Retain in: Training Program File

Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

Simulator Scenario Development Checklist

Mark with an X Yes, No or N/A for any of the following. If the answer is No or N/A, include justification for the answer or the corrective action needed to correct the discrepancy after the item.

- | | |
|--|----------------------------------|
| 1. The scenario contains objectives for the desired tasks and relevant human performance tools. | Yes

No
X |
| 2. The scenario identifies key parameter response, expected alarms, and automatic actions associated with the induced perturbations. (This action applies to all SEG's new or revised for those on the ANS/ANSI-3.5-1998 standard. This action is NOT applicable for those on the ANS/ANSI-3.5-1985 standard.) | Yes

No
N/A
X |
| 3. The scenario content adequately addresses the desired tasks, through simulator performance, instructor-led training freezes, or both. | Yes

No
X |
| 4. Plant PRA initiating events, important equipment, and important tasks are identified. | Yes

No
X |
| 5. Turnover information includes a Daily At Power or Shutdown Safety Risk Assessment. | Yes

No
N/A
X |
| 6. The scenario contains procedurally driven success paths. Procedural discrepancies are identified and corrected before training is given. | Yes

No
X |
| 7. The scenario guide includes responses for all anticipated communications to simulated personnel outside the Control Room, based on procedural guidance and standard operating practices. | Yes

No
N/A
X |
| 8. The scenario includes related industry experience. | Yes

No
N/A
X |
| 9. Training elements and specific human performance elements are addressed in the scenario critique guide to be used by the critique facilitator. The critique guide includes standards for expected performance. | Yes

No
N/A
X |

Developer and Reviewer: Once checklist is completed and deficiencies are corrected, sign the cover page.

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

Simulator Scenario Validation Checklist

Mark with an X Yes, No or N/A for any of the following. If the answer is No or N/A, include an explanation after the item.

- | | | |
|--|----------|-----------|
| 1. The desired initial condition(s) could be achieved. | Yes
X | No |
| 2. All malfunctions and other instructor interface items were functional and responded to support the simulator scenario. | Yes
X | No |
| 3. All malfunctions and other instructor interface items were initiated in the same sequence described within the simulator scenario. | Yes
X | No |
| 4. All applicable acceptance criteria were met for procedures that were used to support the simulator scenario. | Yes
X | No |
| 5. During the simulator scenario, observed changes corresponded to expected plant response. | Yes
X | No |
| 6. Did the scenario satisfy the learning or examination objectives without any significant simulator performance issues, or deviations from the approved scenario sequence? If learning objective(s) could not be satisfied, identify the objectives in the Simulator Action Request | Yes
X | No |
| 7. Evaluation: The simulator is capable of being used to satisfy learning or examination objectives without exceptions, significant performance discrepancies, or deviation from the approved scenario sequence. | Yes
X | No N/A |

Discrepancies noted (Check "none" or list items found) None
SMAR = Simulator Action Request

SMAR: _____ SMAR: _____ SMAR: _____ SMAR: _____

Comments: _____

Validator: Sign the cover page only after noted discrepancies are corrected or compensatory actions are taken to ensure quality training.

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

Facility: Point Beach Scenario No.: 2 Op-Test No.: 2012301

Examiners: _____ Operators: _____

Initial Conditions: Line 111, Point Beach to Sheboygan Energy Center, is OOS for tower repairs, 'B' MFRV controlling channel is shifted to YELLOW, 1W-3B CRDM shroud fan is OOS for bearing failure, Solar Magnetic Disturbance alert for GIC readings greater than 30 amps, IRPI in alternate for 1Y-06-21 breaker replacement, 1PI-2198 Condenser B Pressure Indicator is OOS for I&C calibration.

Turnover: Unit 1 is at 100% power. Normal shift routine is anticipated per the plant schedule.

Event No.	Malf. No.	Event Type*	Event Description
1		C-BOP C-SRO TS-SRO	1P-11A CCW Pump trip and 1P-11B CCW Pump fails to auto start
2		C-RO C-SRO	1CV-135 Letdown Line Backpressure Controller oscillations (controller works in manual)
3		TS-SRO	G02 Emergency Diesel Generator Low Starting Air Pressure alarm
4		R-RO N-BOP N-SRO	Solar Magnetic Disturbance AOP-31 1%/hr ramp per AOP-17A, Rapid Power Reduction
5		I-SRO I-RO	1PT-485 Turbine First Stage Pressure fails low with rod motion failure requiring plant trip
6		M-ALL	Failure of BS-2 345 KV Bus Section, H52-31, 13.8 KV cross-tie breaker, G01 and G03 Emergency Diesel Generators (ECA 0.0 entry)
7		C-RO	3 stuck rods post trip

* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SITE: POINT BEACH

SEG # PBN LOI NRC

SEG TITLE: 2012 ILT NRC SCENARIO #2

REV. # 0

PROGRAM: INITIAL LICENSE TRAINING

#: PBN LOI TPD

COURSE: N/A

#: N/A

TOTAL TIME: 2.0 HOURS

Additional signatures may be added as desired.

Developed by:	<u>Andrew Zommers</u> Instructor	<u>7/27/12</u> Date
Reviewed by:	<u>Joey Trudeau</u> Instructor (Simulator Scenario Development Checklist.)	<u> </u> Date
Validated by:	<u>Andrew Zommers</u> Validation Lead Instructor (Simulator Scenario Validation Checklist.)	<u> </u> Date
Approved by:	<u>Randy Amundson</u> Training Supervision	<u> </u> Date
Approved by:	<u>Tom Larson</u> Training Program Owner	<u> </u> Date

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

Guide Requirements

Goal of Training: To have the crew successfully perform/respond to loss of CCW Pump, CV-135 oscillations, G01 EDG low air pressure, Solar Magnetic Disturbance requiring rapid downpower, 1PT-485 failure with Rod Motion failure requiring trip, ECA-0.0 and stuck rods. Embedded within these events is the expectation to properly utilize Technical Specifications.

Learning Objectives: None

Prerequisites:

1. Simulator available
2. Students enrolled in Initial License Program

Training Resources:

1. Floor Instructor as Shift Manager/Shift Technical Advisor
2. Simulator Booth Operator
3. Communicator
4. Evaluators

References:

1. AOP-9B Component Cooling Water Malfunction Unit 1
2. AOP-1D, Chemical and Volume Control System Malfunction Unit 1
3. 0-SOP-IC-001 – White, Routine Maintenance Procedure Removal of Safeguards or Protection Sensor from Service - White Channels
4. 0-SOP-IC-002, Technical Specifications LCO-Instrument Cross Reference
5. AOP-6C Uncontrolled Rod Motion Unit 1
6. AOP-17A Rapid Power Reduction Unit 1
7. AOP-24 Response to Instrument Malfunction
8. AOP-21 PPCS Malfunction
9. AOP-31 Solar Magnetic Disturbance Unit 1
10. EOP-0 Reactor Trip or Safety Injection Unit 1
11. EOP-0.1 Reactor Trip Response Unit 1
12. ECA 0.0 Loss of All AC Unit 1
13. Alarm Response Books
14. Technical Specifications and associated Bases

Commitments: None

Evaluation Method: Simulator performance will be evaluated IAW NUREG 1021.

Retention: Life of Plant
Retain in: Training Program File
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

Operating Experience: N/A

Related PRA Information: **Initiating Event with Core Damage Frequency:**
Transient w/o PCS 7.3E-07
Station Blackout 5.2E-07

Important Components:
Component Cooling Water 2.1% CDF

Important Operator Actions with Task Number:
Operator Fails to Start CCW Pumps
Operator fails to align G04 to 1A06 per ECA 0.0

Retention: Life of Plant
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Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

QUANTITATIVE ATTRIBUTES (Use this form for Evaluations only.)

Malfunctions:

Before EOP Entry:

1. Running CCW Pump trips with no Auto start of standby CCW Pump
2. Failure of CV-135 Letdown Backpressure Controller
3. G02 EDG alarm, low starting air pressure
4. 1PT-485 Turbine First Stage Pressure fails low

After EOP Entry:

1. Bus section 2 lockout with H52-31 H01 to H02 tie breaker failure
2. G01 and G03 EDG failures
3. 3 stuck control rods

Abnormal Events:

1. 'A' CCW Pump Trip
2. 1CV-135 letdown Backpressure controller failure
3. G02 Starting Air Low Pressure
4. Solar Magnetic Disturbance
5. 1PT-485 failure with Rod Control failure

Major Transients:

1. ECA 0.0 Loss of All AC

Critical Tasks:

1. **RT-7, When a reactor trip is procedurally called for, initiate a manual reactor trip prior to automatic rod motion stopping with AFD in 'UNACCEPTABLE OPERATION' region of TRM Figure 6.**
2. **E-0 – C: SRO/BOP must energize at least one AC emergency bus before placing safeguards equipment in pullout per ECA 0.0.**

Retention: Life of Plant

Retain in: Training Program File

Form retained in accordance with record retention schedule identified in NP 1.3.1.

Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

SCENARIO OVERVIEW:

INITIAL CONDITIONS:

1. This scenario can be run from the following Specific IC set:
 - IC-161, created from IC-02
2. The following equipment is OOS:
 - Line-111 Point Beach to Sheboygan Energy Center is OOS with disconnects open
 - 'B' Main Feedwater Regulating Valve controlling channels shifted to YELLOW
 - 1W-3B, 'B' CRDM Shroud Fan (need OOS tag on Control Board)
 - 1W-3A 'A' CRDM Shroud Fan is Guarded Equipment
 - Red Barrier Tape set up around 1(2) X01 Transformers per AOP-31 (Document on White Board behind the SM desk)
 - 1PI-2198 Condenser B Pressure Indicator is OOS for I&C calibration

SEQUENCE OF EVENTS:

Event 1: Running CCW Pump Shaft Seizure with failure of Standby CCW Pump to Auto start.

- Crew recognizes failure and manually starts Standby CCW Pump
- SRO Enters AOP-9B Component Cooling Water System Malfunction
- SRO Addresses Technical Specifications

Event 2: Failure of CV-135 Letdown Backpressure Controller

- RO takes manual control and stabilizes system parameters
- SRO directs crew response with AOP-1D CVCS Malfunction

Event 3: G02 EDG alarm, low starting air pressure

- Crew responds per ARB C02 F 2-1, G-02 Emergency Diesel alarm
- SRO declares G02 OOS
- SRO addresses Technical Specifications

Retention: Life of Plant

Retain in: Training Program File

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

Event 4: Solar Magnetic Disturbance AOP-31/Rapid Power Reduction

- Crew responds to call from AO that neutral current is 65 amps
- AOP-31 directs rapid power reduction using AOP-17A, Rapid Power Reduction
- Crew coordinates to lower unit power at 1% per minute

Event 5: Turbine First Stage Pressure Transmitter PT-485 fails Low

- Crew recognizes failure and takes manual control of rods.
- SRO Enters AOP-6C Uncontrolled Rod Motion
- RO recognizes rod control failure requiring plant trip

Event 6: Failure of BS-2, H52-31, G01, G02, G03 requiring entry into ECA 0.0

- Crew recognizes failures require entry into ECA-0.0
- Crew aligns a safeguards bus via G04

Event 7: 3 stuck rods

- RO recognizes stuck rods during immediate actions
- Attempts another manual trip of reactor
- SRO later determines boration required for stuck rods

Retention: Life of Plant
Retain in: Training Program File
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>INITIAL CONDITIONS: Standard IC from IC-02 (IC-161)</p> <p>Unit 1</p> <ul style="list-style-type: none"> • Mode: 1 • Burnup: 8100 MWD/MTU • Power: 99.7% • Boron: 947 ppm (MOL) • Temperature: NOT • Pressure: NOP • Xenon: Equilibrium • Rods: Bank D @ 220 steps • Generator: ≈619 Mwe <p>Unit 2</p> <ul style="list-style-type: none"> • Mode: 1 • Burnup: 950 MWD/MTU • Power: 99.4% • Boron: 1461 ppm (BOL) • Temperature: NOT • Pressure: NOP • Xenon: Equilibrium • Rods: Bank D @ 220 steps • Generator: ≈ 623 Mwe 		

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:			
SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	SIMULATOR SET UP (perform set up per the “Simulator Setup Checklist”, including entering action items per the “Simulator Input Summary.”)		<ul style="list-style-type: none"> • Line 111 and 1W-3B (OOS tag) • Line 111 disconnects magnets open • Guarded equipment tag for 1W-3A • Red Barrier Tape listed behind SM desk for AOP-31, around 1(2) X01 Transformers • Extra AO name tags • Marked-up copy of AOP-31 • 1PI-2198 Condenser B PI is OOS for I&C calibration
	Simulator Pre-brief:		
	COMPLETE TURNOVER: Review applicable current Unit Status Review relevant At-Power Risk status Review current LCOs not met and Action Requirements Verify crew performs walk down of control boards and reviews turnover checklists.		

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>Event 1: Running CCW Pump Shaft Seizure with failure of Standby CCW Pump to Auto start.</p> <p>Communications: If asked, the PAB AO reports the 'A' CCW Pump inboard motor bearing is hot to the touch.</p> <p>Communications: If asked, the PAB AO will report nothing unusual with the 'B' CCW Pump discharge pressure indicator, it reads 125 psig locally.</p> <p>Communications: If asked, the TH AO will report the 'A' CCW Pump breaker has tripped on overcurrent.</p> <p>NOTE: Crew may elect to place 1P-11A in pullout to clear 'Motor Breaker Trip' alarm.</p> <p>End of evolution: Proceed to next event at Lead Examiner discretion.</p>	<p>BOP</p> <p>SRO</p> <p>Crew</p> <p>SRO</p> <p>BOP/RO</p> <p>SRO</p>	<p>BOP identifies the 'A' CCW Pump failure with the 'B' CCW Pump not auto starting. BOP requests from the SRO to start 'B' CCW Pump manually. (1C03 1D 2-6 and 1C03 1D 3-7)</p> <p>Enter AOP 9B CCW malfunction to address the lost CCW Pump. BOP will verify adequate tank level and start 1P-11B CCW Pump.</p> <p>Crew will address surge tank level which will require no action.</p> <p>Crew will address system leakage requiring no action.</p> <p>Check Reactor Trip</p> <p>Check RHR Status</p> <p>Chemistry will be requested to analyze CCW.</p> <p>OS1 will ask the SM to call DCS and implement Emergency Plan.</p> <p>Send AO's out locally to the pump/breaker and discharge pressure indicator to find any problems.</p> <p>Address LCO 3.7.7 CCW and enter 72 hour TSAC 3.7.7.A</p>

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>Event 2: Failure of CV-135 Letdown Backpressure Controller</u></p> <p><u>Communications:</u> If asked, the PAB AO reports there is nothing abnormal with CV-135.</p> <p><u>NOTE: AOP-24 may be utilized in addition to AOP-1D. (see next page for AOP-24 actions)</u></p> <p>End of evolution: Proceed to next event at Lead Examiner discretion.</p>	<p>RO/SRO</p> <p>SRO</p> <p>RO</p> <p>SRO</p> <p>RO</p> <p>SRO</p> <p>BOP</p>	<p>RO identifies CV-135 pressure controller malfunction and takes controller to manual to control back pressure.</p> <p>Addresses ARB for Letdown High Pressure (1C04 1C 4-6).</p> <p>Enter AOP 1D to address the malfunction and verify actions taken.</p> <p>Check RCS leak not in progress.</p> <p>Reviews foldout page criteria</p> <p>Determine CVCS malfunction – CV-135 pressure control which directs transition to step 36.</p> <p>Establish manual control of CV-135 – 250#</p> <p>Request SM to notify DCS and I&C Duty and Call</p> <p>The crew will have to keep CV-135 in manual control and discuss how this will be done. (may review step 37 RNO)</p> <p>May send out the PAB AO to look at CV-135 locally.</p>

Retention: Life of Plant
 Retain in: Training Program File
 Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>Event 3: G02 EDG alarm, low starting air pressure</p> <p>Communication: When sent out, have the TH1 AO wait 2 minutes and report that G02 south bank air pressure is 190 psig and stable with the north bank 160 psig and slowly lowering. The north bank air compressor is running with one of the bank relief valves leaking by. (ARB C65A C35 P7)</p> <p>NOTE: IF asked by the OS1, the Shift Manager will state OS2 will prepare the required paperwork per OI-35A to align G01 to 2A05. May ask for Diesel Air Start PID, M-209 Sheet 12 (#134).</p> <p>End of evolution: Proceed to next event at Lead Examiner discretion.</p>	<p>BOP</p> <p>SRO</p>	<p>Address G02 alarm on panel C02. Dispatches AO to determine local alarm that is in.</p> <p>When the report comes back to Control the SRO should determine that G02 EDG is OOS.</p> <p>LCO 3.8.3 not met, enter TSAC 3.8.3.D immediately and declare G02 OOS for both units.</p> <p>LCO 3.8.1 not met, enter TSAC 3.8.1.E for required standby emergency power sources inoperable for both units.</p> <p>Determine the need to align G01 to both 1A05 and 2A05.</p> <p>Call in Maintenance to troubleshoot and fix.</p>

