

# Spent Fuel Transportation Risk Assessment (SFTRA) Draft NUREG Rev. 2.3

SFST Overview 2/8/2012



# Purpose of Briefing/Status of staff review

- Approval to submit Draft SFTRA NUREG Rev. 2.3 to publications for editing by Feb. 15, 2012.
- Yesterday's BC SFTRA briefing outcome proceed with editing by publications
- · SFST's SFTRA Review Team Appreciation Reo. 10 May 220
  - Gordon Bjorkman structural
  - Chris Bajwa, TCB thermal and overall message
  - Bob Einziger, SMMB fuels, source term
  - Anita Gray health physics
- Review team member comments have been incorporated in Rev. 2.3, and all review team members concur in publishing Rev. 2.3 for Public Comment
- Oak Ridge Technical Peer Review Team
  - Matt Feldman
  - Cecil Parks
  - Other technical staff
- · All ORNL comments incorporated in Rev 2.3



#### SFTRA Purpose and goals

- Continuing review
  - FEIS (NUREG-0170)
  - "Modal Study" (NUREG/CR-4829)
  - "Reexamination..." (NUREG/CR-6672)
- NRC's safety mission
  - Considering public comment, provide updated basis for conclusion that NRC's regulations applicable to spent fuel transportation provide adequate public health and safety
- · Outreach responsibilities
  - Reassure public regarding spent fuel shipments
    - · Basic message: Risks are low so safety is high
    - · Improve public understanding and acceptance of spent fuel shipments
- · Update benchmark for environmental assessments
- Potential shipments
  - Significant issue when study began (2006) much less so now (post Yucca Mtn shutdown)
  - Nevertheless applicable to future shipments

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# **SFTRA Basic Methodology**

- Perform finite element analysis of cask response to impact and thermal accident conditions
- Use DOT "event trees" to estimate probabilities of accident conditions
- Use RADTRAN to calculate routine doses and accident dose risks for representative truck and rail shipments
- Approach similar to that in NUREG-0170 and NUREG/CR-6672



# SFTRA improvements over previous NRC spent fuel risk studies

- New rail and truck event trees
- RADTRAN new Version 6:
  - Elevated releases
  - New loss of shielding analysis
- Updated population data (2000 Census; trying to update to 2010)
- Updated traffic density and accident data for truck and rail
- Hi-fidelity HI-STAR 100 and NAC-STC cask finite element models, including impact limiters
- Direct loaded and welded inner canister
- More precise structural (e.g., bolt model) and thermal (e.g., 3-D) analyses
  - better estimate of cask-to-environment release fractions

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## SFTRA Report Structure and Format

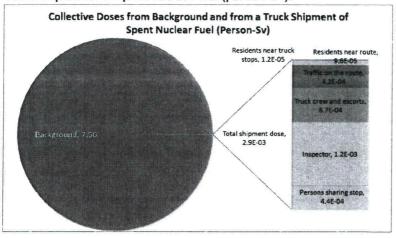
- Audience
  - Public, media, industry, states, elected officials, federal agencies
- Graded structure and content
  - MD 3.7 and NUREG-0650
- Executive Summary and Public Summary [All audiences]
- Main body text [informed public, states, science media]
- Appendices [industry, other federal agencies]
- Electronic and printed versions planned (latter may be limited)

Key dotta results 154 pages, 118 Table, 202 Figures



#### **SFTRA Results: Routine conditions**

Collective doses from background and from Maine Yankee to ORNL truck shipments of spent nuclear fuel (person-Sv).

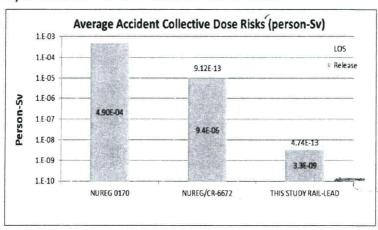


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U.S.NRC
United States Nuclear Regulatory Commission
Protecting People and the Environment

## **SFTRA Results: Accident conditions**

Accident collective dose risks from release and loss of shielding (LOS) accidents. The LOS bars are not to scale.



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## **SFTRA Findings**

- The collective dose risks from routine transportation are vanishingly small.
   Theses doses are about four to five orders of magnitude less than collective background radiation dose.
- The routes selected for this study adequately represent the routes for spent nuclear fuel transport, and there was relatively little variation in the risks per kilometer over these routes.
- Radioactive material would not be released in an accident if the fuel is contained in an inner welded canister inside the cask.
- Only rail casks without inner welded canisters would release radioactive material, and only then in exceptionally severe accidents.
- If there were an accident during a spent fuel shipment, there is only about one in a billion chance the accident would result in a release of radioactive material.
- If there were a release of radioactive material in a spent fuel shipment accident, the dose to the maximum exposed individual would be less than 2 Sv, about the dose given in a single radiotherapy treatment to cancer patients.

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## SFTRA Findings cont'd

- The collective dose risks for the two types of extra-regulatory accidents (accidents involving a release of radioactive material and loss of lead shielding accidents) are negligible compared to the risk from a no-release, no-loss of shielding accident.
- · The risk of loss of shielding from a fire is negligible.
- None of the fire accidents investigated in this study resulted in a release of radioactive material.

new accident



#### **SFTRA Conclusion**

Based on these findings, this study reconfirms that radiological impacts from spent fuel transportation conducted in compliance with NRC regulations are low, in fact generally less than previous, already low, estimates.

Accordingly, with respect to spent fuel transportation, the previous NRC conclusion that the regulations for transportation of radioactive material are adequate to protect the public against unreasonable risk is also reconfirmed by this study.

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#### **SFTRA Current Schedule**

Milestone	Date
1. Submit Rev 2.3 to publications	for NRC edit 2/15/2012
2. Publications returns edited cop	y 3/15/2012
3. Publish for comment in Fed Re	g 4/15/2012
4. Public comments due	6/15/2012
5. Sandia response to public com 3.0)	ments (Rev
6. ACRS subcommittee review	8-9/15/2012 (unscheduled)
7. Sandia delivers final Draft NUI	REG (Rev. 4.0) 9/30/2012 (contract expires)
8. NRC publishes Final NUREG	By 12/31/2012

Plan Bitollow-on controct (n'HOK) for ACRS-directors revisions or other as director. Pat in place now; tooks optional, at discretion of FM



#### **SFTRA Challenges**

- External:
  - Possible post-Fukushima public apprehension over nuclear activities
  - Policy-based opposition by certain environmental groups
- · Internal:
  - Extent/response effort for public comments may exceed that planned
  - Placeholder to update population data to 2010 Census.
  - ACRS review schedule not under our control
  - Sandia contract expires 9/30/2012

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2) - Approved to setup AHDR follow-on continued w/SAIL,

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