10 CFR 50.46(a)(3)(ii)



Serial: RNP-RA/12-0117

# DEC 0 4 2012

Attn: Document Control Desk United States Nuclear Regulatory Commission Washington, DC 20555-0001

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2 DOCKET NO. 50-261/ RENEWED LICENSE NO. DPR-23

#### ANNUAL REPORT OF CHANGES TO OR ERRORS DISCOVERED IN AN ACCEPTABLE LOSS-OF-COOLANT ACCIDENT EVALUATION MODEL APPLICATION FOR THE EMERGENCY CORE COOLING SYSTEM

Ladies and Gentlemen:

In accordance with the provisions of the Code of Federal Regulations, Title 10, Part 50.46 (10 CFR 50.46), Carolina Power & Light Company (CP&L), now doing business as Progress Energy, hereby submits the attached report of non-significant changes to and errors discovered in an acceptable Loss-of-Coolant Accident (LOCA) evaluation model (EM) for the Emergency Core Cooling System at the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2. The applicable LOCA EMs are referenced in the HBRSEP, Unit No. 2, Core Operating Limits Report. This submittal satisfies the 10 CFR 50.46(a)(3)(ii) requirement for annual reporting of LOCA EM changes for HBRSEP, Unit No. 2.

The last annual report was submitted to the Nuclear Regulatory Commission by letter dated November 23, 2011 (ML11340A075). A subsequent letter dated April 12, 2012 (ML12128A057) was submitted to provide notification of a significant change. The April 12, 2012 letter also included the non-significant changes through March 16, 2012. Therefore, this annual report provides the changes covering the period of March 17, 2012 through November 06, 2012. The non-significant changes and error corrections are provided in Attachment I. The latest peak cladding temperature (PCT) estimates for the Large Break (LB) LOCA and Small Break (SB) LOCA are included in Attachment II.

This document contains no new Regulatory Commitments. If you have any questions, please contact me at (843) 857-1329.

Sincerely,

Richard Hightown Richard Hightower

Richard Hightower Supervisor – Licensing/Regulatory Programs



Progress Energy Carolinas, Inc. Robinson Nuclear Plant 3581 West Entrance Road Hartsville, SC 29550 United States Nuclear Regulatory Commission Serial: RNP-RA/12-0117 Page 2 of 2

## WRH/msc

Attachments:

- I. Report of Changes/Errors in Loss-of-Coolant Accident Evaluation Models for the Emergency Core Cooling System
- II. Peak Cladding Temperature Estimates
- c: V. M. McCree, NRC, Region II A. T. Billoch Colón, NRC Project Manager, NRR NRC Resident Inspector, HBRSEP

#### H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

### **REPORT OF CHANGES/ERRORS IN LOSS-OF-COOLANT ACCIDENT EVALUATION MODELS FOR THE EMERGENCY CORE COOLING SYSTEM**

This report provides an estimate of the effect on peak cladding temperature (PCT) of changes and error corrections in the Loss-of-Coolant Accident (LOCA) evaluation models (EMs) and EM applications for the Emergency Core Cooling System (ECCS) at the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2, covering the period of March 17, 2012 through November 6, 2012.

#### Large Break Loss-of-Coolant Accident (LBLOCA) Evaluation Model

CHANGED CONDITION	PCT IMPACT (°F)
No changes or errors	0
Cumulative Impact	0

# Small Break Loss-of-Coolant Accident (SBLOCA) Evaluation Model

CHANGED CONDITION	PCT IMPACT (°F)
No changes or errors	0
Cumulative Impact	0

# H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

# PEAK CLADDING TEMPERATURE ESTIMATES

The current peak cladding temperature (PCT) estimates associated with Loss-of-Coolant Accident (LOCA) Emergency Core Cooling System (ECCS) evaluation models are listed below. These estimates include the cumulative effects of significant and non-significant error corrections and evaluation model changes through November 6, 2012.

<u>Event</u>	<u>PCT (°F)</u>
Large Break LOCA, ECCS Injection Mode	2098
Small Break LOCA, ECCS Injection Mode	1507