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>EVENT 4: SMD AOP-31 / Rapid Power Reduction</u></p> <p><u>Communication:</u> AO at 1X01 Transformers reports that the GSU Neutral Amps are sustained 65 amps for the last 5 minutes.</p> <p><u>Communication:</u> If asked, AO at Unit 2 X01 Transformers reports conditions remain stable.</p>	SRO	<p>Crew is receives a call from AO on Unit 1.</p> <p>Determine that GSU DC Neutral Amps are 65 amps</p> <p>Crew determines rapid downpower per AOP-31 is required.</p> <p>SRO directs rapid power reduction IAW AOP-17A.</p>

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<u>(cont): Rapid Power Reduction Unit 1</u>		
		SRO	Determines desired power level and ramp rate and announces to CR – SRO determines that unit be ramped at 1% per minute as directed by AOP. SRO will direct BOP to set the endpoint at 78% on the turbine. Requests SM to notify PSS.
		RO	Checks Rod control in Auto.
		BOP	Selects Rate Reduction method, likely Operator Auto – 1 st Stage IN per the note prior to step 4.
		RO	(CA STEP) Commences boration, should refer to ROD 1.3 or the reactivity management sheets for power reduction boration and rod bank requirements (CA STEP) Check PZR Pressure – Stable or trending to program (CA STEP) Check PZR Level – Stable at or trending to program

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>(cont): Rapid Power Reduction Unit 1</u></p> <p><u>Communication:</u> When requested, TH AO start 1P-99A and B SGFP Seal Water Pumps</p> <p><u>End of evolution:</u> Proceed to next event at approximately a 5% power reduction or Lead Examiner discretion.</p>	<p>BOP</p> <p>RO</p> <p>BOP</p> <p>SRO</p>	<p>(CA STEP) Check SG levels – controlling in Auto</p> <p>(CA STEP) Maintain T_{avg} – Checks T_{avg} within limits</p> <p>Check MFW Seal Water Pumps Running - Contacts U1 Turbine Operator to Start 1P-99A and B.</p> <p>Determines endpoint is <50% - NO.</p>

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>Event 5: Turbine First Stage Pressure Transmitter PT-485 fails Low and rod motion fails requiring plant trip</p> <p>RT-7, When a reactor trip is procedurally called for, initiate a manual reactor trip prior to automatic rod motion stopping with AFD in 'UNACCEPTABLE OPERATION' region of TRM Figure 6.</p>	<p>RO</p> <p>SRO</p> <p>RO</p>	<p>Identifies inward rod motion, recommends placing rod control in Manual</p> <p>Concurs with recommendation, directs placing rods in Manual</p> <p>Rod Motion not responding, recommends reactor trip</p> <p>Inserts Manual Trip to trip the reactor.</p> <p>Verify Reactor Trip</p> <p>Verify Turbine Trip</p> <p>Verify Safeguards Buses</p> <p>Determine no 4160 and 480 safeguards busses are energized for Unit 1.</p> <p>Try to fast start any EDG's</p> <p>Try to load any running EDG's</p> <p>If power cannot be restored, have the STA start monitoring Unit 1 CSFST's and transition to ECA 0.0.</p>

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>Event 6: Bus Section 2 H52-31, G-01, G-02 and G-03 EDG failures causing entry in ECA 0.0</p> <p>Communication: If asked, wait 2 minutes and the TH AO reports G03 has indications and alarms locally that it oversped.</p> <p>Communication: If asked, wait 2 minutes and the TH AO reports G01 output breaker is open and nothing appears abnormal.</p> <p>Communication: If asked, wait 2 minutes and the TH AO reports G02 EDG is not running and everything looks normal.</p> <p>Communication: If asked, wait 3 minutes and the TH AO can report that H52-31 tie breaker is tripped and there are no other abnormal indications.</p> <p>NOTE: Crew may elect to go right to ECA 0.0 without taking any actions per EOP 0 Step 3 RNO. Entry conditions are met.</p>	<p>SRO</p> <p>RO</p> <p>SRO</p> <p>RO/BOP</p> <p>BOP</p>	<p>Enter ECA 0.0</p> <p>Verify Reactor tripped</p> <p>Verify Turbine tripped</p> <p>Monitor foldout page items</p> <p>Ensure both RCP's - Stopped</p> <p>Maintain RCS inventory – RO and BOP will check various vent valves, sample valves, letdown and PORV's shut.</p> <p>Verify TDAFW Pump operating</p> <p>Check cooling to AFW Pump bearings</p> <p>Check both 4160 VAC busses not energized</p>

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>ECA 0.0 (cont'd)</p> <p>NOTE: Additional SW Pumps may be started per ARB to ensure adequate SW header pressure.</p> <p>Communication: If the SM is asked, direct OS1 that the 3rd RO will start G05.</p> <p>Communication: SM will concur with OS1 recommendations concerning EDG operation (Starting or securing.)</p>	<p>BOP</p> <p>SRO</p> <p>BOP</p> <p>BOP/SRO</p> <p>BOP</p> <p>BOP/SRO</p> <p>BOP</p> <p>SRO</p>	<p>Check Diesels – All Running</p> <p>Check Diesel Status</p> <p>Check H-01 not energized – Start G05 and continue</p> <p>Check 1A-05 not energized</p> <p>Check 1A-05 energized from G01 – breaker won't close go to step 14</p> <p>Energize 1A-05 from G02 – G02 did not start</p> <p>Check 1A-06 not energized</p> <p>Energize 1A-06 from G03 – cannot be done due to G03 overspeed condition. - Go to step 17.</p> <p>Energize 1A-06 from G04 – Manually shut alternate breaker.</p> <p>Check 480 Vac busses at least one energized – Yes</p>

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>ECA 0.0 (cont'd)</p> <p>E-0 C, SRO/BOP must energize at least one AC emergency bus before placing safeguards equipment in pullout per ECA 0.0</p> <p>Communication: If asked, OS2/4th License will take step 7 RNO action.</p>	<p>BOP</p> <p>SRO</p> <p>SRO</p> <p>RO/BOP</p> <p>SRO</p> <p>RO</p> <p>BOP</p>	<p>Verify SW System operation</p> <p>Return to procedure step in effect, which was EOP 0 step 3 RNO</p> <p>Verify safeguards power is available for at least one(1) 4160 and 480 VAC bus</p> <p>Check if SI is actuated – NO go to EOP-0.1</p> <p>Transition to EOP-0.1</p> <p>(CA STEP) Verify RCS Temperature Control using Atmospheric Steam Dumps</p> <p>Verify Feedwater Isolation</p> <p>Transfer Feedwater Control to the Bypasses - NO</p> <p>(CA STEP) Stabilize S/G Levels</p> <p>Verify Generator Trip</p> <p>Ensure Miscellaneous Electrical Loads are Energized</p>

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:			
SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>ECA 0.0 (cont'd)</p> <p>Communication: If asked, OS2/4th License will take step 7 RNO action.</p> <p>Communication: If asked, the PAB AO verifies that 1CV-112 B is SHUT and 1CV-112C is OPEN.</p> <p>The scenario can be terminated at Lead Evaluators discretion or once action has been taken to start the boration.</p>	<p>BOP</p> <p>RO</p>	<p>Check All AC Busses Energized by Offsite Power</p> <p>Verify Charging Pump Suction Alignment</p> <p>Verify Charging Flow</p> <p>Check ALL Control Rods Fully Inserted – NO</p> <p>Check Pressurizer Level >12%</p>
	<p>Event 7: 3 stuck rods</p>	<p>RO</p> <p>Crew</p>	<p>Recognizes 3 control rod stuck after reactor trip.</p> <p>May attempt another trip with second set of trip buttons.</p> <p>While in EOP-0.1 recall there are 3 stuck control rods and take actions per the RNO to borate.</p> <p>Take action to borate for 3 stuck rods by adding 2825 gallons of acid for each stuck rod.</p>
	<p>End the scenario by placing the simulator in freeze</p> <p>Inform Examinees they are to remain at their stations and cannot discuss the scenario. They are to wait for any follow up questions the examiners may have.</p>		<p>Crew:</p> <ul style="list-style-type: none"> No debrief or critique due to this being an evaluated scenario.

