

February 25, 2013

Frederick P. "Ted" Schiffley, II  
BWROG, Chairman  
c/o GE Hitachi Nuclear Energy,  
P.O. Box 780, 3901 Castle Hayne Road,  
M/C A-70, Wilmington, NC 28402

SUBJECT: USE OF CONTAINMENT ACCIDENT PRESSURE IN DEMONSTRATING  
ACCEPTABLE OPERATION OF EMERGENCY CORE COOLING SYSTEM  
AND CONTAINMENT HEAT REMOVAL PUMPS DURING POSTULATED  
ACCIDENTS

Dear Mr. Schiffley:

U.S. Nuclear Regulatory Commission (NRC) licensees, including many licensees operating Boiling Water Reactors (BWRs), have included the use of containment accident pressure (CAP) in demonstrating acceptable operation of the emergency core cooling system (ECCS) and containment heat removal pumps during postulated accidents.

The NRC staff undertook a review of this issue and issued a draft guidance document for your review by letter dated March 1, 2010, Agencywide Documents Access and Management System (ADAMS) Accession No. ML100550903. The draft guidance document is available in ADAMS under the Accession No. ML100550869.

The NRC staff has revised the draft guidance document to provide additional clarifications, including the addition of a flow chart entitled "Flow Chart for LOCA Containment and NPSH Analyses" and its accompanying sketches "Figure a", and "Figure b". This flow chart along with the sketches provide a snapshot summary of the required analyses to be performed to support power uprate and other license amendment applications to demonstrate adequate net positive suction head (NPSH) to the ECCS and containment heat removal pumps. The flow chart clarifies the guidance for the analyses to be performed and does not reflect changes in the guidance from the previous version of the draft guidance document.

We have enclosed the revised draft guidance document, entitled "Nuclear Regulatory Commission Draft Guidance for the Use of Containment Accident Pressure in Determining the Net Positive Suction Head Margin for Emergency Core Cooling System and Containment Heat Removal Pumps in Boiling Water and Pressurized Water Reactors," and ask that you share it with interested members of the BWR community. The document is also available in ADAMS under Accession No. ML13015A437. The NRC staff intends to use this draft guidance in engaging stakeholders and issue an Interim Guidance Document in the near future.

F. Schiffler

- 2 -

If you have any questions regarding this matter, please contact Joseph Golla at (301) 415-1002.

Sincerely,

*/RA/*

Anthony J. Mendiola, Chief  
Licensing Processes Branch  
Division of Policy and Rulemaking  
Office of Nuclear Reactor Regulation

Project No. 691

Enclosure:  
As stated

cc w/encl: See next page

F. Schiffley

- 2 -

If you have any questions regarding this matter, please contact Joseph Golla at (301) 415-1002.

Sincerely,

*/RA/*

Anthony J. Mendiola, Chief  
Licensing Processes Branch  
Division of Policy and Rulemaking  
Office of Nuclear Reactor Regulation

Project No. 691

Enclosure:  
As stated

cc w/encl: See next page

DISTRIBUTION

PUBLIC	RDennig	RidsNrrOd	ASallman
RidsNrrDpr	RKaripineni	RidsNrrDprPlpb	RidsNrrLADBaxley
AMendiola	RidsOpaMail	JGolla	RidsOgcMailCenter
RidsNroOd	RidsNrrDss	RidsResOd	RidsNrrDssScvb
RidsAcrsAcnwMailCenter			

**ADAMS Accession Nos.: (Package) ML13017A463; (Cover Letter) ML13016A013;  
(Enclosure) ML13015A437**

**NR-106**

<b>OFFICE</b>	PLPB/PM	PLPB/LA	SCVB/BC	DSS/D	PLPB/BC
<b>NAME</b>	JGolla	(GLappert for) DBaxley	RDennig	TMcGinty	AMendiola
<b>DATE</b>	02/14/2013	02/05/2013	02/14/2013	02/14/2013	02/25/2013

**OFFICIAL RECORD COPY**

BWR Owner's Group  
cc:

Project No. 691

Mr. Frederick Schiffley  
BWROG Chairman  
c/o GE Hitachi Nuclear Energy,  
P.O. Box 780, 3901 Castle Hayne Road,  
M/C A-70, Wilmington, NC 28402  
[frederick.schiffley@exeloncorp.com](mailto:frederick.schiffley@exeloncorp.com)

Mr. Kenneth McCall  
BWROG, Program Manager  
GE-Hitachi Nuclear Energy  
PO Box 780 M/C F12  
3901 Castle Hayne Road  
Wilmington, NC 28402  
[kenneth.mccall@ge.com](mailto:kenneth.mccall@ge.com)