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

Scenario preloads

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	SWD	LOA	LOA1SWD001	OPEN			SWYD Line 111 breaker
Preload	SWD	LOA	LOA1SWD002	OPEN			SWYD Line 111 disconnects
Preload	CNM	BKR	BRK1CNM018	6-Fail Cntl Fuse			1W-3B Shroud Fan Breaker
Preload							
Preload							

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 1: Loss of 1P-11A and failure of 1P-11B to auto start

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	CCW	BKR	BKR1CCW002	4-Fail to Auto Close			1P-11B CCW Pump fails to auto start
PLE	CCW	PMP	PMP1CCW001	2-Shaft Seizure	1		1P-11A CCW Pump shaft seizure

Expected Communications

If asked, the PAB AO reports the ‘A’ CCW Pump inboard motor bearing is hot to the touch.

If asked, the PAB AO will report nothing unusual with the ‘B’ CCW Pump discharge pressure indicator, it reads 125 psig locally.

If asked, the TH AO will report the ‘A’ CCW Pump breaker has tripped on overcurrent.

LOAs used

None

Retention: Life of Plant

Retain in: Training Program File

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 2: 1CV-135 Letdown Backpressure Controller failure

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	CVC	CNH	CNH1CVC014F	50	3		CV-135 controller oscillations in automatic

Expected Communications

If PAB AO is sent out, inform control room there is nothing abnormal at valve 1CV-135 Letdown backpressure control valve

LOAs used

Retention: Life of Plant
Retain in: Training Program File
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 3: G02 EDG alarm, low starting air pressure

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	G02	ANN	ANN-C02F-B01	0-ON	5		G02 EDG alarm

Expected Communications

When sent out, have the TH1 AO wait 2 minutes and report that G02 south bank air pressure is 190 psig and stable with the north bank 160 psig and slowly lowering. The north bank air compressor is running with one of the bank relief valves is leaking by.

LOAs used

None.

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 4: Solar Magnetic Disturbance requiring rapid plant downpower

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	CFW	LOA	LOA1CFW083	ON	14		1P-99A SGFP Seal Water Pump
PLE	CFW	LOA	LOA1CFW084	ON	14	30 sec delay	1P-99B SGFP Seal Water Pump

Expected Communications

- **When requested, TH AO starts 1P-99A and B SGFP Seal Water Pumps (Trigger 14)**
- **AO at 1X01 Transformers reports that the GSU Neutral Amps are sustained 65 amps for the last 5 minutes.**
- **If asked, AO at Unit 2 X01 Transformers reports nothing has changed.**

LOA used Trigger 14 for 1P-99A/B

Retention: Life of Plant
 Retain in: Training Program File
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 5 1PT-485 First Stage Pressure fails LOW with Rod Motion Failure requiring plant trip

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	MSS	XMT	XMT1MSS008A	0	7	30 sec ramp	1PT-485 First Stage Pressure fails low
PLE	CRF	OVR	OVR-CRF009D	ON	7		Rod Control Selector Switch in AUTO
PLE	CRF	OVR	OVR-CRF009D	OFF	7		Rod Control Selector Switch in MAN

Expected Communications

None

LOAs used

None

Retention: Life of Plant
Retain in: Training Program File
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 6: Failure of BS-2, H52-31, G01, G02, G03 requiring entry into ECA 0.0

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	DSG	MAL	MAL1DSG002B				G03 overspeed during startup
Preload	DSG	BKR	BKR1DSG001	5-fail as is			G01 Output Breaker fail as is 1A52-60
Preload	SWD	BKR	BKR1SWD008	1-trip	9	JCRFTR, 30 sec delay	H52-31 breaker
Preload	DSG	BKR	BKR1DSG004	6-fail close fuse	9		G04 Output Breaker fail to close
PLE	SWD	MAL	MAL1SWD004B		9	JCRFTR, 30 sec delay	345 Bus Section 2 Lockout
PLE	DSG	MAL	MAL2DSG001A				G-02 EDG Failure to Start

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

Expected Communications:

- **If asked, wait 2 minutes and the TH AO reports G01 appears normal**
- **If asked, wait 2 minutes and the TH AO reports G03 has indications and alarms locally that it oversped.**
- **If asked, wait 2 minutes and the TH AO reports G02 output breaker is open and nothing appears abnormal**
- **If asked, wait 3 minutes and the TH AO can report that H52-31 has tripped and there are no other abnormal indications.**
- **When contacted the PAB AO should report that SW-LW-61 and 62 radwaste SW isolation valves are both shut.**
- **When contacted, wait 2 minutes and have the AO report CWPH temperature is at 65°F and stable.**
- **When asked PAB AO will be directed to locally shut 1SI-896 and 1SI-856 MOV's.**

LOAs used:

- **None.**

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 7: 3 stuck rods

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	CRF	MAL	MAL1CRF001-B6	1-Non Trip	7		Stuck Rod – B6
PLE	CRF	MAL	MAL1CRF001-D10	1-Non Trip	7		Stuck Rod – D10
PLE	CRF	MAL	MAL1CRF001-F12	1-Non Trip	7		Stuck Rod – F12

Expected Communications

None

LOAs used

None

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Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

Attach the following information as necessary:

- Simulator Set-up Checklist (before and after training)
- Pre-evaluation Brief Guide (for evaluations only)
- Post-evaluation Critique (for evaluations only)
- Turnover Log

Historical Record: This SEG was developed for the 2012 NRC ILT Examination.

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

1.0 PLANT CONDITIONS:

	UNIT 1	UNIT 2
Time in core life (MWD/MTU):	8100	950
Reactor power (%):	99.7%	99.4%
Boron concentration (ppm):	947 ppm	1461 ppm
Rod height (CBD @):	CBD @ 220	CBD @ 220
Target AFD	-4.6	

2.0 TECHNICAL SPECIFICATION ACTION CONDITIONS IN EFFECT:

<u>TSAC</u>	<u>Description</u>	<u>Reason</u>
None	None	None

3.0 EQUIPMENT OUT OF SERVICE:

- Line 111, Point Beach to Sheboygan Energy Center, is Out of Service for emergent tower repairs.
- 1W-3B Shroud Fan is Out of Service for bearing failure.

4.0 PLANNED EVOLUTIONS:

- Normal Shift Routine

Retention: Life of Plant
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Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

5.0 TURNOVER INFORMATION:

- Safety Monitor is Green
- Line 111, Point Beach to Sheboygan Energy Center is OOS for emergent tower repairs.
- AOP-31, Solar Magnetic Disturbance Alert Response, is in effect for both units due to a Kp6 SMD Alert. ATC has notified PBNP that 1X-01 DC Neutral Current (GIC) readings are 32 amps and continuous monitoring is in effect per step 5 and Attachment B. The 4th license is providing the Auxiliary Operators with control room data for Attachment B. ATC is continuing to monitor 2X-01 DC Neutral Current (GIC) readings. Red barrier tape is put up around 1(2) X01 Transformers.
- 1W-3B, Containment Control Rod Shroud Fan, is Out of Service due to bearing failure.
- 1-43/RPI, Rod Position Indication Power Transfer Switch is in Alternate (1Y-02-15) for 1Y-06-21 breaker replacement.
- SG B Feedwater Flow Control and SG B Steam Flow Control Transfer Switches have been selected to Yellow after performance of 1 ICP 02.001 BL, Reactor Protection and Engineered Safety Features Blue Channel Analog 92 Day Surveillance Test.
- Clock time is **real time** and you have the normal shift complement.

Retention: Life of Plant

Retain in: Training Program File

Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

Simulator Scenario Development Checklist

Mark with an X Yes, No or N/A for any of the following. If the answer is No or N/A, include justification for the answer or the corrective action needed to correct the discrepancy after the item.

- | | |
|--|----------------------------------|
| 1. The scenario contains objectives for the desired tasks and relevant human performance tools. | Yes

No
X |
| 2. The scenario identifies key parameter response, expected alarms, and automatic actions associated with the induced perturbations. (This action applies to all SEG's new or revised for those on the ANS/ANSI-3.5-1998 standard. This action is NOT applicable for those on the ANS/ANSI-3.5-1985 standard.) | Yes

No
N/A
X |
| 3. The scenario content adequately addresses the desired tasks, through simulator performance, instructor-led training freezes, or both. | Yes

No
X |
| 4. Plant PRA initiating events, important equipment, and important tasks are identified. | Yes

No
X |
| 5. Turnover information includes a Daily At Power or Shutdown Safety Risk Assessment. | Yes

No
N/A
X |
| 6. The scenario contains procedurally driven success paths. Procedural discrepancies are identified and corrected before training is given. | Yes

No
X |
| 7. The scenario guide includes responses for all anticipated communications to simulated personnel outside the Control Room, based on procedural guidance and standard operating practices. | Yes

No
N/A
X |
| 8. The scenario includes related industry experience. | Yes

No
N/A
X |
| 9. Training elements and specific human performance elements are addressed in the scenario critique guide to be used by the critique facilitator. The critique guide includes standards for expected performance. | Yes

No
N/A
X |

Developer and Reviewer: Once checklist is completed and deficiencies are corrected, sign the cover page.

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

Simulator Scenario Validation Checklist

Mark with an X Yes, No or N/A for any of the following. If the answer is No or N/A, include an explanation after the item.

- | | | | |
|--|----------|----|-----|
| 1. The desired initial condition(s) could be achieved. | Yes
X | No | |
| 2. All malfunctions and other instructor interface items were functional and responded to support the simulator scenario. | Yes
X | No | |
| 3. All malfunctions and other instructor interface items were initiated in the same sequence described within the simulator scenario. | Yes
X | No | |
| 4. All applicable acceptance criteria were met for procedures that were used to support the simulator scenario. | Yes
X | No | |
| 5. During the simulator scenario, observed changes corresponded to expected plant response. | Yes
X | No | |
| 6. Did the scenario satisfy the learning or examination objectives without any significant simulator performance issues, or deviations from the approved scenario sequence? If learning objective(s) could not be satisfied, identify the objectives in the Simulator Action Request | Yes
X | No | |
| 7. Evaluation: The simulator is capable of being used to satisfy learning or examination objectives without exceptions, significant performance discrepancies, or deviation from the approved scenario sequence. | Yes
X | No | N/A |

Discrepancies noted (Check "none" or list items found) None
 SMAR = Simulator Action Request

SMAR: _____ SMAR: _____ SMAR: _____ SMAR: _____

Comments: _____

Validator: Sign the cover page only after noted discrepancies are corrected or compensatory actions are taken to ensure quality training.

Retention: Life of Plant
 Retain in: Training Program File
 Form retained in accordance with record retention schedule identified in NP 1.3.1.

Facility: Point Beach	Scenario No.: <u> 3 </u>	Op-Test No.: <u> 2012301 </u>	
Examiners: _____	Operators: _____	_____	
_____	_____	_____	
_____	_____	_____	
<p>Initial Conditions: Line 111, Point Beach to Sheboygan Energy Center, is OOS for tower repairs, 'B' MFRV controlling channel is shifted to YELLOW, 1W-3B CRDM shroud fan is OOS for bearing failure, Solar Magnetic Disturbance alert for GIC readings greater than 30 amps, IRPI in alternate for 1Y-06-21 breaker replacement, 1PI-2198 Condenser B Pressure Indicator is OOS for I&C calibration,</p>			
<p>Turnover: Unit 1 is at 45% power per OP 1C Startup to Power Operation step through 5.39.3. After turnover, you are to start a second train of feedwater pumps per the remainder of Step 5.39, then raise power at 12% per hour to 75% power.</p>			

Event No.	Malf. No.	Event Type*	Event Description
1		N-BOP N-SRO	Start 2 nd train of Feedwater Pump
2		R-RO N-BOP N-SRO	OP-1C, Startup to Power Operation, up power 12%/hr from 45%
3		C-RO C-SRO	1RC-431K Pressurizer Pressure Controller fails LOW
4		I-BOP I-SRO TS-SRO	1PT-468 'A' Steam Generator Pressure (Red) fails HIGH
5		M-ALL	'B' Steam Line leak inside containment requiring plant trip
6		TS-SRO	1W-1A Containment Accident Fan fails to start
7		C-BOP	Both Main Steam Isolation Valves fail to auto close
8		C-RO	SI fails to automatically actuate
* (N)ormal, (R)eactivity, (I)nstrument, (C)omponent, (M)ajor			

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SITE: POINT BEACH

SEG # PBN LOI NRC

SEG TITLE: 2012 ILT NRC SCENARIO #3

REV. # 0

PROGRAM: INITIAL LICENSE TRAINING

#: PBN LOI TPD

COURSE: N/A

#: N/A

TOTAL TIME: 2.0 HOURS

Additional signatures may be added as desired.

Developed by:	Andrew Zommers _____ Instructor	
Reviewed by:	Joey Trudeau _____ Instructor (Simulator Scenario Development Checklist.)	
Validated by:	Andrew Zommers _____ Validation Lead Instructor (Simulator Scenario Validation Checklist.)	
Approved by:	Randy Amundson _____ Training Supervision	
Approved by:	Tom Larson _____ Training Program Owner	

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

Guide Requirements

Goal of Training:

To have the crew successfully perform/respond to Starting a second train of feedwater, normal up power from 45%, Pressurizer Pressure Controller fails low, PT-468 fails high, 'B' Steam Line leak inside containment, failure of 'A' Accident Fan to start, 'B' MSIV fails to close and auto SI failure. Embedded within these events is the expectation to properly utilize Technical Specifications.

Learning Objectives:

None

Prerequisites:

1. Simulator available
 2. Students enrolled in Initial License Program
-

Training Resources:

1. Floor Instructor as Shift Manager/Shift Technical Advisor
 2. Simulator Booth Operator
 3. Communicator
 4. Evaluators
-

References:

1. OP-1C, Startup to Power Operation Unit 1
 2. 0-SOP-IC-001 – RED, Routine Maintenance Procedure Removal of Safeguards or Protection Sensor from Service - Red Channels
 3. 0-SOP-IC-002, Technical Specifications LCO-Instrument Cross Reference
 4. AOP-24 Response to Instrument Malfunction
 5. AOP-21 PPCS Malfunction
 6. AOP-2A Secondary Coolant Leak Unit 1
 7. AOP-2B Secondary Malfunction Unit 1
 8. EOP-0 Reactor Trip or Safety Injection Unit 1
 9. EOP-2 Faulted Steam Generator Unit 1
 10. Technical Specifications
 11. Technical Specifications Bases
-

Commitments:

None

Evaluation Method:

Simulator performance will be evaluated IAW NUREG 1021.

Retention: Life of Plant

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

**Operating
Experience:**

N/A

**Related PRA
Information:**

Initiating Event with Core Damage Frequency:

SLB/FB in Containment 9.8E-7

Important Components:

Main Steam 1.2% CDF

Important Operator Actions with Task Number:

None

Retention: Life of Plant

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Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

QUANTITATIVE ATTRIBUTES (Use this form for Evaluations only.)

Malfunctions:

Before EOP Entry:

1. 1W-1A Accident Fan fails to start
2. 1PT-468 'A' S/G Pressure Transmitter fails HIGH
3. 1RC-431K Pressurizer Pressure Controller fails LOW
4. 'B' Steam Line leak inside containment

After EOP Entry:

1. Both MSIV fail to auto close
2. Auto Safety Injection failure

Abnormal Events:

1. 1PT-468 'A' S/G Pressure Transmitter fails HIGH
2. 1RC-431K Pressurizer Pressure Controller fails LOW
3. 'B' Steam Line leak inside containment

Major Transients:

1. 'B' Steam Line Break inside containment

Critical Tasks:

- 1. E-0-D: Manually actuate at least one train of safety injection prior to exiting EOP-0.**
- 2. E-2 A: Isolate the faulted steam generator before transition out of EOP-2.**

Retention: Life of Plant

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Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

SCENARIO OVERVIEW:

INITIAL CONDITIONS:

1. This scenario can be run from the following Specific IC set:
 - IC-163, created from IC-04
2. The following equipment is OOS:
 - Line-111 Point Beach to Sheboygan Energy Center is OOS with disconnects open
 - 'B' Main Feedwater Regulating Valve controlling channels shifted to YELLOW
 - 1W-3B, 'B' CRDM Shroud Fan (need OOS tag on Control Board)
 - 1W-3A 'A' CRDM Shroud Fan is Guarded Equipment.
 - Red Barrier Tape set up around 1(2) X01 Transformers per AOP-31 (Document on White Board behind the SM desk)
 - 1PI-2198 Condenser B Pressure Indicator is OOS for I&C calibration

SEQUENCE OF EVENTS:

Event 1: Start a Second Feed Train

- Crew starts a second feed train per OP-1C

Event 2: Perform a Normal Up Power from 45%

- Crew performs normal up power per OP-1C

Event 3: 1HC-431K Pressurizer Pressure Controller fails LOW

- RO recognizes spray valve controller failure
- SRO enters AOP-24

Event 4: 1PT-468 'A' S/G pressure Transmitter fails HIGH

- Crew recognizes transmitter failure
- BOP takes manual control of 'A' S/G Atmospheric Steam Dump and 'A' MRFV
- SRO enters AOP-2B and/or AOP-24

Retention: Life of Plant

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

- SRO addresses Technical Specifications

Event 5: 'B' Steam Line Leak in containment requiring plant trip

- Crew identifies secondary leak in containment
- SRO enters AOP-2A Secondary Coolant Leak

Event 6: 1W-1A Accident Fan fails to start

- Crew recognizes failure of 1W-1A to start
- SRO addresses Technical Specifications

Event 7: Both MSIV's fails to auto close

- BOP recognizes MSIV's failed to shut when required

Event 8: Automatic Safety Injection failure

- RO recognizes SI failed to actuate when required
- Perform manual SI and CI

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>INITIAL CONDITIONS: Standard IC-163 from IC-04</p> <p>Unit 1</p> <ul style="list-style-type: none"> • Mode: 1 • Burnup: 1000 MWD/MTU • Power: 45.6% • Boron: 1737 ppm (BOL) • Temperature: NOT • Pressure: NOP • Xenon: Equilibrium • Rods: Bank D @ 129 steps • Generator: ≈283 MWe <p>Unit 2</p> <ul style="list-style-type: none"> • Mode: 1 • Burnup: 9100 MWD/MTU • Power: 99.6% • Boron: 857 ppm (MOL) • Temperature: NOT • Pressure: NOP • Xenon: Equilibrium • Rods: Bank D @ 220 steps • Generator: ≈ 623 MWe 		

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:			
SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	SIMULATOR SET UP (perform set up per the “Simulator Setup Checklist”, including entering action items per the “Simulator Input Summary.”)		<ul style="list-style-type: none"> • Line 111 and 1W-3B (OOS tag) • Line 111 disconnects magnets open • Guarded equipment tag for 1W-3A • Red Barrier Tape listed behind SM desk for AOP-31, around 1(2) X01 Transformers • Extra AO name tags • Marked-up copy of AOP-31 • 1PI-2198 Condenser B PI is OOS for I&C calibration • Verify PPCS Blowdown Flows are 5 klbs below actual
	Simulator Pre-brief:		
	<p>COMPLETE TURNOVER:</p> <p>Review applicable current Unit Status</p> <p>Review relevant At-Power Risk status</p> <p>Review current LCOs not met and Action Requirements</p> <p>Verify crew performs walk down of control boards and reviews turnover checklists.</p>		

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>Event 1: Start Second Feed Train per OP-1C</u></p> <p><u>Communication:</u> If asked, the TH AO will report that 1CS-182B, 1P-25B-M Cond Pump motor bearing cooling Condensate return to suction, was adjusted for 30 GPM to motor bearing cooler. (Step 5.39.6)</p> <p><u>Communication:</u> If asked, the TH AO will report that 1CS-50 and IA-434 are OPEN. (Step 5.39.9)</p> <p><u>Communication:</u> If asked, the TH AO will report that the supply breaker for the 1P-28B SGFP is RACKED IN with the 125 VDC Trip and Close fuse blocks on. (Step 5.39.9)</p> <p><u>Communication:</u> If asked, the TH AO will report that the Top Case Temp of 305F, Bottom Case Temp of 305F and SGFP Discharge Temp of 305F and all stable for 1P-28B. (Step 5.39.10)</p> <p><u>Communication:</u> If asked, the TH AO will report that the 1P-73A control switch is positioned in AUTO, prior to placing switch to TEST. (Step 5.39.16.b.1)</p> <p><u>Communication:</u> If asked, the TH AO will report that 1P-73A has not restarted and oil pressure is NORMAL at 18 psig and stable.</p>	<p>SRO</p> <p>BOP</p>	<p>Crew will brief the evolution prior to entering the simulator.</p> <p>Start 1P-25B Condensate Pump.</p> <p>Verify starting conditions for 1P-28B SGFP.</p> <p>Position 1P-28B to AUTO and verify mini-recirc valve operation.</p> <p>Start 1P-28B SGFP.</p> <p>Verify operation of second feed train.</p> <p>Have AO secure AC Oil Pump for 1P-28B.</p>

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>EVENT 1: (Cont'd)</u></p> <p><u>Communication:</u> If asked, the TH AO will report that no moisture is dripping from or steam leaving either bracket and the seal outlet temperatures is at 165°F. (Step 5.39.17 a & b)</p> <p><u>Communication:</u> If asked, the TH AO will report that 1TI-3642B indicates 115°F and you will continue to monitor. (Step 5.39.18.b)</p> <p><u>NOTE:</u> Once step 5.39.18.b communication is complete, inform crew that one (1) hour has elapsed with stable SGFP oil temperatures and that they are to proceed with power ascension starting at step 5.39.18.d of OP-1C.</p> <p>End of evolution: Proceed to next event at Lead Examiner discretion.</p>		
	<p><u>EVENT 2: Normal Up Power from 45%. Per OP-1C</u></p> <p>End of evolution: Proceed to next event after noticeable power increase or at Lead Examiner discretion.</p>	<p>Crew</p> <p>RO</p>	<p>Briefing on power ascension may be performed in classroom prior to beginning of the scenario.</p> <p>Withdraws rods and/or Dilute in addition to adjusting steam demand as needed to establish power increase.</p>

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>EVENT 4: 1PT-468 'A' S/G Pressure Transmitter fails HIGH</p> <p>Communications: SM may be requested to contact I&C to troubleshoot and repair PT-468.</p> <p>Communications: If requested by OS1, SM agrees to let Crew take 'A' FRV to auto per ARB guidance by shifting controlling feed flow channels (1C03 1E2 2-2 or 3-2).</p> <p>NOTE: Crew may initially enter either AOP-2B or AOP-24</p> <p>Communications: If requested by OS1 inform them OS2 will be preparing the 0-SOP-IC-001 RED to remove 1PT-468 from service.</p> <p>NOTE: IF asked for references, the Shift Technical Advisor will provide the following logic diagrams as referenced in 0-SOP-IC-001-RED page 29. Sheets #169, #172 and #179 and 0-SOP-IC-002 in its entirety.</p>	<p>BOP/SRO</p> <p>RO</p> <p>BOP</p> <p>SRO</p> <p>BOP</p>	<p>Acknowledge and respond to receipt of annunciators. Realize feed control issues and S/G Atmospheric opening are due to instrument failure and place affected controllers in Manual.</p> <p>The crew may go to "hold" on the load ramp.</p> <p>Crew enters AOP-2B.</p> <p>(CA Step) Maintain Power <100%</p> <p>Determine Secondary malfunction – FRV response</p> <p>Check Feed Regulating Valve Response</p> <p>Place affected controller in manual if desires</p> <p>Stabilize S/G level</p> <p>If transient due to instrument failure go to AOP-24</p> <p>Crew enters AOP-24</p> <p>Identify Failed Instrument:</p> <p>Check if failed instrument is a controlling channel:</p> <p>Establish Manual Control</p>

Retention: Life of Plant

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>EVENT 4: (Cont'd)</u></p> <p>End of evolution: Proceed to next event at Lead Examiner discretion.</p> <p><u>NOTE:</u> Upon initiation of next event, takes about 7 minutes before first PPCS alarm comes in.</p>		<p>1FT-464</p> <ul style="list-style-type: none"> • Table 3.3.1-1 item 14-2 SF/FF Mismatch • Table 3.3.2-1 item 4d-1 SLI-High Steam Flow • Table 3.3.2-1 item 4e-1 SLI-High High Steam Flow <p>TSAC 3.3.1.A and D, TSAC 3.3.2.A and D.</p>

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>Event 5 (and 6): 'B' Steam Line Break inside Containment (Failure of 'A' Accident Fan)</p> <p>NOTE: Once the crew attempts to start the 'A' Containment Accident Fan or per Lead Evaluator, raise the severity of the SLB to 1E006 lbm/hr.</p>	<p>Crew</p> <p>SRO</p> <p>RO/BOP</p> <p>BOP</p> <p>Crew</p> <p>SRO</p> <p>RO</p>	<p>Containment humidity will rise, Sump A level will rise, no RMS alarms and reactor power rise. Crew should diagnose a secondary fault inside containment and trip the reactor prior to reaching 5 psig in containment.</p> <p>Enter AOP-2A</p> <p>(CA STEP) Determine Secondary Leakage NOT Hazardous to Personnel or Equipment</p> <p>(CA STEP) Maintain Plant within limits</p> <p>(CA STEP) Maintain RCS Tavg</p> <p>Check Containment Conditions Normal – NO</p> <p>RNO – Start All Containment Accident Fans – NO, 1W-1A Accident Fan Fails to start (See page 18 for details)</p> <p>Make determination that SLB is getting worse, Trip Unit based on CA STEP 1, SRO may direct manual SI and manual CI</p> <p>Enter EOP-0 after reactor trip</p> <p>Verify Reactor Tripped</p> <p>Verify Turbine Tripped</p> <p>Verify Safeguards Busses Energized</p>

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>Steam Line breaks (cont'd)</u></p> <p>CRITICAL TASK: E-0-D: Manually actuate at least one train of safety injection prior to exiting EOP-0.</p>	<p>RO</p> <p>BOP</p> <p>RO</p> <p>SRO</p> <p>RO</p> <p>BOP</p>	<p>Check if SI is actuated - No perform manual SI/CI</p> <p>Isolate Feed Flow to 'B' S/G per Foldout Page</p> <p>Perform Attachment A</p> <p>Verify Secondary Heat Sink</p> <p>Verify RCP Seal cooling</p> <p>(CA STEP) Verify RCS Temperature Control</p> <p>Check PZR PORV's shut</p> <p>Check PZR Spray Valves shut</p> <p>Check if RCP's should remain running</p> <p>Start monitoring critical safety function status trees</p> <p>(CA STEP) Verify containment sump recirc not required</p> <p>Check if Secondary System is intact – No</p> <p>Transition to EOP-2</p> <p>Check RCS wide range hot leg temperatures stable</p> <p>Isolate both main steam lines</p> <p>Check if any S/G not faulted</p> <p>Identify Faulted S/G</p> <p>Reset Loss of Feedwater Turbine Trip</p>

Retention: Life of Plant

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>Steam Line breaks (cont'd)</u></p> <p>CRITICAL TASK: E-2 A: Isolate the faulted steam generator before transition out of EOP-2.</p> <p>End of evaluation: Lead evaluator can end scenario any time after the Transition to EOP-1.</p>	<p>BOP</p> <p>RO</p> <p>SRO</p>	<p>Isolate Feed to the Faulted S/G</p> <p>Isolate Flow from the Faulted S/G</p> <p>Check CST Level – Greater than 4 ft</p> <p>Check Secondary System Radiation Normal</p> <p>Transition to EOP-1.</p>

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:			
SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p><u>Event 6: 1W-1A1 Accident Fan fails to start</u></p> <p>NOTE: If the movement of the scenario is such that the SRO does not verbalize the LCO is not met for the failed Accident Fan, a follow up question after the scenario ends may be required.</p>	<p>BOP</p> <p>SRO</p>	<p>Recognize 1W-1A1 failure to start during performance of AOP-2A.</p> <p>Declare 1W-1A OOS and address Technical Specifications.</p> <p>LCO 3.6.6 NOT MET and TSAC 3.6.6.C entered</p>
	<p><u>Event 7: Both MSIV's fail to auto close</u></p> <p>NOTE: Depending on timing of actions in the scenario the BOP may not get this malfunction. The MSIV's should be checked closed in EOP-0 Attachment A. It may be possible for the RO to shut these valves during Step 8 of EOP-0 for temperature control.</p>	<p>BOP</p>	<p>After immediate actions the Crew may notice both MSIV's are still open. If the Crew does not notice either Attachment A of EOP-0, ECA 2.1 or EOP-2 actions will try to shut the MSIV's.</p>
	<p><u>Event 8: Failure of SI to automatically actuate</u></p>	<p>RO</p>	<p>During performance of immediate actions the crew may recognize that the Safety Injection signal failed to automatically actuate. The crew will be required to manually actuate SI and CI.</p>

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>EOP-0 Attachment A</p> <p>Unit 2 will enter TSAC 3.7.8.F when SW-2907 and 2908 are opened.</p> <p>Communication: When contacted the PAB AO should report that SW-LW-61 and 62 Radwaste SW isolation valves are both shut.</p>	BOP	<p>Verify Feedwater isolation.</p> <p>Verify Containment isolation.</p> <p>Verify AFW actuation.</p> <p>Check SI pumps running.</p> <p>Check RHR pumps running.</p> <p>Check only 1 CCW pump running.</p> <p>Verify Service Water System alignment. BOP should contact the PAB to check 2 local valves.</p> <p>Verify Containment Accident Cooling.</p> <p>Check CR Ventilation in mode 5. - CUE PURPLE LIGHT LIT</p> <p>Check if Main Steam lines can remain open. NO, BOP should close. This may be completed by the RO depending on scenario timing.</p> <p>Verify proper SI valve alignment.</p>

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:

SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>EOP 0 Attachment A (cont'd)</p> <p>Communication: When contacted, wait 2 minutes and have the AO report CWPH temperature at 70°F and stable.</p> <p>Communication: When contacted, wait 2 minutes and have the PAB AO report SFP temperature at 75°F and stable with level at 63 feet and stable.</p>	BOP	<p>(CA STEP) Verify Containment Spray not required.</p> <p>Verify SI flow.</p> <p>Stop any boration via the blender in progress</p> <p>(CA STEP) Check CSR ventilation operating. - CUE W13A1 RED LIGHT IS LIT</p> <p>(CA STEP) Check Computer Room ventilation operating.</p> <p>(CA STEP) Ensure Aux Building filter/exhaust fans operating.</p> <p>(CA STEP) Check AFW Area ventilation operating.</p> <p>(CA STEP) Energize façade Freeze protection at the discretion of Operations Shift Management.</p> <p>(CA STEP) Check CWPH temperature.</p> <p>(CA STEP) Periodically check the status of SFP cooling.</p>

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SCENARIO TIME-LINE:			
SEQ	SEQUENCE OF EVENTS / INSTRUCTOR NOTES	CREW POS	EXPECTED STUDENT RESPONSES
	<p>End the scenario by placing the simulator in freeze</p> <p>Inform Examinees they are to remain at their stations and cannot discuss the scenario. They are to wait for any follow up questions the examiners may have.</p>		<p>Crew:</p> <ul style="list-style-type: none"> • No debrief or critique due to this being an evaluated scenario.

Retention: Life of Plant
 Retain in: Training Program File
 Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

Scenario preloads

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	SWD	LOA	LOA1SWD001	OPEN			SWYD Line 111 breaker
Preload	SWD	LOA	LOA1SWD002	OPEN			SWYD Line 111 disconnects
Preload	CNM	BKR	BRK1CNM018	6-Fail Cntl Fuse			1W-3B Shroud Fan Breaker

Retention: Life of Plant
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 1: Start Second Feed Train per OP-1C

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	CFW	LOA	LOA1CFW079	OFF	12		1P-73A AC Lube oil pump for SGFP B
							Also need to place back to auto after using trigger 12

Expected Communications

- If asked, the TH AO will report that 1CS-182B, 1P-25B-M Cond Pump motor bearing cooling Condensate return to suction, was adjusted for 30 GPM to motor bearing cooler. (Step 5.39.6)
- If asked, the TH AO will report that 1CS-50 and IA-434 are OPEN. (Step 5.39.9)
- If asked, the TH AO will report that the supply breaker for the 1P-28B SGFP is RACKED IN with the 125 VDC Trip and Close fuse blocks on. (Step 5.39.9)
- If asked, the TH AO will report that the Top Case Temp of 305F, Bottom Case Temp of 305F and SGFP Discharge Temp of 305F and all stable for 1P-28B. (Step 5.39.10)
- If asked, the TH AO will report that the 1P-73A control switch is positioned in AUTO, prior to placing switch to TEST. (Step 5.39.16.b.1)
- If asked, the TH AO will report that 1P-73A has not restarted and oil pressure is NORMAL at 18 psig and stable.
- If asked, the TH AO will report that no moisture is dripping from or steam leaving either bracket and the seal outlet temperatures is at 165°F. (Step 5.39.17 a & b)
- If asked, the TH AO will report that 1TI-3642B indicates 115°F and you will continue to monitor. (Step 5.39.18.b)

LOAs used

- When order to place 1P-73A test switch to TEST, initiate Trigger 12 then return switch position back to AUTO.

Retention: Life of Plant

Retain in: Training Program File

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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 2: Raise power per OP-1C from 45%

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description

Expected Communications

None

LOAs used

None

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 3: 1HC-431K Pressurizer Pressure Controller fails LOW

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	PCS	CNH	CNH1PCS009E	0	1	15 sec ramp	431K PZR Pressure Controller failure

Expected Communications

None.

LOAs used

None.

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 4: 1PT-468 'A' S/G Pressure Transmitter fails HIGH

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	SGN	MAL	MAL1SGN015A	1400	3	15 sec ramp	PT-468 fails HIGH

Expected Communications

None

LOAs used

None

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 5 'B' Steam Line leak inside Containment

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
PLE	SGN	MAL	MAL1SGN003B	3000	5	60 sec ramp	B steam leak inside containment (6.5 min to humidity alarm)
PLE	SGN	MAL	MAL1SGN003B	1E006			Change B steam leak inside containment PLE
PLE	SGN	LOA	LOA1SGN025	0	14		1MS-237 AFP/ Radwaste Steam

Expected Communications

- **If actions are directed for the AO to go near the steam lines near containment, inform control room you cannot gain access. Monitor 'B' S/G status, when the S/G is blown down, then actions can be performed, report only residual steam.**
- **When contacted the PAB AO should report that SW-LW-61 and 62 Radwaste SW isolation valves are both shut.**
- **When contacted, wait 2 minutes and have the AO report CWPH temperature at 70°F and stable.**
- **When contacted, wait 2 minutes and have the PAB AO report SFP temperature at 75 °F and stable with level at 63 feet and stable**

LOA used Trigger 14 to shut 1MS-237. 1MS-238 is not modeled.

Retention: Life of Plant
 Retain in: Training Program File
 Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 6: 1W-1A Accident Fan fails to start

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	CNM	BKR	BKR1CNM001	5-Fail as is			1W-1A Accident Fan breaker fail as is

Expected Communications:

None

LOAs used:

None

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

SIMULATOR INPUT SUMMARY

EVENT 7: Both MSIV's fail to auto close

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	PPL	BST	BST1PPL068	1-fail as is			TC401A fails as is
Preload	PPL	BST	BST1PPL069	1-fail as is			TC402D fails as is
Preload	PPL	BST	BST1PPL070	1-fail as is			TC403D fails as is
Preload	PPL	BST	BST1PPL058	1-fail as is			Cont High Pressure fails as is
Preload	PPL	BST	BST1PPL062	1-fail as is			Cont High Pressure fails as is
Preload	PPL	BST	BST1PPL014	1-fail as is			Cont High Pressure fails as is
Preload	PPL	BST	BST1PPL016	1-fail as is			Cont High Pressure fails as is

Expected Communications:

None

LOAs used:

None

Retention: Life of Plant
Retain in: Training Program File
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**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

EVENT 8: Safety Injection fails to Automatically actuate

Relative Order	System Or Panel Drawing	Type	Code	Severity Or Value	Event Trigger	Timing	Description
Preload	PPL	RLY	RLY1PPL0078	2-Fail as is			Fail Auto SI Train A
Preload	PPL	RLY	RLY1PPL0079	2-Fail as is			Fail Auto SI Train B

Expected Communications:

None

LOAs used:

None

Retention: Life of Plant
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Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

Attach the following information as necessary:

- Simulator Set-up Checklist (before and after training)
- Pre-evaluation Brief Guide (for evaluations only)
- Post-evaluation Critique (for evaluations only)
- Turnover Log

Historical Record: This SEG was developed for the 2012 NRC ILT Examination.

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Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

1.0 PLANT CONDITIONS:

	UNIT 1	UNIT 2
Time in core life (MWD/MTU):	1000	9100
Reactor power (%):	45.6%	99.6%
Boron concentration (ppm):	1737 ppm	857 ppm
Rod height (CBD @):	CBD @ 129	CBD @ 220
Target AFD	-0.83	

2.0 TECHNICAL SPECIFICATION ACTION CONDITIONS IN EFFECT:

<u>TSAC</u>	<u>Description</u>	<u>Reason</u>
None	None	None

3.0 EQUIPMENT OUT OF SERVICE:

- Line 111, Point Beach to Sheboygan Energy Center, is Out of Service for emergent tower repairs.
- 1W-3B Shroud Fan is Out of Service for bearing failure.

4.0 PLANNED EVOLUTIONS:

After shift turnover, start a second train of feedwater per OP-1C Startup to Power Operation Step 5.39.4.

Once the second feed train is started raise power at 12% per hour up to 75% power.

Retention: Life of Plant
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Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

5.0 TURNOVER INFORMATION:

- Safety Monitor is Green.
- Line 111, Point Beach to Sheboygan Energy Center is OOS for emergent tower repairs.
- AOP-31, Solar Magnetic Disturbance Alert Response, is in effect for both units due to a Kp6 SMD Alert. ATC has notified PBNP that 1X-01 DC Neutral Current (GIC) readings are 32 amps and continuous monitoring is in effect per step 5 and Attachment B. The 4th license is providing the Auxiliary Operators with control room data for Attachment B. ATC is continuing to monitor 2X-01 DC Neutral Current (GIC) readings. Red barrier tape is put up around 1(2) X01 Transformers.
- 1W-3B, Containment Control Rod Shroud Fan, is Out of Service due to bearing failure.
- 1-43/RPI, Rod Position Indication Power Transfer Switch is in Alternate (1Y-02-15) for 1Y-06-21 breaker replacement.
- SG B Feedwater Flow Control and SG B Steam Flow Control Transfer Switches have been selected to Yellow after performance of 1 ICP 02.001 BL, Reactor Protection and Engineered Safety Features Blue Channel Analog 92 Day Surveillance Test.
- Clock time is **real time** and you have the normal shift complement.

Retention: Life of Plant

Retain in: Training Program File

Form retained in accordance with record retention schedule identified in NP 1.3.1.

**Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)**

Simulator Scenario Development Checklist

Mark with an X Yes, No or N/A for any of the following. If the answer is No or N/A, include justification for the answer or the corrective action needed to correct the discrepancy after the item.

- | | |
|--|--|
| 1. The scenario contains objectives for the desired tasks and relevant human performance tools. | Yes

No

<u>X</u> |
| 2. The scenario identifies key parameter response, expected alarms, and automatic actions associated with the induced perturbations. (This action applies to all SEG's new or revised for those on the ANS/ANSI-3.5-1998 standard. This action is NOT applicable for those on the ANS/ANSI-3.5-1985 standard.) | Yes

No

N/A

<u>X</u> |
| 3. The scenario content adequately addresses the desired tasks, through simulator performance, instructor-led training freezes, or both. | Yes

No

<u>X</u> |
| 4. Plant PRA initiating events, important equipment, and important tasks are identified. | Yes

No

<u>X</u> |
| 5. Turnover information includes a Daily At Power or Shutdown Safety Risk Assessment. | Yes

No

N/A

<u>X</u> |
| 6. The scenario contains procedurally driven success paths. Procedural discrepancies are identified and corrected before training is given. | Yes

No

<u>X</u> |
| 7. The scenario guide includes responses for all anticipated communications to simulated personnel outside the Control Room, based on procedural guidance and standard operating practices. | Yes

No

N/A

<u>X</u> |
| 8. The scenario includes related industry experience. | Yes

No

N/A

<u>X</u> |
| 9. Training elements and specific human performance elements are addressed in the scenario critique guide to be used by the critique facilitator. The critique guide includes standards for expected performance. | Yes

No

N/A

<u>X</u> |

Developer and Reviewer: Once checklist is completed and deficiencies are corrected, sign the cover page.

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.

Point Beach Nuclear Plant
SIMULATOR EXERCISE GUIDE (SEG)

Simulator Scenario Validation Checklist

Mark with an X Yes, No or N/A for any of the following. If the answer is No or N/A, include an explanation after the item.

- | | | |
|--|----------|-----------|
| 1. The desired initial condition(s) could be achieved. | Yes
X | No |
| 2. All malfunctions and other instructor interface items were functional and responded to support the simulator scenario. | Yes
X | No |
| 3. All malfunctions and other instructor interface items were initiated in the same sequence described within the simulator scenario. | Yes
X | No |
| 4. All applicable acceptance criteria were met for procedures that were used to support the simulator scenario. | Yes
X | No |
| 5. During the simulator scenario, observed changes corresponded to expected plant response. | Yes
X | No |
| 6. Did the scenario satisfy the learning or examination objectives without any significant simulator performance issues, or deviations from the approved scenario sequence? If learning objective(s) could not be satisfied, identify the objectives in the Simulator Action Request | Yes
X | No |
| 7. Evaluation: The simulator is capable of being used to satisfy learning or examination objectives without exceptions, significant performance discrepancies, or deviation from the approved scenario sequence. | Yes
X | No N/A |

Discrepancies noted (Check "none" or list items found) None
SMAR = Simulator Action Request

SMAR: _____ SMAR: _____ SMAR: _____ SMAR: _____

Comments: _____

Validator: Sign the cover page only after noted discrepancies are corrected or compensatory actions are taken to ensure quality training.

Retention: Life of Plant
Retain in: Training Program File
Form retained in accordance with record retention schedule identified in NP 1.3.1.